



June 23, 2005

NRC-05-081 10 CFR 50.55a

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Kewaunee Nuclear Power Plant Docket 50-305 License No. DPR-43

Inservice Inspection Third 10-Year Interval June 16, 1994 - June 16, 2004 Limitation
To Examination Relief Requests of the Kewaunee Nuclear Power Plant

Kewaunee Nuclear Power Plant (KNPP) is required to have an Inservice Inspection (ISI) Program for examination of American Society of Mechanical Engineers (ASME) Class 1, 2, and 3 systems and their supports. During the third Inservice Inspection interval, ISI examinations were performed on the required number of components in accordance with 10 CFR 50.55a and ASME Section XI, 1989 Edition with no Addenda.

On June 16, 2005, KNPP ended its third interval ISI Program (incorporating the ASME Boiler and Pressure Vessel Code Section XI 1989 Edition Section IWA-2430(d) One Year Extension).

All examinations were completed by November 30, 2004. The required examinations were conducted in a manner that met all Section XI Code requirements to the extent practical. There were areas where the Code required coverage could not be obtained. Where possible, KNPP substituted other welds and supports to avoid those situations where complete Code compliance could not be attained. In some cases, KNPP was successful with finding an alternate area where full Code compliance could be achieved. This Request for Relief details those areas where the Code required examination criteria could not be completely met, and to what extent Code compliance was achieved.

A047

Document Control Desk Page 2

Photographs showing the Examination Limitations in most cases were taken well after the actual performance of the examinations. This was due to improved technology from the start of the third Ten-Year Interval, the desire to document the limitation in addition to the required procedure sketch, and to supply an actual photograph of the limitation as a process improvement for better understanding of the restriction. Nuclear Management Company, LLC requests relief in accordance with 10 CFR 50.55a(g)(5)(iii), as these areas either could not be examined in accordance with Section XI Code requirements or without significant modifications to the plant.

Michael G. Gaffney

Site Vice President, Kewaunee Nuclear Power Plant

Nuclear Management Company, LLC

Enclosure (1)

cc: Administrator, Region III, USNRC

Project Manager, Kewaunee Nuclear Power Plant, USNRC

Resident Inspector, Kewaunee, USNRC

Mr. Mike Verhagen, Department of Commerce, State of Wisconsin

ENCLOSURE 1

INSERVICE INSPECTION THIRD 10-YEAR INTERVAL
JUNE 16, 1994 - JUNE 16, 2004 LIMITATION TO EXAMINATION RELIEF
REQUESTS OF THE KEWAUNEE NUCLEAR POWER PLANT

Component Identification

Code Class: 1, 2

Examination Categories: Reference attached RR-G-7-1 through RR-G-7-73

Item Numbers: Reference attached RR-G-7-1 through RR-G-7-73

Description: Limited Section XI Code examinations during Code required examinations

Code Requirements

Third Inservice Inspection Interval

1989 Edition of Section XI, no Addenda

ASME Code Case N-460, Alternative Examination Coverage for Class 1 and 2 Welds

Volumetric and surface examinations of welds and base material will be examined in accordance with the applicable Examination Category and Item Number.

Basis for Relief

During the third inservice inspection interval, Kewaunee Nuclear Power Plant (KNPP) used the requirements of the 1986 Edition of ASME Section XI for examination of components. This edition of Section XI has specific requirements for what components will be examined and to what extent they will be examined. This edition of Section XI was followed with the exception of the selection of Class 1 piping welds. In accordance with 10 CFR 50.55a(b)(2)(ii), KNPP used the 1974 Edition of Section XI for selection of Class 1 Category B-J piping welds. This selection criteria states that for each inservice inspection interval, a different 25% of the population of class 1 piping welds shall be examined. Since this was the third 10-year interval, essentially 50% of the Class 1 piping welds had already been examined, leaving only 50% of the total Class 1 welds available for selection and examination. This factor led to many of the welds being examined having a higher probability of having limited examinations. For all other Examination Categories, KNPP used the Section XI required examination selection criteria.

During the performance of scheduled examination, there were numerous instances where examiners reported some type of interference. KNPP personnel evaluated every instance where this situation was reported, and when possible, alternative examination areas were selected to avoid having to examine restricted areas. This reduced the population of welds where a limited examination would be encountered. On welds where alternatives were not available, additional techniques were performed to increase coverage where possible, such as using steeper angle beam ultrasonic techniques or approaching the examination area from a different direction. In each case, this increased examination area coverage to the extent practicable and reduced the number of welds with limited examinations.

In order to gain additional access to the areas where limited examinations were encountered, major modification of components would be required. These modifications could be extensive, up to and including complete replacement. In cases where minor grinding would allow additional coverage, this was performed.

Major modification of components is not a feasible approach nor is it required to obtain additional coverage. Modifying systems and components to improve examination area coverage would result in additional dose with marginal improvement in quality or safety.

10 CFR 50.55a(g)(1) states that plants of KNPP's vintage (construction permit before January 1, 1971) must meet the requirements, except design and access provisions and preservice examination requirements, set forth in ASME Section XI. Access has been improved over the years to many areas of the plants, further reducing those areas with limitations, but it is not a requirement to make every area available for examination.

Removal of structural interferences to gain additional examination area coverage is costly in time and dose with little impact to quality or safety. Structural steel and component supports, were not designed for removal. If any were removed, temporary support structures would need to be designed and installed to compensate for the reduction in load bearing capacity. Temporary modifications, such as these, could require extensive scaffold building and temporary structural components, resulting in additional accumulated radiation dose.

For volumetric examinations, KNPP examined the required areas to the extent practical using ultrasonic examination techniques. Since May 22, 2000, many of these components were examined using personnel qualified in accordance with Appendix VIII of Section XI and as implemented by the Performance Demonstration Initiative (PDI). All examinations performed prior to the implementation of the 10 CFR 50.55a(g)(6)(i)(C) Appendix VIII requirements, show the limitations of the 1989 Edition of Section XI. All examinations performed after the implementation of the Appendix VIII requirements show limitations based on component configuration, material, or limitations in the PDI procedures being used. Where possible and allowed by the applicable PDI Generic Procedure, additional angles were used to increase coverage. In many cases, no combination of ultrasonic angle beam examinations would cover the entire examination area. In each case, the maximum feasible coverage was obtained.

KNPP performs system leakage tests in accordance with the Pressure test requirements of Section XI, Examination Categories B-P and C-H. These pressure tests cover every component within the Code boundaries established by KNPP. Where leakage was noted at mechanical connections, these were corrected in accordance with maintenance procedures.

Proposed Alternative

KNPP proposes to use the examination volume or surface coverage obtained during the third interval examinations on the listed components in lieu of the Code required volumes and surfaces (Reference RR-G-7-1 through RR-G-7-73 (see attachments). The coverage obtained meets the intent of ASME Section XI and provides an acceptable level of quality and safety.

Conclusion

The examinations performed during the third inservice inspection interval were performed to the extent possible. Additional coverage was impractical, as modification of systems, structures, and components would have resulted in significant radiation exposure with minimal increase in the level of quality and safety.

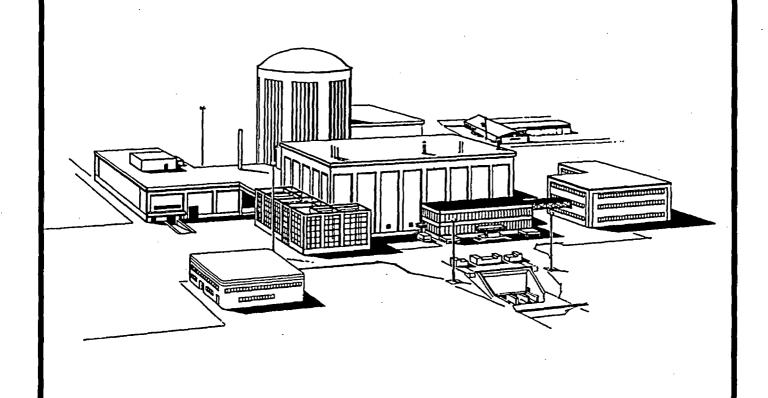
Period for Which Relief is Requested

Relief is requested for the third inservice inspection interval at KNPP, which ended on June 16, 2005 incorporating the ASME Boiler and Pressure Vessel Code Section XI 1989 Edition Section IWA-2430(d) One Year Extension.

