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August 30, 2005

10 CFR 50.73

2130-05-20163

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555 - 0001

Oyster Creek Generating Station
Facility Operating License No. DPR-16
NRC Docket No. 50-219

Subject: Correction of Licensee Event Report Dates for LER 2005-002-00 and
LER 2005-001-01

It has come to our attention that the Report Date shown in Block 7 on page 1 of each of the subject Licensee Event Reports is inconsistent with the date the reports were submitted to the NRC. A corrected page 1 for each LER is enclosed. There were no other changes to the LERs or to the commitments made in the LERs.

Any confusion caused by this error is regretted. If any further information or assistance is needed, please contact William Stewart at 609-971-4775.

Sincerely,



C. N. Swenson
Vice President, Oyster Creek Generating Station

CNS/WVS

Enclosure: NRC Form 366, LER 2005-002-00, page 1
NRC Form 366, LER 2005-001-01, page 1

cc: S. J. Collins, Administrator, USNRC Region I
P. S. Tam, USNRC Senior Project Manager, Oyster Creek
R. J. Summers, USNRC Senior Resident Inspector, Oyster Creek
Files 05041, 05030, and 05011

JE22

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by Internet e-mail to infocollect@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME Oyster Creek, Unit 1	2. DOCKET NUMBER 05000 219	3. PAGE 1 OF 3
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4. TITLE Actuation of Reactor Protection System Due to An Anticipatory Generator Load Reject Caused by Faulted Lightning Arrestors in a Local Sub-Station
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5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
06	01	2005	2005	002	00	07	29	2005	FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE N	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)									
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)						
10. POWER LEVEL 100	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> Specify in Abstract below or in NRC Form 366A						

12. LICENSEE CONTACT FOR THIS LER	
FACILITY NAME Robin Brown, Operations Support Manager	TELEPHONE NUMBER (Include Area Code) (609) 971-4979

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED					15. EXPECTED SUBMISSION DATE		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)					<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)
<p>On June 1, 2005, at 21:09, with Oyster Creek at 100% power, an Anticipatory Generator Load Reject scram occurred. During restoration of a transformer by the transmission utility at their substation, a failure of lightning arrestors resulted in a phase-to-phase-to-ground short circuit. This resulted in a grid transient of sufficient magnitude that the Oyster Creek Turbine-Generator sensed a load rejection condition, which resulted in a reactor scram signal. The reactor scrambled and the turbine-generator tripped as expected for this condition. All safety systems performed as expected. The plant was stabilized in the hot shutdown mode.</p> <p>Corrective actions included completing restart required evaluations, testing and confirmation from the involved transmission utility that conditions in the substation would not result in recurrence of the grid disturbance.</p> <p>The apparent cause of this event was equipment failure of the lightning arrestors in the transmission utility substation that created a grid disturbance.</p> <p>There have been several grid disturbances over the life of the plant, including two LERs: LER 2003-003, Actuation of Reactor Protection System Due to a Grid Transient (August 14, 2003). LER 1994-007 was a reactor scram caused by a 230 KV bus section differential relay trip while a switchyard worker was installing a Digital Fault Recorder.</p>

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME Oyster Creek, Unit 1	2. DOCKET NUMBER 05000 219	3. PAGE 1 OF 3
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4. TITLE Supplemental Report to "A" Control Rod Drive Pump Was Returned to Service Prior to Correcting the Cause of Failure Resulting in a Technical Specification Violation
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5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
03	18	2005	2005	001	01	08	04	2005	FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE N	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)									
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
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	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
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10. POWER LEVEL 100	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input checked="" type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A						

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CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED					15. EXPECTED SUBMISSION DATE			MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)					<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

During a functional test of the 1A2 Local Shutdown Panel (LSP) at 1254 on February 17, 2005, the "A" Control Rod Drive (CRD) Pump failed to start from the Main Control Room (MCR). The cause of this failure was attributed to contact high resistance on relay TR-2 in the 1A2 LSP. It was concluded that the high resistance cleared itself during subsequent relay operation. The relay was then scheduled for future replacement. "A" CRD Pump was successfully surveilled and returned to service. On March 16, 2005 at 1834, during a regular monthly surveillance, "A" CRD Pump did not start. Subsequent investigation found the 480 VAC breaker-closing spring was not charged. The closing spring is recharged by an electric motor immediately after the breaker opens. On March 18, 2005, investigation revealed a loose terminal wire connection to relay TR-2 within the LSP, which prevented charging of the closing spring. It is believed this condition existed on February 17, 2005 and should have been corrected. Technical Specifications (TS) only allow a 7-day out of service time for CRD Pumps resulting in a violation of TS 3.4.D.

Corrective actions included replacing the relay, sending the relay out for failure analysis and performing verification of closing spring condition on all safety related 480 VAC breakers.

This Supplemental LER is based on determination of the root causes to be a manufacturing deficiency consisting of a loose connection in the LSP, which caused the closing spring to not recharge, and a failure to detect that the closing spring was discharged prior to exceeding the Tech Spec out of service allotment.

There were no previous similar events at Oyster Creek Generating Station involving a breaker failing to close on demand due to the closing spring being discharged.