

September 1, 2005

LICENSEE: Nuclear Management Company, LLC  
FACILITY: Monticello Nuclear Generating Plant  
SUBJECT: SUMMARY OF A TELEPHONE CONFERENCE CALL HELD ON AUGUST 10, 2005, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC (NMC) TO DISCUSS FOLLOW-UP ITEMS REGARDING THE SEVERE ACCIDENT MITIGATION ALTERNATIVES (SAMA) REQUEST FOR ADDITIONAL INFORMATION (RAI) RESPONSES FOR MONTICELLO NUCLEAR GENERATING PLANT (TAC NO. MC6441)

On August 10, 2005, the U.S. Nuclear Regulatory Commission (NRC or the staff) and its contractor from Information Systems Laboratory (ISL) conducted a conference call (teleconference) with representatives from NMC to discuss the Monticello Nuclear Generating Plant (Monticello) SAMA RAI responses (see ADAMS Accession No. ML052130197). The SAMA RAI responses were submitted to the NRC on July 27, 2005. The teleconference was held to clarify these RAI responses. Enclosure 1 provides a listing of the teleconference participants.

The NRC staff asked for clarification regarding RAI 1.d, 1.e, 2.b, and 5.b. NMC staff provided information to help clear up those questions. No staff decisions were made during the teleconference. Enclosure 2 contains an email response from NMC clarifying their response to RAI 5.b.

**/RA/**

Jennifer Davis, Project Manager  
Environmental Section  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No.: 50-263

Enclosures: As stated

cc w/encls: See next page

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OFFICE	PM:RLEP:DRIP	LA:RLEP:DRIP	SC:RLEP:DRIP
NAME	JDavis	YEdmonds	RFranovich
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D. Matthews/F. Gillespie (RidsNrrDrip)

P.T. Kuo (RidsNrrDripRlep)

R. Franovich

J. Davis

D. Merzke

A. Hodgdon, OGC

L. M. Padovan

S. Imboden

R. Schaaf

J. Flemming

A. Stone, RIII

B. Burgess, RIII

J. Adams, RIII

R. Orlikowski, RIII

P. Lougheed, RIII

Jan Strasma, OPA

RLEP R/F

C. Quinly (LLNL)

**LIST OF PARTICIPANTS  
TELECONFERENCE WITH  
NUCLEAR MANAGEMENT COMPANY, LLC  
AUGUST 10, 2005**

**Names**

Jim Holthaus  
Doug Johnson  
Britta Johnson  
Anthony Sarrack  
Rick Loeffler  
Steve Callis  
Jennifer Davis  
Robert Palla  
Jason Flemming  
Robert Schmidt

**Affiliations**

Nuclear Management Company, LLC (NMC)  
NMC  
NMC  
NMC  
NMC  
NMC  
U.S. Nuclear Regulatory Commission (NRC)  
NRC  
NRC  
Information Systems Laboratory

**From:** "Holthaus, James J." <James.Holthaus@nmcco.com>  
**To:** <JXD10@nrc.gov>  
**Date:** 8/12/05 10:28AM  
**Subject:** Supplement SAMA RAI 5b

To supplement Response to Request for Additional Information Regarding Severe Accident Mitigation Alternates for Monticello Nuclear Generating Plant (TAC NO. MC6441), dated July 27, 2005 (Accession Number ML052130197) and telecon on August 10, 2005.

NMC provides herein clarifying information for our response to NRC SAMA RAI 5.b regarding SAMA 37 (Manual Reactor Core Isolation Cooling (RCIC) Operation) and combined impact of recommended SAMAs:

Given a prolonged station blackout (SBO), SAMA 37 (Manual RCIC Operation) alone does not provide a success path for preventing core melt, because containment heat removal and containment venting are both unavailable. This eventually results in containment rupture due to overpressure, precluding operators from occupying the RCIC room to support manual operation of the system. In addition, RCIC would eventually deplete the condensate storage tank (CST) inventory and would overheat if torus water (saturated conditions) was used as the suction source.

For these reasons, RCIC only shifts risk dominant sequences from leading to core melt in 6 hours (2 hours after batteries deplete) to 20 hours (a few hours after containment ruptures or CST inventory is depleted). This has competing effects on risk. The delay allows an increased probability of recovering offsite power or power from an emergency diesel generator, which reduce risk. At the same time, it also increases risk because the additional time allows containment to heat up and pressurize.