Attachments

RR-G-7-1 through RR-G-7-73 – Listing and sketches of Limited Examinations for the Third Inservice Inspection Interval

INSERVICE INSPECTION
3RD 10-YEAR INTERVAL
JUNE 16, 1994 – JUNE 16, 2004
LIMITATION TO EXAMINATION
RELIEF REQUESTS
OF THE
KEWAUNEE NUCLEAR POWER PLANT
N490 HIGHWAY 42
KEWAUNEE, WISCONSIN 54216-9511



INSERVICE INSPECTION

3RD 10-YEAR INTERVAL

JUNE 16, 1994 – JUNE 16, 2004

LIMITATION TO EXAMINATION

RELIEF REQUESTS

OF THE

KEWAUNEE NUCLEAR POWER PLANT

N490 HIGHWAY 42

KEWAUNEE, WISCONSIN 54216-9511

PREPARED BY:

KEWAUNEE NUCLEAR POWER PLANT

ENGINEERING PROGRAMS

N490 HIGHWAY 42

KEWAUNEE, WISCONSIN 54216-9511

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-1

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

INSERVICE INSPECTION IMPRACTICALITY

- 1. ASME Code Component Affected: Integrally Welded Attachment
 AHRS1-SW1 and AHRS1-SW2
 to Residual Heat Exchanger AHRS1-1A
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-C; Item No. C3.10
- 4. Impracticality of Compliance: 20.7% of the Integrally Welded Attachments were inaccessible due to configuration of the Welded Attachment in close proximity to the Residual Heat Exchanger thus restricting application of the Surface Examination Liquid Penetrant.
- 5. Burden Caused by Compliance: To provide for access to the 20.7% weld area between the welded attachment and the Residual Heat Exchanger would require modification from the Original Design of the Residual Heat Exchanger and the Welded Attachment.

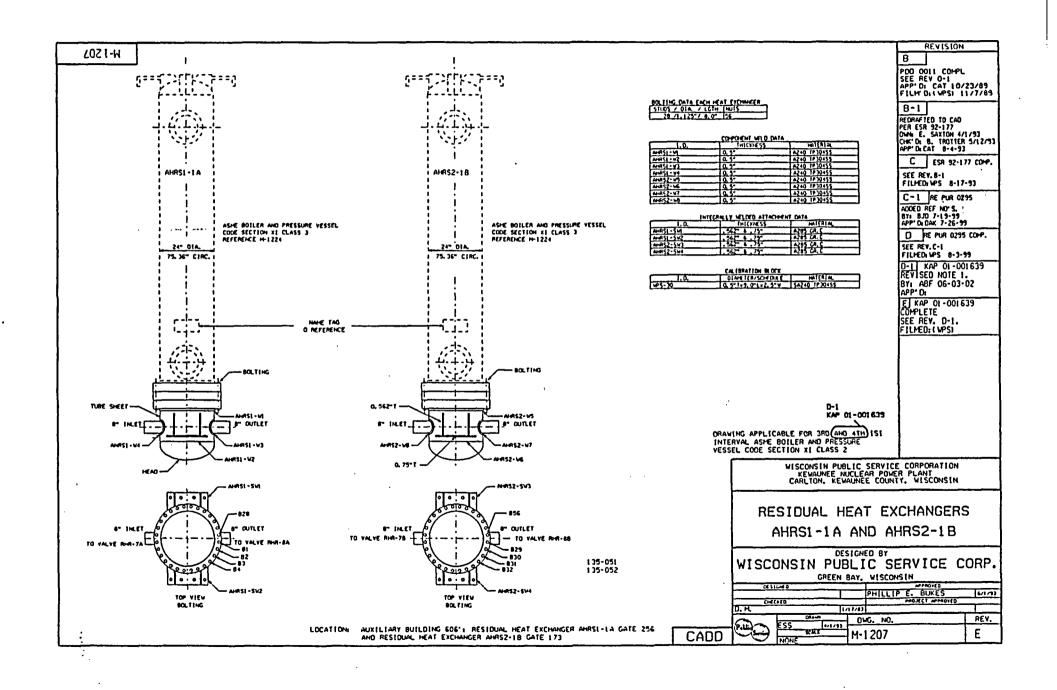
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-1

- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface examination is available due to limited access. VT-2 and VT-3 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: VT-2 Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage and VT-3 as required by Table IWF-2500-1; Examination Category F-A; Item No. F1.40B for general mechanical and structural condition.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

