

September 28, 2005

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
San Onofre Nuclear Generating Station
P.O. Box 128
San Clemente, CA 92674-0128

SUBJECT: SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2 - UPCOMING
STEAM GENERATOR TUBE INSERVICE INSPECTION (TAC NO. MC8395)

Dear Mr. Ray:

Inservice inspections of steam generator (SG) tubes play a vital role in assuring that adequate structural integrity of the tubes is being maintained. As required by the plant Technical Specifications, reporting requirements range from submitting a special report, within 15 days following completion of each inservice inspection of SG tubes, that identifies the number of tubes plugged and/or repaired; to submitting a special report, within 12 months following completion of the inspection, that provides complete results of the SG tube inservice inspection. The special report containing the complete results shall include the following:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged and/or repaired.

A phone conference has been arranged with your staff to discuss results of the SG tube inspections to be conducted during the upcoming refueling outage for San Onofre Nuclear Generating Station, Unit 2. This phone call is currently not scheduled. It will occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Attached is a list of discussion points to facilitate this phone conference. The Nuclear Regulatory Commission (NRC) staff will document a brief summary of the conference call as well as any material that your staff has provided the NRC staff in support of the call.

Sincerely,

/RA/
Jack Donohew, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-361

Enclosure: List of Discussion Points

cc w/encl: See next page

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Jack Donohew, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

DISTRIBUTION

Docket No. 50-361

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ACCESSION NO: ML052690119

NRR-106

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STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SOUTHERN CALIFORNIA EDISON COMPANY

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

DOCKET NO. 50-361

The following discussion points have been prepared by the Nuclear Regulatory Commission (NRC) staff to facilitate the phone conference that has been arranged with Southern California Edison Company (the licensee) to discuss the results of the steam generator (SG) tube inspections to be conducted during the upcoming refueling outage for San Onofre Nuclear Generating Station, Unit 2. This phone call will occur toward the end of the planned SG tube inspection interval, but before the unit completes the inspections and repairs.

The NRC staff plans to document the conference call in a letter with a brief summary of what was discussed in the call that will include any material that is provided by the licensee in support of the call.

1. Discuss any trends in the amount of primary-to-secondary leakage observed during the recently completed cycle.
2. Discuss whether any secondary side pressure tests were performed during the outage and the associated results.
3. Discuss any exceptions taken to the industry guidelines.
4. For each steam generator, provide a description of the inspections performed including the areas examined and the probes used (e.g., dents/dings, sleeves, expansion-transition, U-bends with a rotating probe), the scope of the inspection (e.g., 100 percent of dents/dings greater than 5 volts and a 20 percent sample between 2 and 5 volts), and the expansion criteria. Also, discuss the extent of the rotating probe inspections performed in the portion of the tube below the expansion transition region (reference NRC Generic Letter 2004-01, "Requirements for Steam Generator Tube Inspections").
5. For each area examined (e.g., tube supports, dents/dings, sleeves, etc.), provide a summary of the number of indications identified to-date of each degradation mode (e.g., number of circumferential primary water stress corrosion cracking indications at the expansion transition). For the most significant indications in each area, provide an estimate of the severity of the indication (e.g., provide the voltage, depth, and length of the indication). In particular, address whether tube integrity (structural and accident induced leakage integrity) was maintained during the previous operating cycle. In addition, discuss whether any location exhibited a degradation mode that had not previously been observed at this location at this unit (e.g., observed circumferential primary water stress corrosion cracking at the expansion transition for the first time at this unit).

6. Describe repair/plugging plans.
7. Describe in-situ pressure test and tube pull plans and results (as applicable and if available).
8. Provide the schedule for steam generator-related activities during the remainder of the current outage.
9. Discuss the following regarding loose parts:
 - What inspections are performed to detect loose parts?
 - A description of any loose parts detected and their location within the SG
 - If the loose parts were removed from the SG
 - Indications of tube damage associated with the loose parts
 - The source or nature of the loose parts, if known

San Onofre Nuclear Generating Station
Units 2 and 3

cc:

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August 2005