



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
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October 19, 2005

Mr. James Shetler, Assistant General Manager
Energy Supply
Sacramento Municipal Utility District
6201 'S' Street
P.O. Box 15830
Sacramento, California 95852

SUBJECT: NRC INSPECTION REPORT 050-00312/05-003

Dear Mr. Shetler:

An NRC inspection was conducted on September 27, 2005, at your Rancho Seco Nuclear Generating Station. At the conclusion of the site visit, a briefing was conducted with the acting Plant Manager and other members of your staff. The inspector reviewed and evaluated additional licensee documents submitted after the visit. These reviews and the inspection were completed on October 12, 2005. The enclosed report presents the scope and results of the inspection.

The inspection was an examination of activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection included reviews of the status of the reactor vessel internals segmentation; other decommissioning activities; and radioactive waste treatment, effluent and environmental monitoring. During this inspection, a violation of site procedures related to the entry of an individual into a secured high radiation area was reviewed. This non-repetitive, licensee-identified and corrected violation is being treated as a Non-Cited Violation, consistent with Section VI.A.8 of the NRC Enforcement Policy.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/Adams.html>. To the extent possible, your response should not include any personal privacy, proprietary, or safeguards information so that it can be made available to the public without redaction.

Should you have any questions concerning this inspection, please contact Mr. Emilio Garcia, Health Physicist, at (530) 756-3910 or the undersigned at (817) 860-8191.

Sincerely,
/RA/ J. V. Everett for

D. Blair Spitzberg, Ph.D., Chief
Fuel Cycle and Decommissioning Branch

Sacramento Municipal Utility District

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Docket No.: 050-00312

License No.: DPR-54

Enclosure:

NRC Inspection Report

050-00312/05-003

cc w/enclosure:

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ENCLOSURE

U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket Nos.: 050-00312
License Nos.: DPR-54
Report Nos.: 050-00312/05-003
Licensee: Sacramento Municipal Utility District
Facility: Rancho Seco Nuclear Generating Station
Location: 14440 Twin Cities Road
Herald, California
Dates: September 27, 2005
Inspector: Emilio M. Garcia, Health Physicist
Approved By: D. Blair Spitzberg, Ph.D., Chief
Fuel Cycle and Decommissioning Branch
Attachments: Supplemental Information
Partial List of Documents Reviewed

EXECUTIVE SUMMARY

Rancho Seco Nuclear Generating Station
NRC Inspection Reports 050-00312/05-003

The licensee was actively conducting dismantling activities in the reactor building, auxiliary building, spent fuel building and exterior areas. All spent fuel had been removed from the spent fuel pool and placed in the Independent Spent Fuel Storage Installation (ISFSI). The reactor vessel head, pressurizer, pressurizer drain tank and the two steam generators had been removed from the reactor building and shipped to an offsite disposal facility. Segmentation of reactor vessel internals was underway.

Organization, Management and Cost Controls

- The decommissioning group had been reorganized to accommodate the retirement of the previous manager and to focus the organization on the primary goals of completing activities related to major component dismantlement, site characterization, site remediation and the final radiological site surveys. The revised organization was consistent with the Rancho Seco Quality Manual (Section 1).

Decommissioning Performance and Status Review

- Dismantlement activities continued in the reactor building, auxiliary building, spent fuel building and other exterior areas. Problems with methods planned for the segmentation of the reactor vessel internals resulted in changes in techniques, work locations and schedule. As a result of the changes, the ALARA exposure estimate for the reactor vessel internals project was increased from 19 person-rem to slightly more than 30 person-rem (Section 2).

Occupational Radiation Exposure

- A violation of radiation safety procedures related to entry into a secured high radiation area was identified by the licensee and has been documented as a non-cited violation in this inspection report (Section 3).

Effluent and Environmental Monitoring

- Internal audits and assessments were being conducted of the effluent and environmental monitoring program by the licensee using performance-based audit techniques (Section 4.1).
- The licensee had appropriately revised the offsite dose calculation manual to incorporate changes resulting from the status of dismantling and decommissioning of the facility (Section 4.2).