The risk reduction gained by allowing an increased probability of power recovery (ending SBO) is far outweighed by the increase in risk due to shifting to more severe consequences of failing a pressurized reactor vessel into a pressurized containment at saturated conditions. However, SAMA 37 is highly effective at mitigating risk if other supporting SAMAs are also implemented to assure its viability and reliability. Those supporting SAMAs are summarized as follows:

SAMA 28 (Refill CST) establishes the use of the fire protection system to ensures RCIC has a long-term suction source.

SAMA 12 (Additional Diesel Fire Pump for Fire Service Water System) improves the reliability of the fire protection system, especially during SBO scenarios. This dramatically improves the suction source reliability for RCIC since CST is filled from fire protection system).

SAMA 2 (Enhanced DC Power Availability) provides power to solenoid valves, allowing containment to be vented during prolonged SBO

scenarios, preventing RCIC failure.  
SAMA 36 (Divert Water from Turbine Building 931-Foot Elevation East)  
improves RCIC reliability by preventing flooding of supporting batteries  
before enough time is available to establish manual RCIC operation.

If you have any questions regarding this information please contact Jim  
Holthaus, Monticello License Renewal, at (763) 295-1309.

James J. Holthaus

Monticello Nuclear Generating Plant License Renewal

Environmental Lead

Nuclear Management Company

2807 West County Road 75

Monticello, Minnesota 55362-9637

**CC:** <RLP3@nrc.gov>, <erschmidt@adelphia.net>, "Musolf, David M."  
<David.Musolf@nmcco.com>, "Burke, Patrick B." <Patrick.Burke@nmcco.com>, "Sarrack,  
Anthony G." <Anthony.Sarrack@nmcco.com>

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**From:** "Holthaus, James J." <James.Holthaus@nmcco.com>

**Created By:** James.Holthaus@nmcco.com

**Recipients**

nrc.gov  
TWGWPO01.HQGWD001  
RLP3 CC (Robert Palla)

nrc.gov  
owf4\_po.OWFN\_DO  
JXD10 (Jennifer Davis)

nmcco.com  
Anthony.Sarrack CC (Anthony G. Sarrack)  
Patrick.Burke CC (Patrick B. Burke)  
David.Musolf CC (David M. Musolf)

adelphia.net  
erschmidt CC

**Post Office**

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owf4\_po.OWFN\_DO

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Monticello Nuclear Generating Plant

cc:

Jonathan Rogoff, Esquire  
Vice President, Counsel & Secretary  
Nuclear Management Company, LLC  
700 First Street  
Hudson, WI 54016

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
2807 W. County Road 75  
Monticello, MN 55362

Manager, Regulatory Affairs  
Monticello Nuclear Generating Plant  
Nuclear Management Company, LLC  
2807 West County Road 75  
Monticello, MN 55362-9637

Robert Nelson, President  
Minnesota Environmental Control  
Citizens Association (MECCA)  
1051 South McKnight Road  
St. Paul, MN 55119

Commissioner  
Minnesota Pollution Control Agency  
520 Lafayette Road  
St. Paul, MN 55155-4194

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
801 Warrenville Road  
Lisle, IL 60532-4351

Commissioner  
Minnesota Department of Health  
717 Delaware Street, S. E.  
Minneapolis, MN 55440

Douglas M. Gruber, Auditor/Treasurer  
Wright County Government Center  
10 NW Second Street  
Buffalo, MN 55313

Margo Askin  
Head Librarian, Monticello Public Library  
200 W. 6<sup>th</sup> Street  
Monticello, MN 55362

Commissioner  
Minnesota Department of Commerce  
85 7th Place East, Suite 500  
St. Paul, MN 55101-2198

Manager - Environmental Protection  
Division  
Minnesota Attorney General's Office  
445 Minnesota St., Suite 900  
St. Paul, MN 55101-2127

John Paul Cowan  
Executive Vice President & Chief Nuclear  
Officer  
Nuclear Management Company, LLC  
700 First Street  
Hudson, WI 54016

Nuclear Asset Manager  
Xcel Energy, Inc.  
414 Nicollet Mall, R.S. 8  
Minneapolis, MN 55401

Mr. James Ross  
Nuclear Energy Institute  
1776 I Street, NW, Suite 400  
Washington, DC 20006-3708

Patrick Burke  
License Renewal Project Manager  
Monticello Nuclear Generating Plant  
Nuclear Management Company, LLC  
2807 West County Road 75  
Monticello, MN 55362-9637

Mr. Douglas F. Johnson  
Director, Plant Life Cycle Issues  
Nuclear Management Company, LLC  
700 First Street  
Hudson, WI 54016

Amy Wittmann  
Branch Librarian, Buffalo Public Library  
18 Northwest Lake Boulevard  
Buffalo, MN 55313