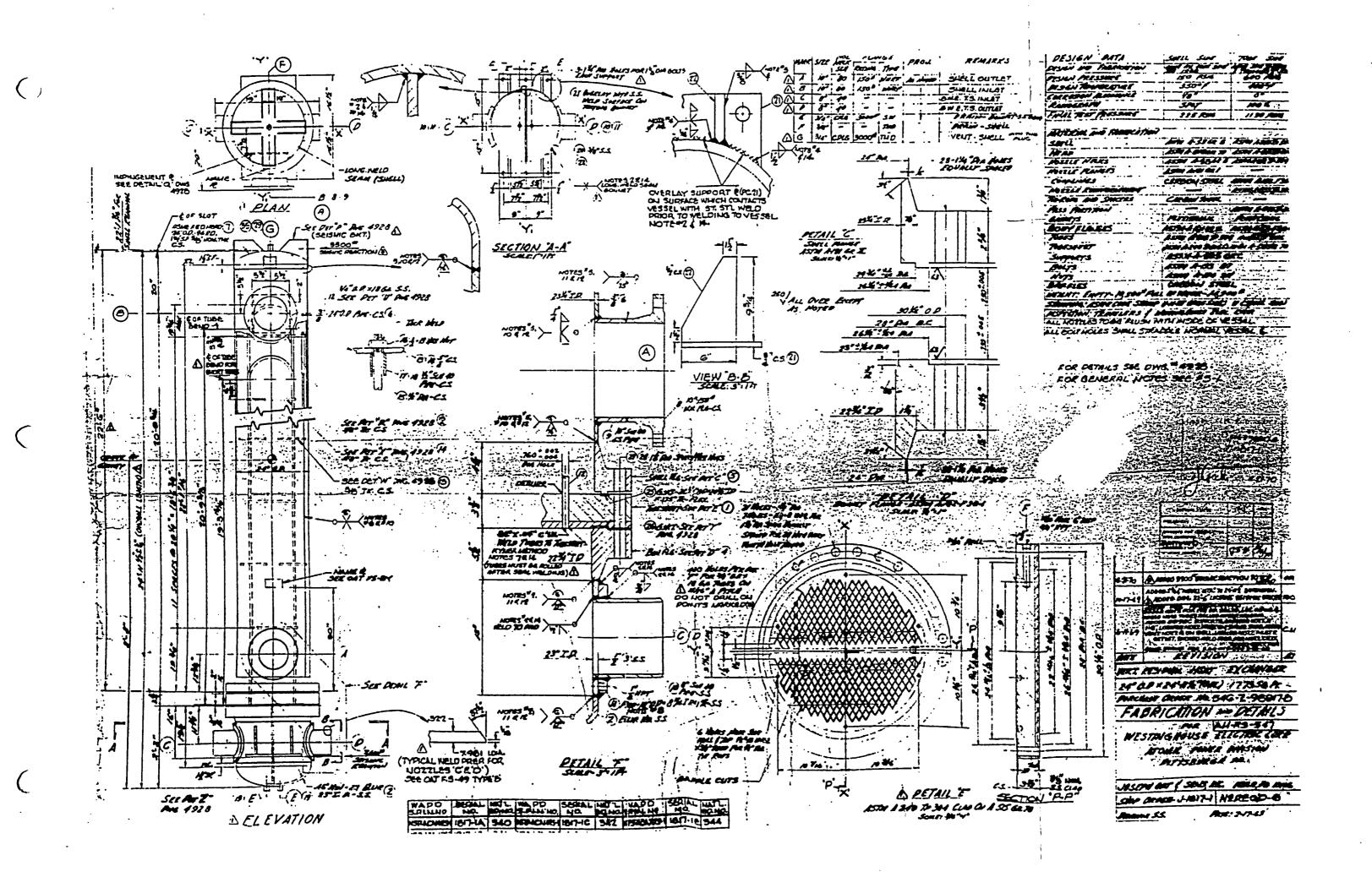
8. Precedents: Not Applicable

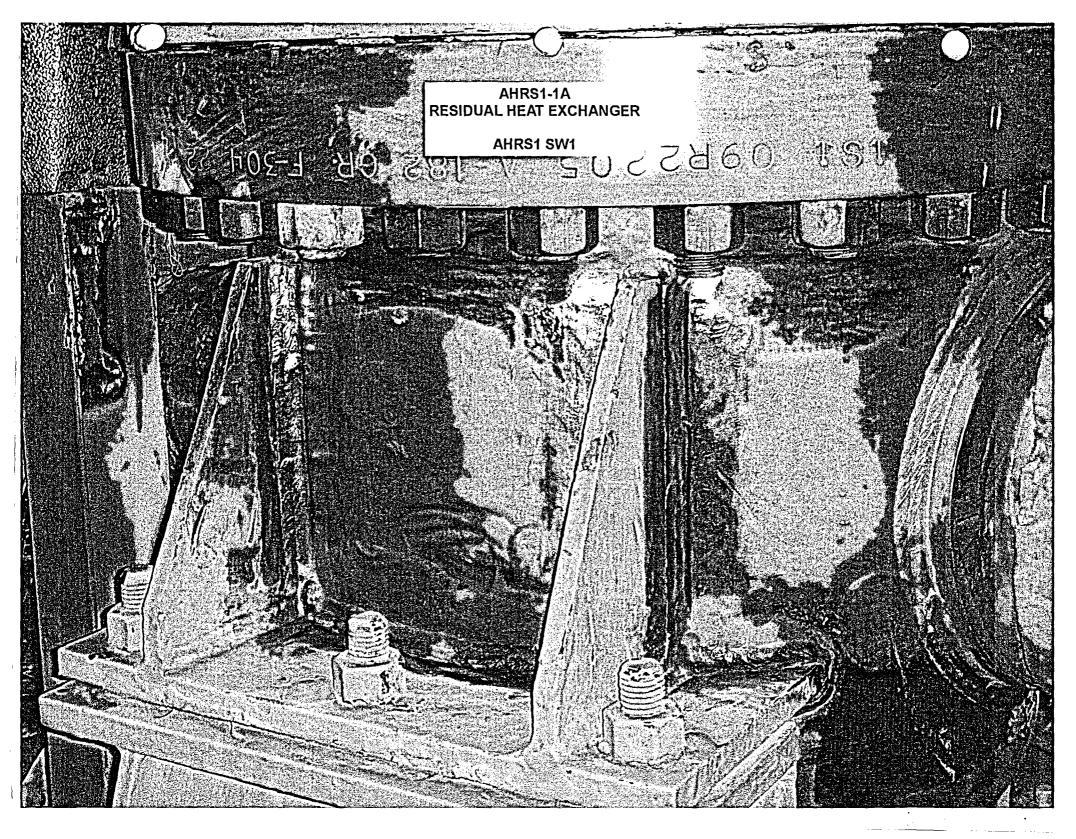
9. References: Not Applicable



TIOURE EXAMINATION EMINIMATION TO EXAMINATION TEOURD				
Residual Heat Exchangers SYSTEM OR COMPONENT: AHRS1-1A AND AHRS2-1B DRAWING NO.: M-1267				
COMPONENT IDENTIFICATION: AHRS1 _ Sw1 PROCEDURE: QCP_961 REVISION: ORTG.				
ULTRASONIC: LIQUID PENETRANT:X_ MAGNETIC PARTICLE: VISUAL:				
EXAMINER: NAM. By DATE: 4.25.95				
EXAMINER: NA NA DATE: NA LEVEL				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
OUTLET VIEW A AHRS1-SW1 INACCESSABLE SecTion OF WELD Behind Support Leg. PERCENTAGE OF REDUCED EXAMINATION CONCEAGE: 20.7% WELD WELD				
KEWAUNEE NUCLEAR : POWER PLANT REVIEW: 5 A Balata DATE: 4/27/95				
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogn Trif- DATE: 4/27/92				

SYSTEM OR COMPONENT: RESIDUAL HERT EXCHANGER RHRS1-1A AND AHRS2-1B				
DRAWING NO.:				
COMPONENT IDENTIFICATION: AHRS	1-SW2_PROCEDURE:	NEP NO. 15.6 REVISION: OR 1G.		
ULTRASONIC: LIQUID PENETR	ANT: X MAGNETIC PA	RTICLE: VISUAL:		
EXAMINER: NA	LEVEL .	DATE: 10-23-98		
	LEVEL			
THLET OUT	LET RSI-SWZ ACCESSABLE TION OF WELD TIND SUPPORT LEG. PERC			
KEWAUNEE NUCLEAR POWER PLANT REVIEW: AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	C. Bukes	DATE: October 30,1998		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	Roger Motivin	DATE: /1-30-98		





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-2

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

INSERVICE INSPECTION IMPRACTICALITY

- 1. ASME Code Component Affected: Integrally Welded Attachments
 APSI-1A-S1, APSI-1A-S3 and APSI-1AS4 to Safety Injection Pump APSI-1A
 and APSI-1B-S1, APSI-1B-S2 and
 APSI-1B-S4 to Safety Injection Pump
 APSI-1B
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-C; Item No. C3.30
- 4. Impracticality of Compliance: 14.6% of each Integrally Welded Attachment was inaccessible due to configuration of the Welded Attachment in close proximity to the Safety Injection Pump thus restricting application of the Surface Examination Magnetic Particle.
- 5. Burden Caused by Compliance: To provide for access to the 14.6% weld area between the welded attachments and the Safety Injection Pump would require modification from the Original Design of the Safety Injection Pump and the Welded Attachments.

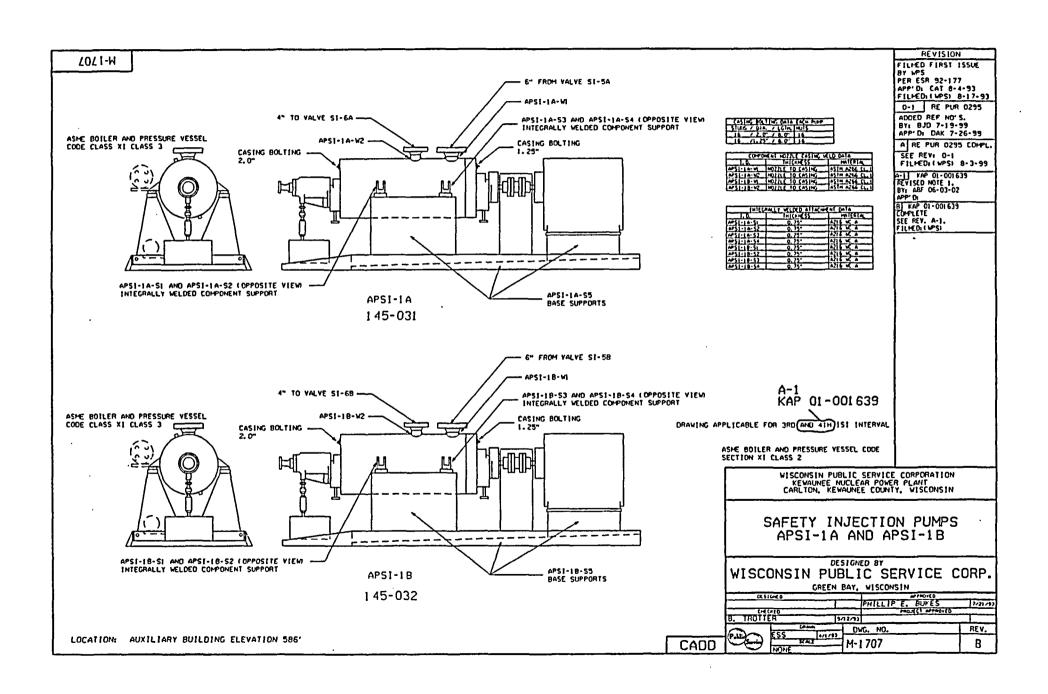
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-2

- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface examination is available due to limited access. VT-2 and VT-3 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: VT-2 Table IWC-2500-1; Examination Category C-H; Item C7.50 each 3 1/3 Year Period for evidence of leakage and VT-3 as required by Table IWF-2500-1; Examination Category F-A; Item No. F1.40B for general mechanical and structural condition.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

