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September 26, 2005 GO2-05-159

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555-0001

#### Subject: COLUMBIA GENERATING STATION, DOCKET NO. 50-397 LICENSEE EVENT REPORT NO. 2005-005-00

Dear Sir or Madam:

Transmitted herewith is Licensee Event Report No. 2005-005-00 for Columbia Generating Station. This report is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B) as an operation or condition prohibited by technical specifications. The enclosed report discusses items of reportability and corrective actions taken.

If you have any questions or require additional information, please contact Mr. GV Cullen at (509) 377-6105.

Respectfully

WS Oxenford Vice President, Technical Services Mail Drop PE04

Enclosure: Licensee Event Report 2005-005-00

cc: BS Mallett – NRC RIV BJ Benney – NRC NRR INPO Records Center NRC Sr. Resident Inspector – 988C (2) RN Sherman – BPA/1399 WA Horin – Winston & Strawn CE Johnson – NRC RIV/fax

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NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (6-2004) LICENSEE EVENT REPORT (LER) (See reverse for required number of							NC	APPROVED BY OMB NO. 3150-0104 EXPIRES 6-30-2007 Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (1-5 F52), U.S. Nuclear Regulatory Commission, Washington DC 20555-0001, or by internet e- mail to infocollect@mrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the						
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4. TITL Inope	4. TITLE Inoperable Standby Liquid Control Pump due to Incorrect Fuse													
<u>5. E</u>			6	LER NUM	BER	7. RF	PORT D	ATE	T	8. OTI	HER FACILIT	IES INVO	LVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIA NUMBER	L REV NO.	MONTH	DAY	YEAR	FACILITY	Y NAME DOCKET NUMBER			T NUMBER	
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9. OPERATING MODE 1 10. POWER LEVEL 100			1 20. 20 20 20 20 20 20 20 20 20 20 20 20 20	1. THIS RE 2201(b) 2201(d) 2203(a)(1) 2203(a)(2)(i 2203(a)(2)(i 2203(a)(2)(i 2203(a)(2)(i 2203(a)(2)(i 0.2203(a)(2)(i 0.2203(a)(2)(i)	PORT IS SI ) i) iii) iv) v) vi) 11	UBMITTE 20.2 20.2 20.2 50.3 50.3 50.4 50.7 50.7 50.7 2. LICENS	DPURSU 203(a)(3)( 203(a)(3)( 203(a)(4) 36(c)(1)(i)( 36(c)(1)(ii)( 36(c)(2) 46(a)(3)(ii) 73(a)(2)(i)( 73(a)(2)(i)( EE CON	JANT T (i) (ii) (A) (A) (A) (B) TACT F	TO THE REQUIREMENTS OF 10 CFR§: (Check   50.73(a)(2)(i)(C) 50.7   50.73(a)(2)(ii)(A) 50.7   50.73(a)(2)(ii)(B) 50.7   50.73(a)(2)(ii)(B) 50.7   50.73(a)(2)(ii)(B) 50.7   50.73(a)(2)(iii) 50.7   50.73(a)(2)(ii)(B) 50.7   50.73(a)(2)(ii)(A) 50.7   50.73(a)(2)(v)(A) 73.7   50.73(a)(2)(v)(B) 73.7   50.73(a)(2)(v)(C) OTH   50.73(a)(2)(v)(C) OTH   50.73(a)(2)(v)(D) Spec   in NE FOR THIS LEF				k all that apply) 73(a)(2)(vii) 73(a)(2)(viii)(A) 73(a)(2)(viii)(B) 73(a)(2)(ix)(A) 73(a)(2)(ix)(A) 73(a)(2)(x) 71(a)(4) 71(a)(5) HER AC Form 366A	
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ABSTF	ACT (I	imit to 14	100 spac	es, i.e., apr	proximately	15 single	e-spaced	typewr	itten lines	s)				

On July 28, 2005, an incorrect fuse was found to have been installed in the circuitry associated with Standby Liquid Control System Pump 1A. This condition resulted in the pump being inoperable for a period of approximately 22.5 days. The fuse was promptly replaced with the correct fuse and the pump was restored to an operable status. The cause of this event was an inadequate self verification process combined with a less than adequate implementing procedure for controlling the fuse replacement process. An extent of condition review has identified additional instances of incorrect fuses; however, no additional operability concerns have been identified. Long term corrective actions to prevent recurrence are being developed within the Energy Northwest Corrective Action Program. A risk assessment of this condition was performed and this condition was not safety significant. No recent similar events have been reported by Energy Northwest.