- The required liquid effluent monitor was operational and in calibration. Although no longer required, the licensee was continuing to monitor air particulate effluents from the reactor building (Section 4.3).
- The Annual Radiological Environmental Operating Reports for calendar year 2004 was submitted on a timely basis and met applicable requirements. Radioactivity levels in the sampled media were consistent with previous years and were below the NRC required reportable levels (Section 4.4).
- The 2004 Annual Radioactive Effluent Release Report was submitted on a timely basis and met applicable requirements. Some minor typographical errors in the report were identified by the NRC inspector which will be documented in an errata sheet with the next annual report. The report documented that the releases of radioactivity in gaseous and liquid effluents in 2004 did not exceed applicable limits (Section 4.5).

Report Details

Summary of Facility Status

The Rancho Seco Nuclear Generating Station was permanently shut down in June 1989. All spent reactor fuel has been moved to an onsite Independent Spent Fuel Storage Installation (ISFSI). At the time of this inspection, the licensee was conducting decommissioning activities at the site. Decommissioning was being performed under the provisions of the incremental decommissioning option of Rancho Seco's Post Shutdown Decommissioning Activities Report, dated March 20, 1997.

Decommissioning work activities included the auxiliary building, reactor building, spent fuel building and exterior areas. All major components in the auxiliary building had been removed, packaged and shipped for disposal. In the reactor building, the major piping, the four reactor coolant pumps, the core flood tanks, reactor vessel head, pressurizer, pressurizer drain tank, and the two steam generators had been removed, packaged and shipped offsite for disposal. In the fuel handling building, the spent fuel pool water had been processed and released. Most of the pool liner plates had been cut, removed and shipped for disposal. During this inspection, the reactor vessel internals segmentation project was underway.

1 Organization, Management and Cost Controls (IP 36801)

a. Inspection Scope

The reorganization of the decommissioning group was reviewed with the acting Plant Manager.

b. Observations and Findings

The Project Manager, Decommissioning had announced his intention to retire at the end of calendar year 2005. This resulted in the licensee reorganizing the decommissioning organization to permit a period of transition prior to the departure of the current Project Manager, Decommissioning. Under the new organization, the position of Project Manager, Decommissioning was eliminated and replaced with two Dismantlement Superintendents reporting directly to the Manager, Plant Closure and Decommissioning. The Dismantlement Superintendent (Physical) was responsible for project controls and decommissioning operations. The Dismantlement Superintendent (Radiological) was responsible for radioactive waste, decommissioning radiological engineering, and specialty contractors. This revised organization permitted one superintendent to focus on the completion of major component dismantlement and the other superintendent to focus on radiological controls including the radioactive characterization of the site, re-remediation of residual radioactivity and the final status radiological surveys. Both individuals selected for these positions had many years of service at Rancho Seco with prior responsibilities in the areas assigned.

The inspector reviewed Rancho Seco's Administrative Procedure RSAP 0101, "Nuclear Organization Responsibilities and Authorities," and Procedure RSAP-0260, "Commitment Management Review Group and Commitment Tracking System."

Procedure RSAP-0101, had been revised to reflected the new organization. However, Procedure RSAP-0260, had not been revised to reflect the change in the decommissioning organization nor a recent change in the quality organization. The Quality/Licensing/Administration/Training Superintendent position had been replaced with the Supervising Quality Engineer. According to the Supervising Quality Engineer, the licensee was waiting to revise Procedure RSAP-0260 once management implemented the new company wide Sacramento Municipal Utility District (SMUD) commitment tracking system. The licensee projected that the new tracking system would be implemented by the end of October 2005.

The revised organization remained consistent with the Rancho Seco Quality Manual, Section I, Organization, which had been revised to reflect the change in the quality organization. The licensee had issued an updated Rancho Seco decommissioning organization chart to reflect the individuals assigned to each position. This organization chart was dated July 2005.

c. Conclusion

The decommissioning group had been reorganized to accommodate the retirement of the previous manager and to focus the organization on the primary goals of completing activities related to major component dismantlement, site characterization, site remediation and the final radiological site surveys. The revised organization was consistent with the Rancho Seco Quality Manual.