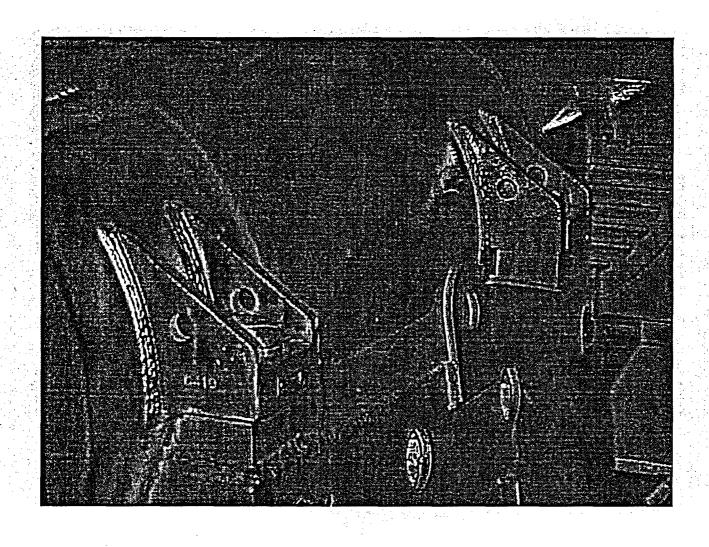
8. Precedents: Not Applicable

9. References: Not Applicable

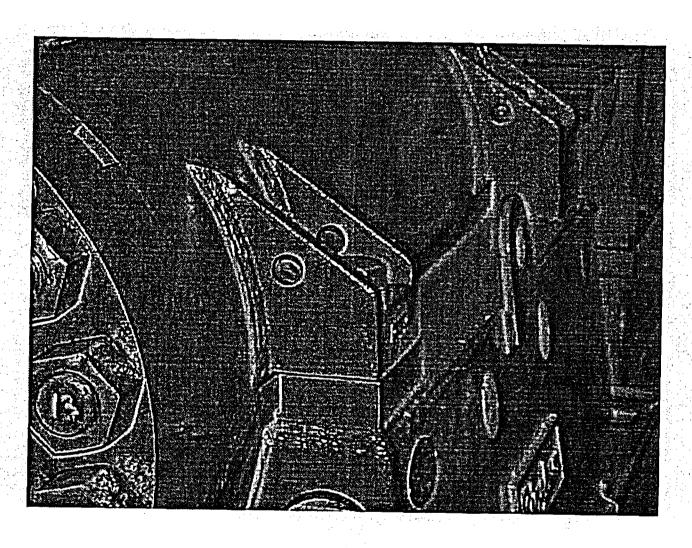


SYSTEM OR COMPONENT: SAFETY INSECTION PUMPS APSI-1A AND APSI-1B
DRAWING NO.:
COMPONENT IDENTIFICATION: APSI- 18-31 PROCEDURE: NEP NO. 15.7 REVISION: OKIG
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: _X VISUAL:
EXAMINER: TRAUIS THOMAS IT DATE: 10-27-01 LEVEL
EXAMINER: DATE: DATE:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
TYP. OF 3 SUPPORTS EXAMINED
SAFETY INSECTION PUMP CASING
LIMITATION AREA ON BACK SIDE LIMITED AREA DUE TO NO ACCESS
←
AREA NOT EXAMINED = 14.6%.
CASTS AND CAR
REWAUNEE NUCLEAR POWER PLANT REVIEW: Phellip C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: From Majorin Date: 10/30/01
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Toga Myrin DATE: 10/30/01

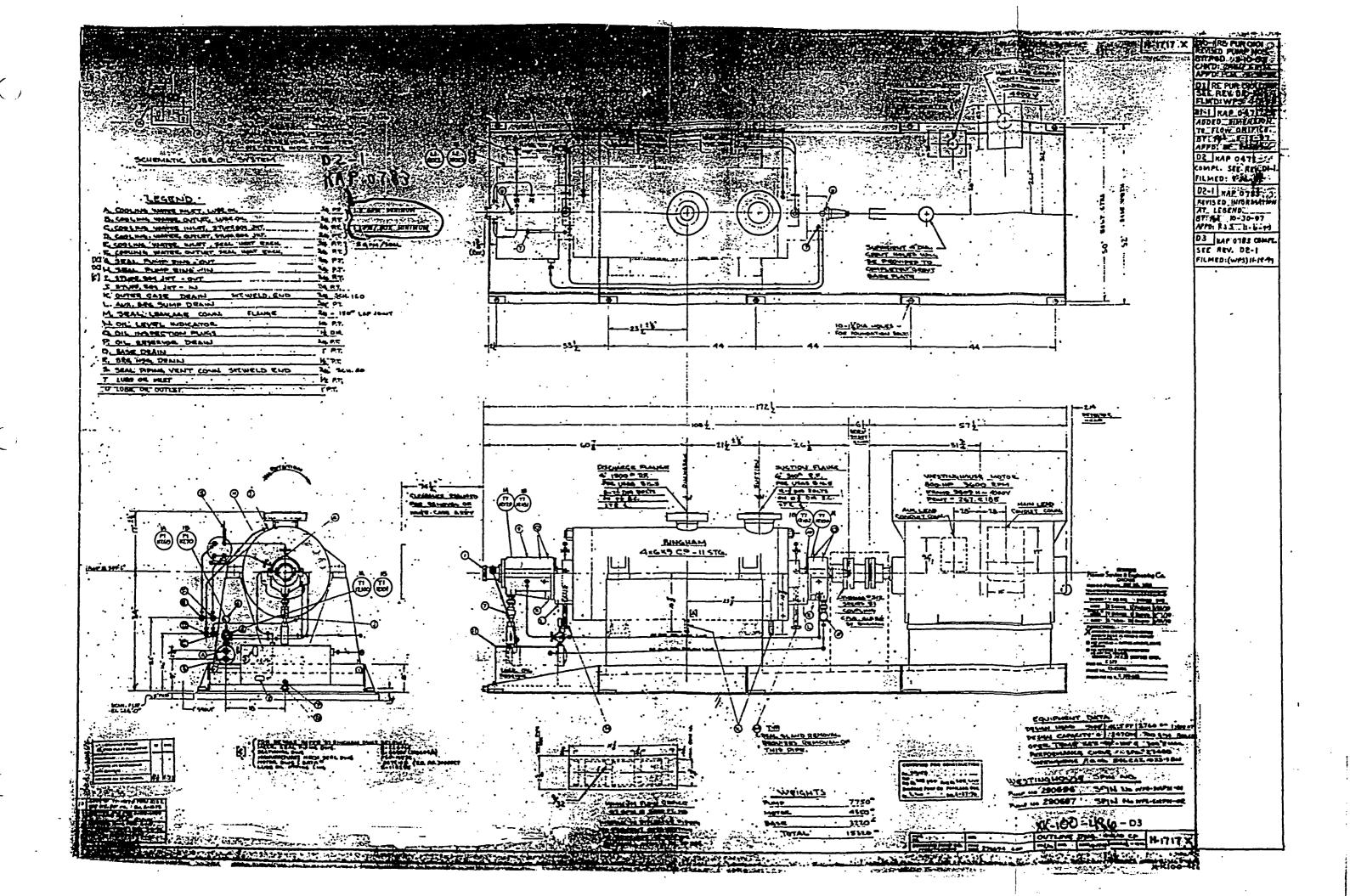
SAFETY INJECTION PUMPS SYSTEM OR COMPONENT: <u>APSI-IA AND APSI-IB</u> DRAWING NO.: M-1707				
APSI -18 -52 COMPONENT IDENTIFICATION: APSI - 14 - 51 PROCEDURE: QCP-902 REVISION: ORIG				
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:				
EXAMINER: DATE: 4-22-95 LEVEL				
EXAMINER: NAR BOY IT DATE: 4-22-95 LEVEL				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
PERCENTAGE OF REDUCED EXAM COVERAGE 85.4% EXAMINED 14.6% NOT EXAMINED				
TYP. OF 3 SUPPORTS EXAMINED LIMITATION AREA ON BACK SIDE SIDE VIEW END JIEN				
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: april 34,1995 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: And DATE: 4/34/55				
INSERVICE INSPECTOR REVIEW: The DATE: 4/34/85				



SaFeTY INJECTION PUMP aPSI-1a INTEGRALLY WELDED attachments TYPICAL



SaFETY INJECTION PUMP aPSI-1B INTEGRALLY WELDED attachmenTS TYPICAL-PRIOR TO Paint Removal



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-3

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

INSERVICE INSPECTION IMPRACTICALITY

- 1. ASME Code Component Affected: Integrally Welded Attachments

 APSI-1A-S2 to Safety Injection Pump

 APSI-1A and APSI-1B-S3 to Safety

 Injection Pump APSI-1B
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-C; Item No. C3.30
- 4. Impracticality of Compliance: 17.0% of each Integrally Welded Attachment was inaccessible due to configuration of the Welded Attachment in close proximity to the Safety Injection Pump thus restricting application of the Surface Examination Magnetic Particle.
- 5. Burden Caused by Compliance: To provide for access to the 17.0% weld area between the welded attachments and the Safety Injection Pump would require modification from the Original Design of the Safety Injection Pump and the Welded Attachments.