NRC FORM 366A (1-2001)	ICENSEE EVE	NT REPO	U.S. NUCLEA	R REGULATOR	RY COMMISSION
1. FACILITY NAME	2. DOCKET		6. LER NUMBER	3. PAGE	
Columbia Generating Station	05000397	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
	00000391		2005-005-00		2013
17. NARRATIVE (If more space is required, i	use additional copies c	of NRC Form 36	36A)		- <u></u>
PLANT CONDITION					

The plant was operating in Mode 1 at 100 percent power at the time of this event.

# EVENT DESCRIPTION

On July 6, 2005 preventive maintenance was performed on the motor control center [MCC] bucket for Standby Liquid Control (SLC) System [BR] Pump [P] 1A, which included fuse [FU] replacement. During the evolution, the existing time delay fuse was replaced with a standard quick blow fuse. On July 28, 2005 a subsequent review of the fuse control log identified that the replacement fuse was incorrect and a work request (WR) was generated to correct the condition. Operations reviewed this WR and questioned the effect of this condition on the operability of the pump. This concern was immediately investigated and it was determined this configuration did not meet the electrical design requirements of the pump. The pump was subsequently declared inoperable. The fuse was promptly replaced and the pump was restored to an operable status at 0137 hours on July 29, 2005. This condition resulted in the pump being inoperable for a period of approximately 22.5 days. Since this period of inoperability is longer than the seven day Completion Time provided in the Columbia Technical Specification 3.1.7 for one inoperable SLC subsystem, this condition is reported as a condition prohibited by the technical specifications pursuant to 10 CFR 50.73(a)(2)(i)(B).

IMMEDIATE CORRECTIVE ACTION

The SLC Pump 1A was restored to operable status on July 29, 2005 by replacing the fuse with the correct type.

The corresponding fuse in the opposite train, SLC Pump 1B, was verified to be the correct fuse.

Energy Northwest performed a review of the fuse replacements for the last two years.

CAUSE

The root cause of this event is an inadequate self verification process combined with a less than adequate implementing procedure for controlling the fuse replacement process.

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(1-2001)	

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### FURTHER CORRECTIVE ACTION

Interim action: Revise PPM 1.3.47 to require "exact for exact" fuse replacement (amps, voltage, and model/type). If an alternate fuse must be used, then a supervisor or appropriate engineering personnel must verify that the replacement fuse is the correct type.

Additional corrective actions will be taken as developed in PER 205-0502 of the Energy Northwest Corrective Action Program.

## ASSESSMENT OF SAFETY CONSEQUENCES

This event posed no threat to the health and safety of the public or plant personnel. A risk assessment was performed on this condition. For a condition of the SLC Pump 1A being inoperable for a period of 23 days, a delta core damage probability ( $\Delta$ CDP) of 1.78E-7 was calculated. The calculated  $\Delta$ CDP is well below the threshold of a safety significant condition and is therefore not safety significant.

The SLC System is not credited for the mitigation of design basis events. The SLC System satisfies the requirements of 10 CFR 50.62. Per 10 CFR 50.62, the purpose of the SLC System is to reduce the risk associated with a postulated anticipated transient without a scram. The SLC System was not designed to meet the single failure criteria. The remaining SLC Pump would have been capable of injecting the entire contents of the boric acid storage tank. This condition is not considered to be a safety system functional failure.

#### SIMILAR EVENTS

No recent similar events have been reported at Columbia. Following the identification of this condition, a review to assess the extent of condition was performed. Several conditions involving incorrect fuses were identified. None of these instances represented a reportable condition or an operability concern.

EIIS information denoted as [XX]