2 Decommissioning Performance and Status Review (IP 71801)

a. Inspection Scope

A tour of the site was conducted to observe work activities underway, including observation of housekeeping, safety practices, fire loading and radiological controls.

b. Observations and Findings

A tour of the reactor building, auxiliary building, spent fuel building and other areas of the plant was conducted to observe dismantling and decommissioning activities in progress. The work observed was being conducted in a safe and orderly manner. Radiological controls, including postings and barriers, were in place. Also noted were good housekeeping and fire protection practices in all areas.

The reactor vessel head, pressurizer, pressurizer relief tank and steam generators had been removed from the reactor building and shipped for disposal. This left only the reactor vessel and reactor internals as the last major components. The licensee was in the process of segmenting the reactor vessel internals to separate and package the irradiated components into the various waste categories.

The licensee experienced problems with the methods planned for the segmentation that necessitated changes in techniques, work locations and schedule. Inability of some of the cutting tools to perform as expected had resulted in the need to use other tools and

techniques. The most significant radiological aspect was the cutting of a portion of the reactor plenum while out of the water, increasing both direct exposures to personnel and the risk for discrete particle contamination. The original as low as reasonably achievable (ALARA) projected exposure for the project was slightly more than 19 person-rem. As a result of the changes, the new ALARA exposure estimate was slightly more than 30 person-rem. The project schedule completion had also been extended by several months. Originally the schedule called for completing the cutting activities by the end of August 2005. The new schedule projects completing cutting activities by the end of December 2005.

c. Conclusion

Dismantlement activities continued in the reactor building, auxiliary building, spent fuel building and other exterior areas. Problems with methods planned for the segmentation of the reactor vessel internals resulted in changes in techniques, work locations and schedule. As a result of the changes, the ALARA exposure estimate for the reactor vessel internals project was increased from 19 person-rem to slightly more than 30 person-rem.

3 Occupational Radiation Exposure (IP 83750)

a. Inspection Scope

On August 10, 2005, a worker entered a secured high radiation area without following required procedures. The inspector reviewed documents related to this event and interviewed individuals involved with the event and its followup investigation.

b. Observations and Findings

On August 10, 2005, while working in the reactor building, a contract employee failed to follow procedures during entry into a posted secured high radiation area to remove a drum of radioactive material. The individual entered the area alone and without conducting the required radiological survey upon entry. The individual's personal electronic dosimeter alarmed and a nearby radiation protection technician responded. The individual's electronic dosimeter was set to alarm at a dose rate of 80 millirem/hr. It was later noted that the electronic dosimeter had recorded a total of 34 millirem.

Entry requirements for a secured high radiation area are established by the licensee in the Radiation Control Manual Procedure RP 305.07 "Area Definitions, Posting, and Requirements." Steps 6.4.3 and 6.4.4 required a radiological survey prior to entry into a secured high radiation area and a second person to be present during the entry, who was responsible for providing radiological monitoring. A secured high radiation area is defined in Technical Specification 6.12.1b and Procedure RP 305.07 as an area with radiation levels of 1000 millirem/hr or greater at 30 centimeters in any accessible location.

The licensee documented their investigation of the unauthorized entry in a radiological problem occurrence report and a deviation from quality report # 05-0014. The licensee

determined that the area was correctly posted as a secured high radiation area. Proper instructions had been given to the worker by the radiation protection technician, but the worker had misunderstood the instruction. The licensee determined that the cause of this problem was lack of effective communications between the radiation protection technician and the worker. The licensee initiated remedial and preventative actions.