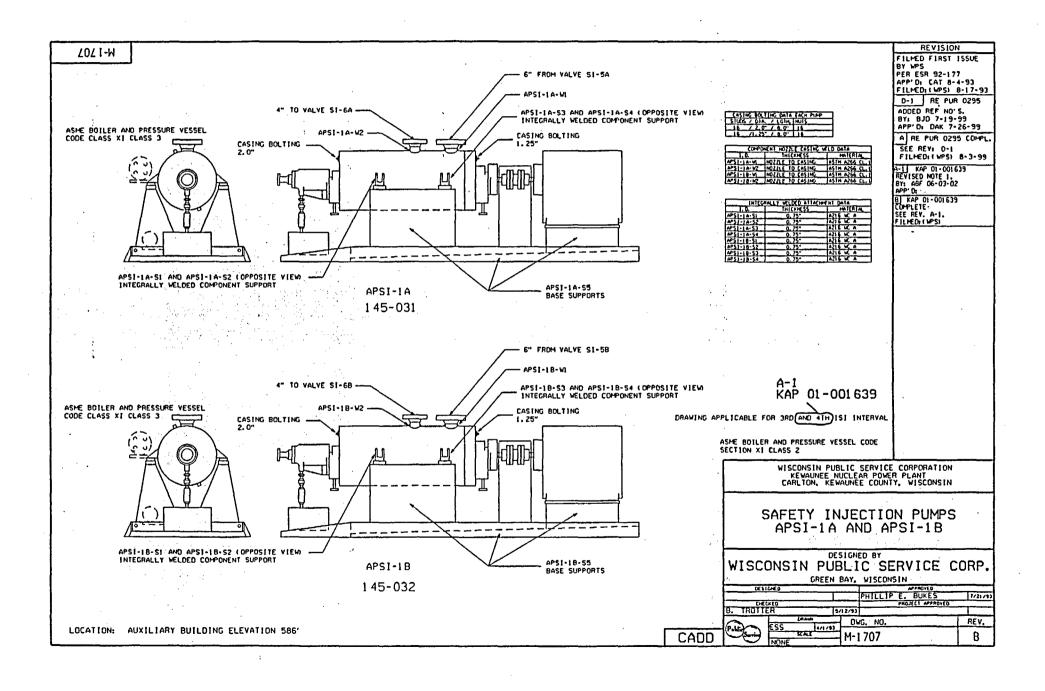
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-3

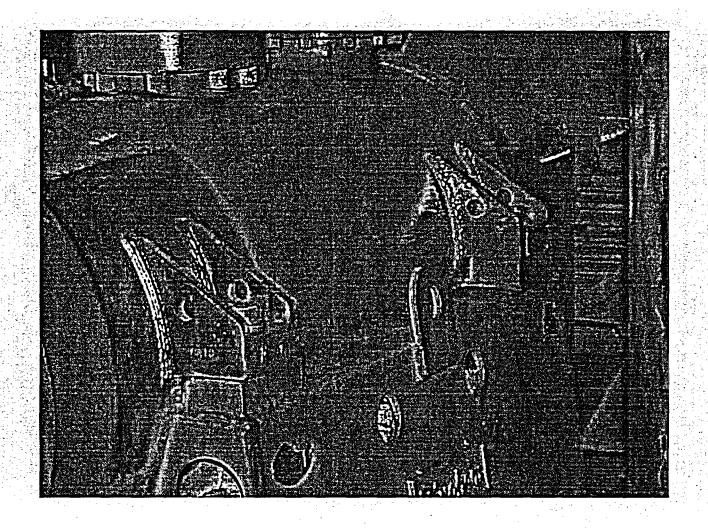
- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface examination is available due to limited access. VT-2 and VT-3 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: VT-2 Table IWC-2500-1; Examination Category C-H; Item C7.50 each 3 1/3 Year Period for evidence of leakage and VT-3 as required by Table IWF-2500-1; Examination Category F-A; Item No. F1.40B for general mechanical and structural condition.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

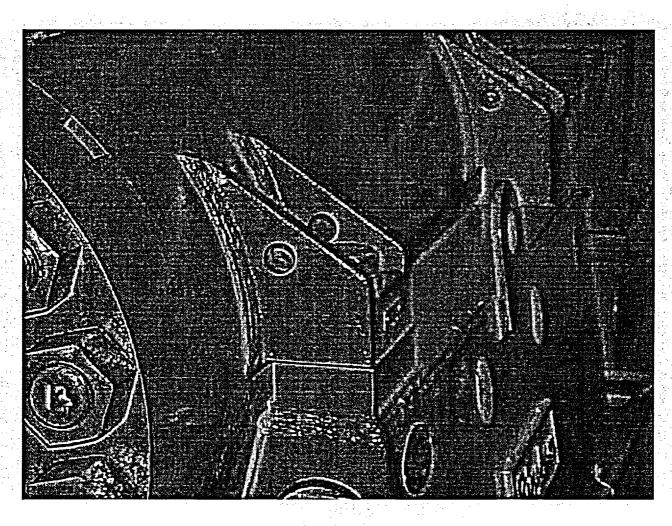
9. References: Not Applicable



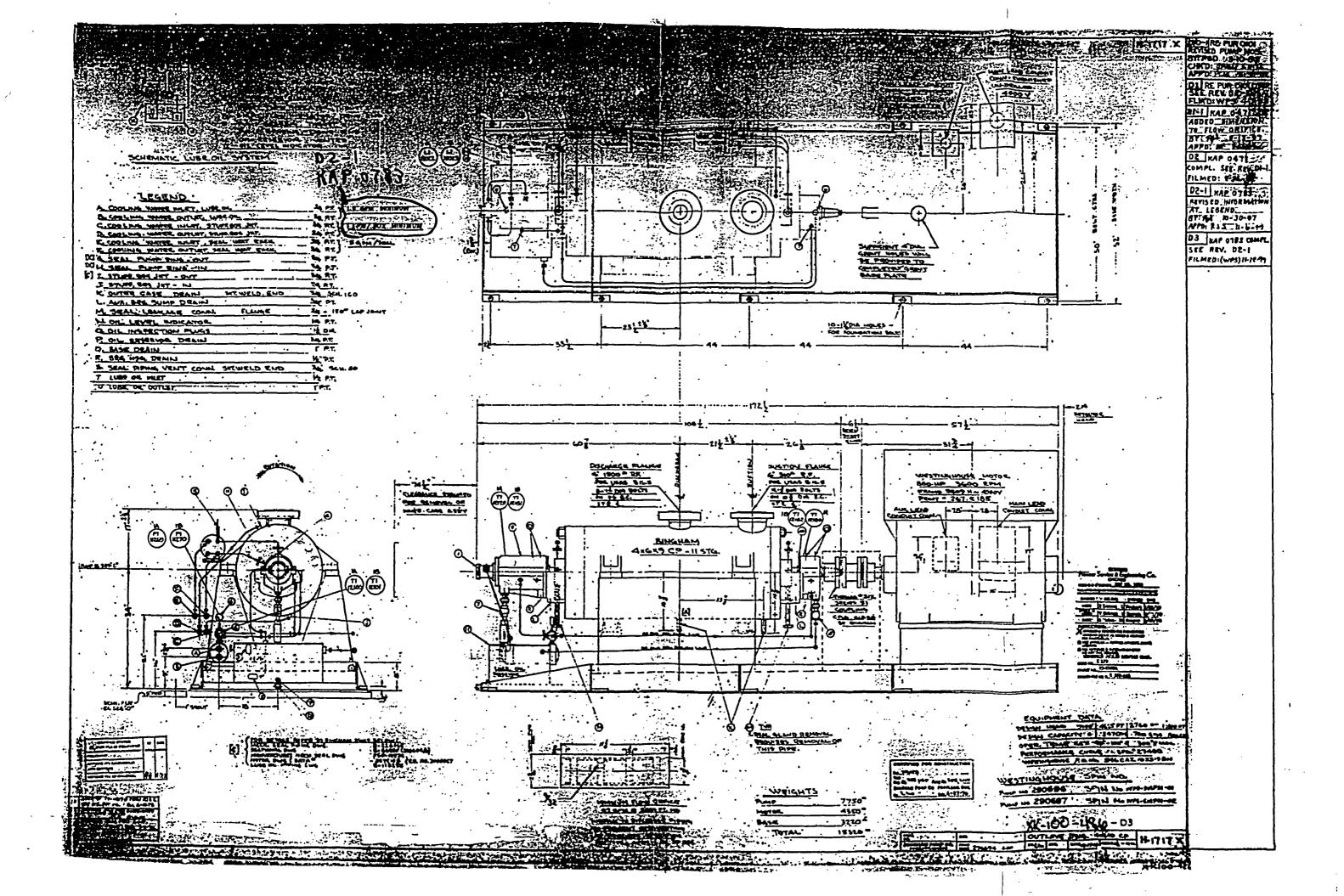
SYSTEM OR COMPONENT: <u>SAFETY IN JECTION PUMPS APSI - 1A AND APSI - 1B</u>				
DRAWING NO.:				
A PSI - 1B - 53 COMPONENT IDENTIFICATION: APSI - 1A - 52 PROCEDURE: NEP No. 15. 7 REVISION: Orig.				
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:				
EXAMINER: Jun Valorne II DATE: 11-3-98 LEVEL				
EXAMINER: DATE: 11-3-98 LEVEL				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
83% EXAMINED 17% NOT EXAMINED (due to limitation) LIMITED WELD EVANA AREA				
EXAM AREA BEHIND ATTACHMENT				
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: November 5,1998				
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roya Mines DATE: 11-6-98				



Safety Injection Pump APSI-1A Integrally Welded Attachments Typical



SaFety Injection Pump aPSI-1B Integrally Welded attachments TYPICAL-PRIOR TO Paint Removal



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-4

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

INSERVICE INSPECTION IMPRACTICALITY

- 1. ASME Code Component Affected: Reactor Vessel Closure Head To Flange Weld RV-W12
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-A; Item No. B1.40
- 4. Impracticality of Compliance: 23.0% of the Reactor Vessel Head to Flange Weld RV-W12 was inaccessible due to configuration of the Reactor Vessel Closure Head Flange and the 3 Welded Lifting Lugs thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 23.0% of the Head to Flange Weld RV-W12 would require modification from the Original Design of the Reactor Vessel Closure Head and removal of the 3 Welded Lifting Lugs. During the Fall 2004 Refueling Outage Kewaunee Nuclear Power Plant replaced it's Reactor Vessel Closure Head. The Replacement Reactor Vessel Closure Head was manufactured as a one piece forging. Thus a Reactor Vessel Closure Head to Flange Weld does not exist and ASME Boiler and Pressure Vessel Code Section XI 1989 Edition Table IWB-2500-1 Examination Category B-A, Item No. B1.40 no longer applies.

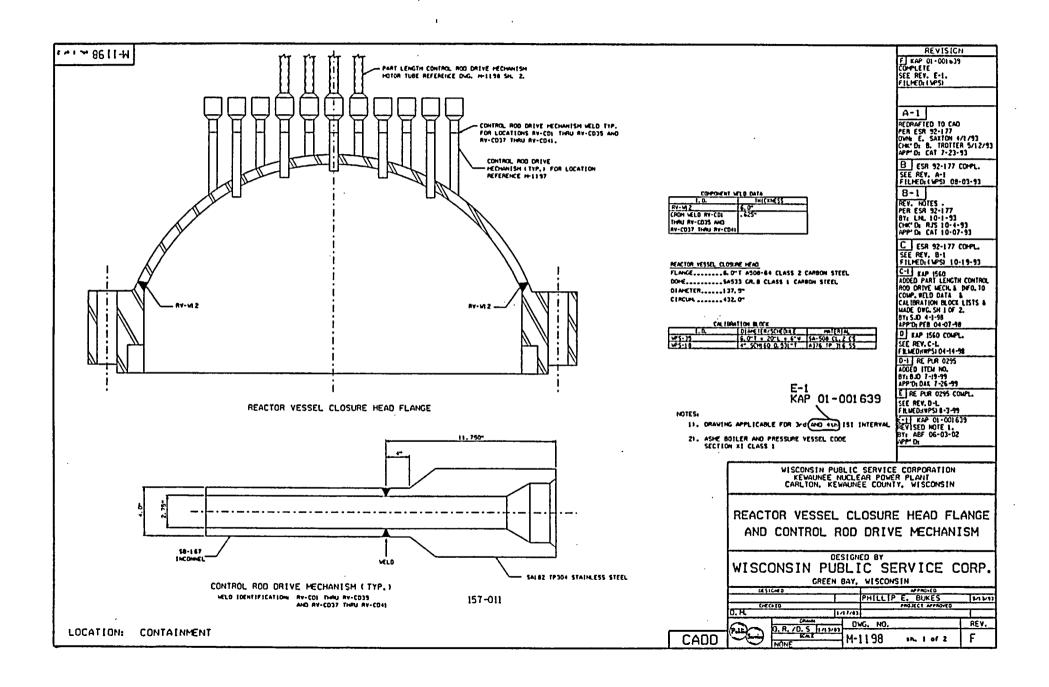
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-4

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: VT-2 Table IWB-2500-1; Examination Category B-P; Item B15.10 each Refueling Outage for evidence of leakage. Surface Magnetic Particle Examinations were performed as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition Table IWB-2500-1; Examination Category B-A; Item No. B1.40. During the Fall 2004 Refueling Outage Kewaunee Nuclear Power Plant replaced it's Reactor Vessel Closure Head. The Replacement Reactor Vessel Closure Head was manufactured as a one piece forging. Thus a Reactor Vessel Closure Head to Flange Weld does not exist and ASME Boiler and Pressure Vessel Code Section XI 1989 Edition Table IWB-2500-1Examination Category B-A, Item No. B1.40 no longer applies.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable



REACTOR VESSLE CLOSURE HEAD FLANGE SYSTEM OR COMPONENT: AND CONTROL ROD DREVE MECHANISM DRAWING NO.: M- 1198				
COMPONENT IDENTIFICATION: RV - W/3 PROCEDURE: QCP 90-1 REVISION: ORIG.				
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:				
EXAMINER: Mahl Man II DATE: 4-11-95 EXAMINER: Mahl Man II DATE: 4-11-95 LEVEL				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
REACTOR VESSLE CLOSURE HEAD METALLUSATE AL REFLECTOR				
STUD #1 1				
FLANGE				
LILTRASONIC SCAN LIMITATIONS - LIFTING LUG AT 88" CW FRONT Q OF #1 STUD. SCAN LIMITED ON FLANGE SIDE TO 3" FROM WELD & REFLECTOR IS OF METALLURGICAL ORIGIN AND DEDNOT ENTERFERE WITH THE 45' OR 60' EVAMS				
KEWAUNEE NUCLEAR POWER PLANT REVIEW: S. A. Balstad DATE: 4/13/95 AUTHORIZED NUCLEAR				
INSERVICE INSPECTOR REVIEW: Ann. Profum. DATE: 4/14/55				

REV.: ORIG.

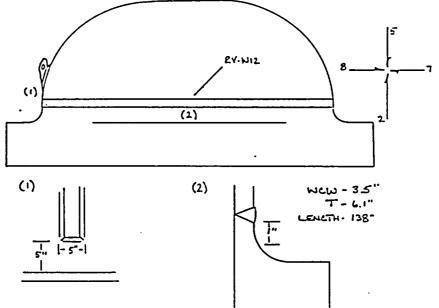
SYSTEM OR COMPONENT: REACTOR VESSEL CLOSURE HEAD	
DRAWING NO .: M-1198 SH. 1 0 F Z	
COMPONENT IDENTIFICATION: RV-W12 PROCEDURE: NEP-15.09 REVISION: A	
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:	
EXAMINER: Prian O. Knott II DATE: 10/2/01 EXAMINER: Iman Cothes I DATE: 10-2-01	_
EXAMINER: Simon Crothes II DATE: 10-2-01 LEVEL	_
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.)
D SCAN S, 45'+60' EXAMS LIMITED DUE TO LIFTING LUG AT STUD HOLE 43. ALL SCANS 0' 45' +60' EXAMS LIMITED DUE TO HEAD TO FLANGE CONFIGURATION.	
COVERAGE: REDUCED CODE/PROCEDURE COVERAGE = 23% OF 138° EXAM LENGTH, FROM & OF STUD HOLE 33 TO & OF STUD HOLE 1.	,
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Fran Philips DATE: 10-4-01	<u></u>
INSERVICE INSPECTOR REVIEW: Kryn Prym DATE: 10-4-01	_

REV.: ORIG.

WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

REACTOR VESSEL CLOSURE HEAD FLANGE SYSTEM OR COMPONENT: AND CONTROL ROD DRIVE MECHANISM
DRAWING NO.: M-1198 SH.1 of Z
COMPONENT IDENTIFICATION: RV-WIZ PROCEDURE: NEP NO. 15.9 REVISION: ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER:
EXAMINER: DATE: 10-27-98
EXAMINER: DATE: 10-27-98 LEVEL

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.