Technical Specification D6.0 states that the administrative control requirements are in the Rancho Seco Quality Manual. Section II of the Rancho Seco Quality Manual, states, in part, that radiation protection procedures and instructions shall be implemented as set forth in Section V of the Rancho Seco Quality Manual. Section V of the Rancho Seco Quality Manual states, in part, that activities affecting operational safety shall be described and implemented in accordance with documented procedures. Steps 6.4.3 and 6.4.4 of Procedure RP 305.07 "Area Definitions, Posting, and Requirements," required that an individual may not enter a posted secured high radiation area without the presence of a second individual whose sole responsibility will be to provide radiation monitoring and access control and the secured-high radiation area must be surveyed upon entry. Contrary to the above, on August 10, 2005, Procedure RP 305.07 was not implemented in that an individual entered a secured high radiation area without conducting a survey upon entry and a second individual was not present whose sole responsibility was to provide radiation monitoring. This failure was identified by the licensee and will be treated as a non-cited violation (NCV 050-00312/0503-01).

The inspector interviewed two individuals that were present during the incident and noted that there were pertinent details that were not part of the licensee's radiological problem occurrence report nor the deviation from quality report. These details related to the actions of the radiation protection technician and other individuals in the area. The inspector interviewed the principal individuals involved in the preparation of the radiological problem occurrence report and the deviation from quality report and determined that they were not aware of these additional details. The licensee initiated an independent assessment of the initial investigation. The results of this assessment will be reviewed during a future inspection (IFI 050-00312/0503-02).

c. Conclusions

A violation of radiation safety procedures related to entry into a secured high radiation area was identified by the licensee and has been documented as a non-cited violation in this inspection report.

4 Radioactive Waste Treatment, Effluent and Environmental Monitoring (IP 84750)

4.1 Audits and Surveillances

a. Inspection Scope

Selected audits and surveillances of the radiological effluent monitoring program were performed.

b. Observations and Findings

Audit Report 05-A-009 "REMP and QA Program for Effluent Control and Environmental Monitoring," issued on May 31, 2005, was reviewed to verify implementation of the commitments made in the Rancho Seco Quality Manual, Section XVIII "Audits" as it related to radioactive effluents and environmental monitoring. The audit was conducted on January 26 through March 9, 2005 by an individual who was qualified and authorized to perform the audit and was independent of the function being audited. The audit included an approved checklist, did not identify any item that constituted a potential deviation from quality and was conducted in a timely manner. Overall the quality of the audit was very good. The audit report included one recommendation for a revision to a procedure which was implemented by the licensee.

Surveillances 04-S-005 and 05-S-003 were conducted on February 12, 2004, and February 2-3, 2005, respectively. The surveillance reports were issued on February 12, 2004, and February 23, 2005, respectively. These surveillances incorporated performance-based audit techniques to review the Rancho Seco waste stream annual evaluations to verify their accuracy. The 2004 surveillance report included four recommendations to improve the document which were adopted and reflected in the 2005 evaluation. The 2005 surveillance report included five recommendations for improving report readability and to correct some minor errors. These recommendations were being adopted.

c. Conclusions

Internal audits and assessments were being conducted of the effluent and environmental monitoring program by the licensee using performance-based audit techniques.

4.2 Changes in the Offsite Dose Calculation Manual

a. Inspection Scope

The inspector discussed the changes to the Offsite Dose Calculation Manual (ODCM) with the plant chemistry specialist and reviewed the current ODCM.

b. Observations and Findings

Chemistry Administrative Procedure CAP-0002 "Offsite Dose Calculation Manual," contained the methodology and parameters used in the calculation of off-site doses due to radioactive gaseous and liquid effluents. This procedure was last revised January 27, 2005. This revision incorporated changes resulting from the status of dismantling and decommissioning of the facility. Changes included the deletion of the auxiliary building stack, deletion of the partition factor, revision of the minimum analysis frequency for reactor building air samples to monthly from weekly and the addition of the instrumentation and surveillance tables for the retention basin discharge flow rate instruments. These changes were reviewed and approved by the Commitment Management Review Group on January 26, 2005.

c. Conclusions

The licensee had appropriately revised the offsite dose calculation manual to incorporate changes resulting from the status of dismantling and decommissioning of the facility.