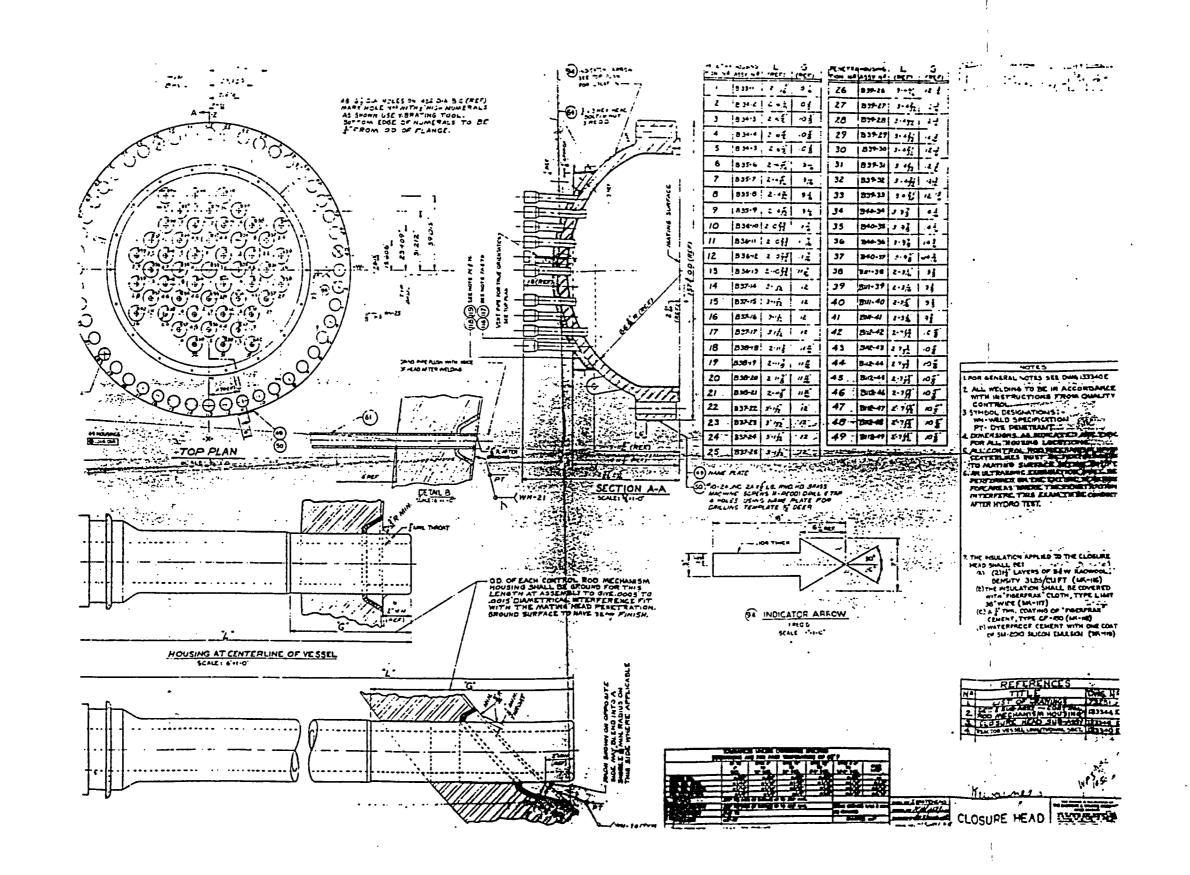
NOTE: ALL SCAUS. 0. 45°: 60° EXEMPLETIONS LIMITED DIE TO REACTOR HELD TO FLANCE CONFIGURATION.

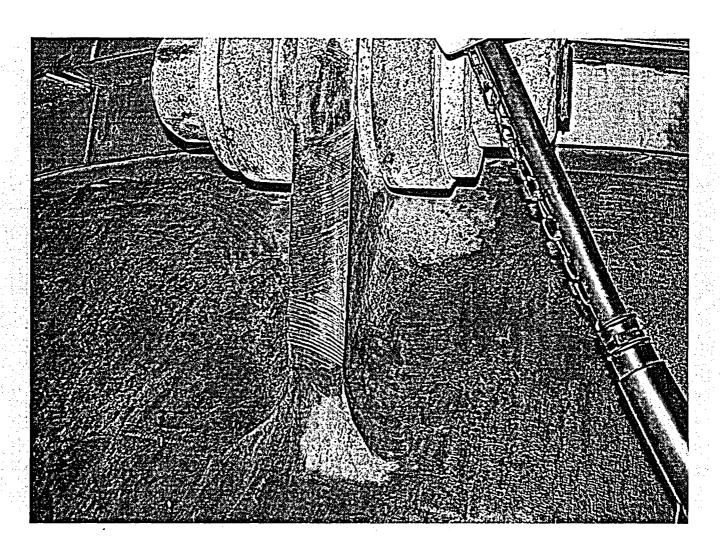
SCAN 5. 45° (60° EXEMPLETIONS LIMITED DIE TO INTEGRAL WELLED ATTACAMENT (LIFTING INC.) LOCATED AT STUD HOLE 27 (85.5°).

REDUCED CODE / PROCEDUCAL CONSERLE: 23% *

4 23% of 138"(E of stud ble 17 to E of stud ble 33)

REWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes	DATE: October 29,1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royal Magnetic	DATE: 10-30-98





Reactor Vessel Closure Head LIFTING LUG



Reactor Vessel Closure Head Flance to Head Configuration

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-5

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

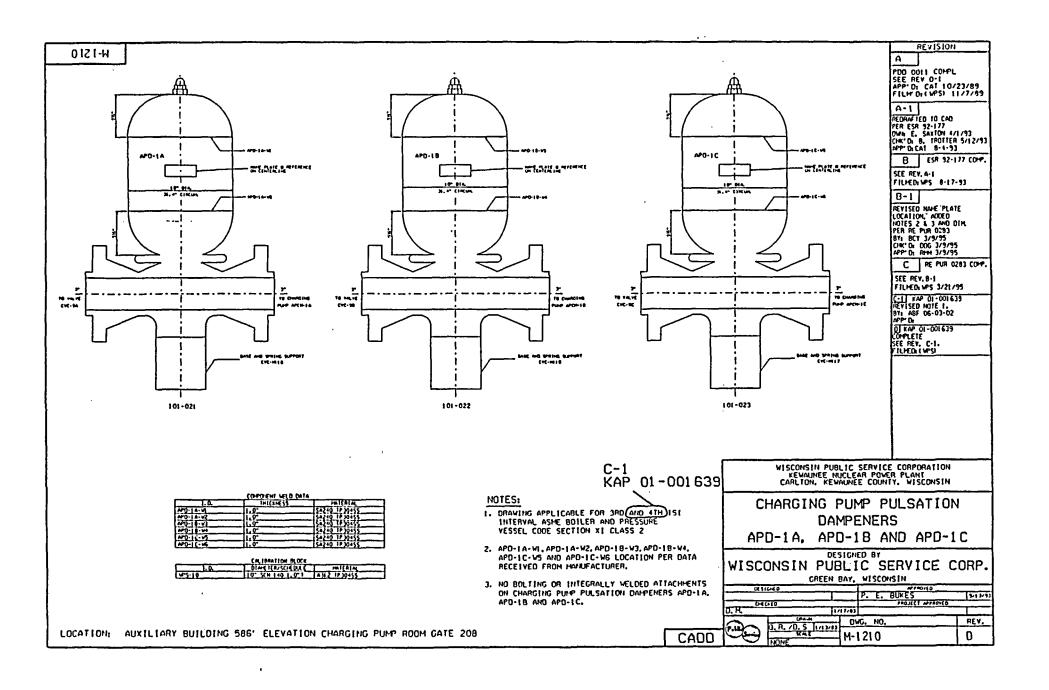
- 1. ASME Code Component Affected: Head Circumferential Weld APD-1A-W1 to Charging Pump Pulsation Dampener 1A; Head Crcumferential Weld APD-1B-W4 to Charging Pump Pulsation Dampener 1B and Head Circumferential Welds Top and Bottom to Spare Charging Pump Pulsation Dampener
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 9.0% of the Weld APD-1A-W1, 8.6% of Weld APD-1B-W4 and 9.0% of Top and Bottom Spare Head Welds were inaccessible due to a Welded Name Plate in close proximity to the Charging Pump Pulsation Dampener Circumferential Head Welds thus restricting the Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide access to the 9.0% of APD-1A-W1, 8.6% of APD-1B-W4 and 9.0% of Top and Bottom Spare Head weld scan area between the Welded Name Plate and circumferential weld on the Charging Pump Pulsation Dampener would require removal and reinstallation of the Welded Name Plate.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