4.3 Process and Effluent Radiation Monitors

a. Inspection Scope

The inspector toured the locations of the effluent radiation monitors and discussed the monitors with cognizant licensee staff.

b. Observations and Findings

On September 27, 2005, the inspector verified that liquid effluent monitor R15017A was operational. This monitor was located in an enclosure protected from the elements. Records reviewed indicated that this monitor had last been calibrated on March 10, 2005. The last ODCM required quarterly testing of the alarms and discharge trip relay had been conducted on September 8, 2005. The calibration record and test record indicated that the instrument was within calibration and was functioning properly.

On September 27, 2005, the inspector observed that the reactor building gaseous effluent was being sampled by an air sampler. This air sampler was operational and stickers indicated it was in calibration. As noted in Inspection Report 50-312/2003-02, the licensee was no longer required to monitor the gaseous effluent pathway but had decided to continue sampling the air particulate effluents.

c. Conclusions

The required liquid effluent monitor was operational and in calibration. Although no longer required, the licensee was continuing to monitor air particulate effluents from the reactor building.

4.4 Annual Radiological Environmental Operating Report

a. Inspection Scope

The 2004 Annual Radiological Environmental Operating Reports was reviewed.

b. Observations and Findings

Step 1.5.2.3 of Appendix A to the Rancho Seco Quality Manual required that an Annual Radiological Environmental Operating Report covering the previous year be submitted to the NRC prior to May 1 of each year. On April 27, 2005, the licensee submitted the 2004 report. This report indicated that atmospheric, terrestrial and aquatic environments and the land use adjacent to Rancho Seco Nuclear Station were being

monitored. Radioactivity levels in the sampled media were consistent with previous years and were below the NRC required reportable levels. The report concluded that the operation of Rancho Seco Nuclear Station had no significant radiological impact on the environment.

c. Conclusions

The Annual Radiological Environmental Operating Reports for calendar year 2004 was submitted on a timely basis and met applicable requirements. Radioactivity levels in the sampled media were consistent with previous years and were below the NRC required reportable levels.

4.5 Annual Radioactive Effluent Release Report for 2004

a. Inspection Scope

The 2004 Annual Radioactive Effluent Release Report was reviewed.

b. Observations and Findings

Step 1.5.3 of Appendix A to the Rancho Seco Quality Manual required that an Annual Radioactive Effluent Release Report covering the previous 12 months be submitted to the NRC within 90 days of January 1 of each year. On March 21, 2005, the licensee submitted the 2004 Annual Radioactive Effluent Release Report on a timely basis. The report included summaries of radioactive gaseous and liquid releases from the site. The report concluded that the releases of radioactivity in gaseous and liquid effluents did not exceed the limits of 10 CFR 20 or the numerical guidelines of 10 CFR 50, Appendix I.

There were no unplanned gaseous releases during 2004. However, there was one unplanned liquid release that occurred in 2004. This unplanned release was due to a heavy rain storm on September 19, 2004. The plant experienced a loss of off-site power that led to a series of events causing multiple equipment failures. The multiple equipment failures resulted in all water entering the site from Folsom South Canal or Rancho Seco Lake to be diverted into the Retention Basins. Subsequently, the South Retention Basin overflowed.

Technical Requirement 6.12.3 of the licensee's Offsite Dose Calculation Manual identified effective dose commitment limits from liquid effluents to members of the public at or beyond the site boundary. These limits were based on the numerical guidelines of 10 CFR 50, Appendix I, which are 3 millirem per calendar year to the total body or 10 millirem to any organ. The 2004 annual calculated total effective dose due to liquid effluents, including the unplanned release, was 1.04E-02 millirem or approximately 0.35 percent of the applicable limit. The maximum calculated annual organ dose commitment was 2.34E-02 millirem or approximately 0.234 percent of the applicable limit.

Technical Requirement 6.12.7 of the licensee's Offsite Dose Calculation Manual specified effective dose commitment limits from gaseous effluents to members of the public at or beyond the site boundary. These limits were based on the numerical guidelines of 10 CFR 50, Appendix I, which for Tritium and radioactive material in particulate form with half-lives greater than 8 days are 7.5 millirem per calendar quarter to any organ and 15-millirem per calendar year to any organ. During 2004 there were no fission or activation gases airborne releases. The annual calculated dose at the site boundary due to tritium and particulates was 1.42E-02 millirem which is less than 0.1 percent of the annual limit.

During the review of the 2004 Annual Radioactive Effluent Release Report, the NRC inspector identified that the reported values for the "percent of annual dose limits" due to gaseous effluents in Table III-D were incorrect. It was concluded that when preparing the 2004 report, the 2003 report wordprocessor file was used as a template. Not all of the 2003 values were updated to the 2004 values. The incorrect values included in the 2004 report resulted in a conservative overestimate of the percent of the annual dose limit by a factor of five. The licensee agreed to review the report for other typographical errors and submit an errata sheet with next year's report. The corrective actions taken by the licensee will be reviewed during a future inspection (IFI 05000312/0503-03).

In 2004, there were 57 shipments of solid waste made. All solid waste shipments were transported by highway or rail. Forty-six of the shipments went to a licensed low-level radioactive waste disposal facility. Eleven shipments went to a licensed material recovery facility. Based on the information provided, the inspectors calculated that the total volume of waste shipped was 917.2 m³ with a total activity of 131.1 curies.

c. Conclusions

The 2004 Annual Radioactive Effluent Release Report was submitted on a timely basis and met applicable requirements. Some minor typographical errors in the report were identified by the NRC inspector which will be documented in an errata sheet with the next annual report. The report documented that the releases of radioactivity in gaseous and liquid effluents in 2004 did not exceed applicable limits.

5. Followup (IP 92701)

(Closed) IFI 050-00312/0403-01: Followup of Licensee's Actions to Correct ODCM Discrepancy for Particulate Sampling of Gaseous Effluents: During a previous inspection, the NRC inspector identified that the licensee had been analyzing the collection media for gaseous effluent continuous particulate samples on a bi-weekly frequency. The current Offsite Dose Calculation Manual (ODCM) specified that the collection media should be analyzed weekly. Historically, sampling analysis had always been conducted on a bi-weekly basis. A review of ODCM revisions determined that Revision 14 of the ODCM had apparently changed the sampling analysis frequency from monthly to weekly without explanation. Revision 17 of the ODCM will change the sampling requirement to a minimum of monthly. The basis for conducting monthly sampling instead of weekly or bi-weekly sampling was justified due to the significantly reduced amount of radioactive material currently onsite. This item is consider closed.

6 Exit Meeting Summary

The inspector presented the inspection results to the acting plant manager and other members of licensee staff at the exit meeting on September 27, 2005. The licensee did not identify as proprietary any information provided to, or reviewed by, the inspector.

ATTACHMENT 1

PARTIAL LIST OF PERSONS CONTACTED

Sacramento Municipal Utility District

M. Bua, Radiation Protection/Chemistry Superintendent
D. Gardner, Decommissioning Project Manager
R. Jones, Sr. Nuclear Engineer
S. Nicolls, Radiological Health Supervisor
L. Penafrancia, Plant Chemistry Specialist

Contractors

T. Garcia, Site Supervisor, Bigge Power Constructors
M. Steinbacher, Health Physics Site Coordinator, Bartlett Services, Inc.
C. Martens, Sr. Radiation Protection Technician, Bartlett Services, Inc.

INSPECTION PROCEDURES USED

IP 36801	Organization, Management and Cost Controls
IP 71801	Decommissioning Performance and Status Review
IP 83750	Occupational Radiation Exposure
IP 84750	Radioactive Waste Treatment, Effluent and Environmental Monitoring
IP 92701	Followup

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

050-00312/0503-01	NCV	Unauthorized Entry into a Secured High Radiation Area
050-00312/0503-02	IFI	Licensee's Independent Assessment of the Initial Investigation into the Unauthorized Entry into a Secured High Radiation Area.
050-00312/0503-03	IFI	Minor Errors in the 2004 Annual Radioactive Effluent Release Report

Closed

050-00312/0403-01	IFI	Followup of Licensee's Actions to Correct ODCM Discrepancy for Particulate Sampling of Gaseous Effluents.
050-00312/0503-01	NCV	Unauthorized Entry into a Secured High Radiation Area

Discussed

None

LIST OF ACRONYMS

ALARA	As Low As Reasonably Achievable
CAP	Chemistry Administrative Procedure
CFR	Code of Federal Regulations
DQ	Deviation from Quality
IFI	Inspection Follow-up Item
IP	Inspection Procedure
ISFSI	Independent Spent Fuel Storage Installation
PED	Personal Electronic Dosimeter
NCV	Non-Cited Violation
ODCM	Offsite Dose Calculation Manual
RP	Radiation Protection
RSAP	Rancho Seco Administrative Procedure

ATTACHMENT 2

PARTIAL LIST OF DOCUMENTS REVIEWED

Audits and Surveillances

- Rancho Seco Audit Report No. 05-A-009, REMP + QA Program for Effluent Control and Environmental Monitoring, Audit dates January 26 - March 9, 2005, report issued on May 31, 2005.
- Surveillance Report 04-S-005, Verify the accuracy of the Rancho Seco 2004 Waste Stream Evaluation, surveillance period February 12, 2004, issued February 12, 2004.
- Surveillance Report 05-S-003, Verify the accuracy of the Rancho Seco 2005 Waste Stream Evaluation, surveillance period February 2-3, 2005, issued February 23, 2005.

Data Sheets

- Surveillance Procedure Manual, SP-418A, Quarterly Test of Liquid Effluent Radiation Monitor (R-15017A), Revision 21, effective April 19, 2004. Surveillance approved March 9, 2005.
- Surveillance Procedure Manual, SP-418A, Quarterly Test of Liquid Effluent Radiation Monitor (R-15017A), Revision 21, effective April 19, 2004. Surveillance approved June 23, 2005.
- Surveillance Procedure Manual, SP-418A, Quarterly Test of Liquid Effluent Radiation Monitor (R-15017A), Revision 21, effective April 19, 2004. Surveillance approved September 9, 2005.
- Surveillance Procedure Manual, SP-488A, Refueling Interval Calibration of Liquid Effluent Radiation Monitor (R-15017A), Revision 16, effective March 9, 2005. Surveillance approved March 10, 2005.

Procedures

- Chemistry Administrative Procedures Manual CAP-0002, Offsite Dose Calculation Manual, Revision 17, effective January 27, 2005.
- Rancho Seco Administrative Procedure RSAP 0101, Nuclear Organization Responsibilities and Authorities, Revision 30, effective August 1, 2005.
- Rancho Seco Administrative Procedure RSAP-0260, Commitment Management Review Group and Commitment Tracking System, Revision 12, effective September 4, 2003.
- Rancho Seco Quality Manual RSQM-Policy, Quality Assurance Policy, Revision 8, effective August 4, 2005.
- Rancho Seco Quality Manual RSQM-Section I, Organization, Revision 12, effective August 4, 2005.

- Radwaste Control Manual RP.309.I.03, Radioactive Waste Classification and Waste Form, Revision 10, effective August 1, 2005.

Reports

- 2004 Annual Radiological Environmental Operating Report, transmitted under SMUD letter MPC&D 05-041, dated April 27, 2005.
- Deviation from Quality Report DQ # 05-0014, Individual entered Secured Hi-Rad area w/out RP coverage, final review September 19, 2005.
- Procedure Change Request and Approval, Procedure CAP-0002, Approved January 26, 2005.
- Radiological Protection Occurrence Report Number 2002-002, final review July 8, 2002.
- Radiological Protection Occurrence Report Number 2005-001, final review August 17, 2005.

Other Documents

- J. A. Jones, JAJ 02-13, Memorandum to Personnel File, Corrective Actions Taken, dated June 13, 2002.
- Radiation Work Permit 05-103, Dismantlement Tasks Performed Inside the Reactor Building, effective March 31, 2005.
- Watts Happening, September 26, 2005 issue.