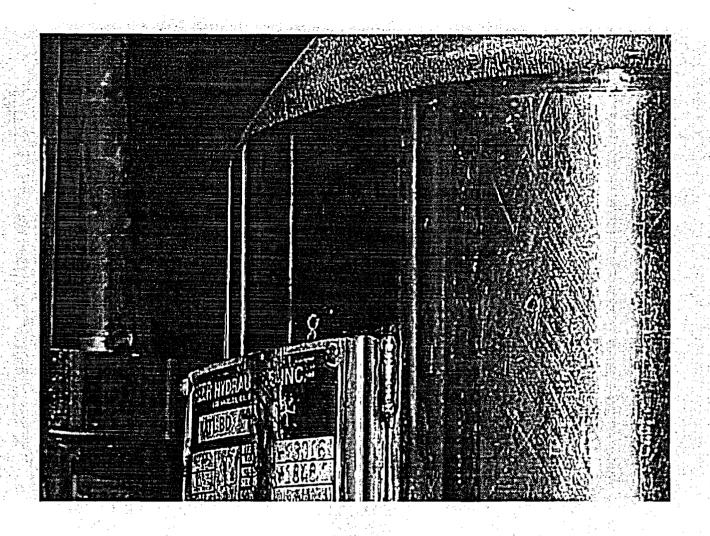
RELIEF REQUEST NO: RR-G-7-5

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

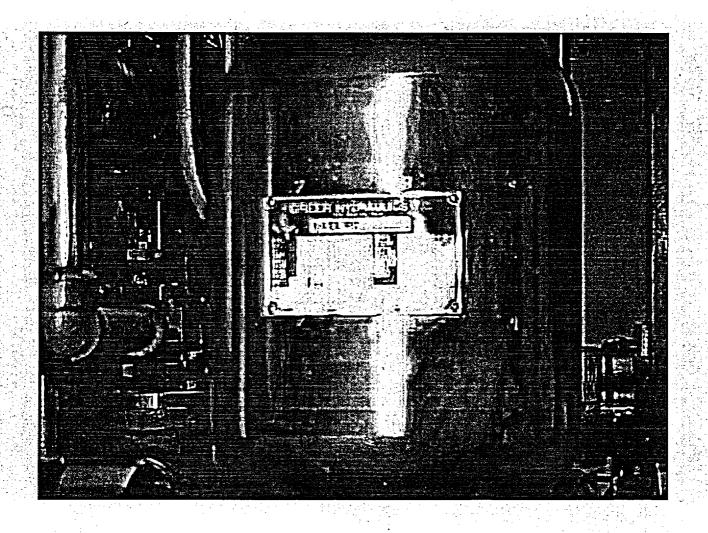


CHARDING PUND PUISATION SYSTEM OR COMPONENT: DAMDENER SIN GHI 1848 DRAWING NO .: M-1210		
COMPONENT IDENTIFICATION: APO - IA - WI PROCEDURE: GCP-911 REVISION: URL		
ULTRASONIC: _X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: Johl P. Blf I DATE: 4.3-85		
LEVEL		
EXAMINER: DATE: 04/03/55 LEVEL DATE: 04/03/55		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
WeiDed Name Plate Located affraximately 31%" From Centerline of each weid.		
WELDED HENE PLATE LIMITS 08 L. 45°5, 45°L CHD 60°S.		
Percentage of Reduced Coverage FORCED TAGE OF REDUCED COVERAGE FORCEDURE: 15% ASME Code: 9%		
WELDED TOP WELD		
Scan Scan Nane PLate 3.0		
Bottom WELD		
7512		
* WELD LOCATION POR VENDOR (GREER HYDROULICS) INFORMATION RECEIVED FEDRUARY 24.1995.		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: 4/5/95		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Angu Profession DATE: 4/6/55		



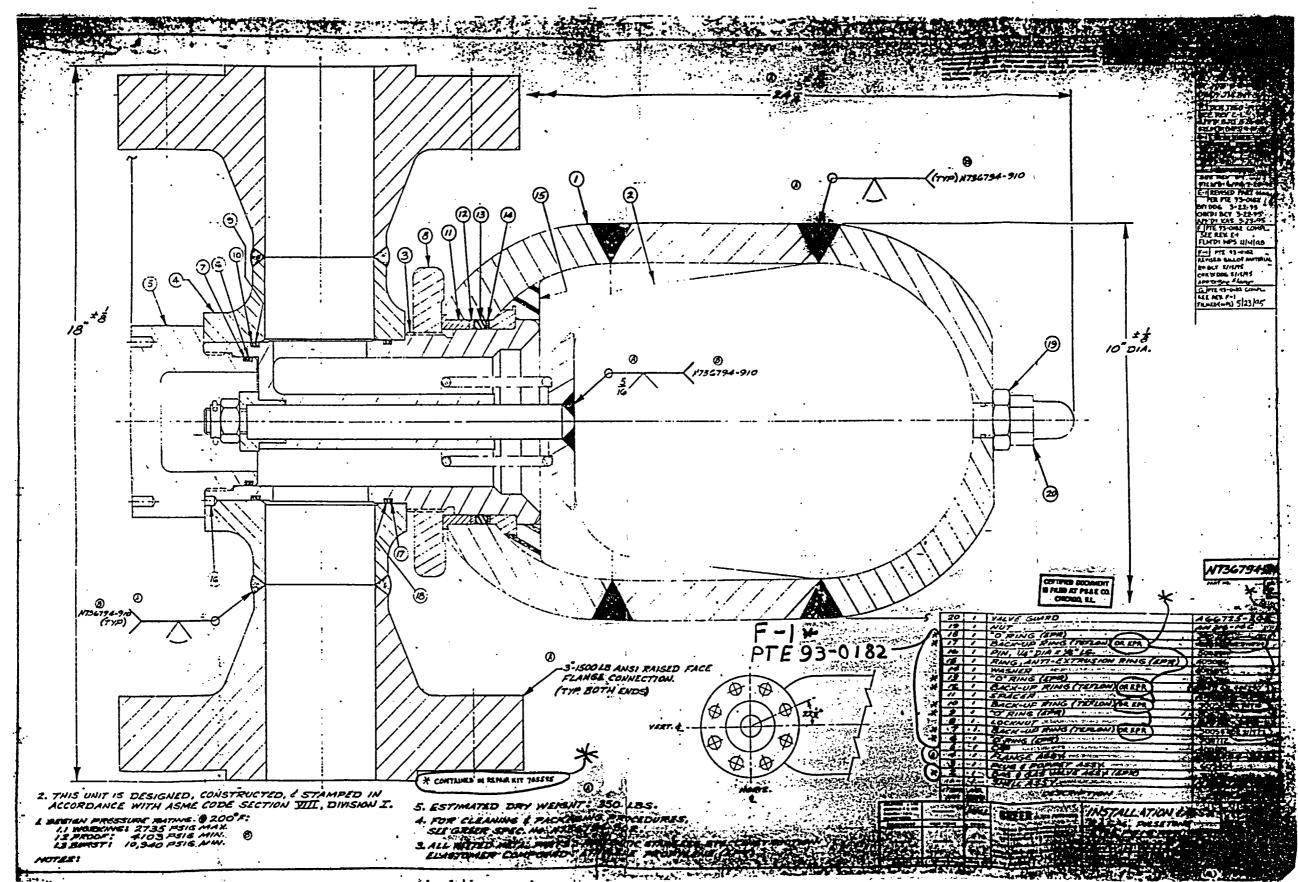
Charging Pump Pulsation Dampener 1A Weld APD-1A-W1

SYSTEM OR COMPONENT: CHARGING PUMP PULSATION DAMPENERS APD - 1A, APD-1B AND APD-1C
DRAWING NO.: M -1210
COMPONENT IDENTIFICATION: APD -1B - W4 PROCEDURE: NEP-15.41 REVISION: A
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: MARIBEL DATE: 10-18-01 LEVEL
EXAMINER: TIM COBURN MM I DATE: 10/18/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
OREFERENCE & OF NAME TAG A
SSIDE 1.
2510E APO-18-W4
I. NAME TAG - LIMITS 45° SCANS 5,7,8 ON 5 SIDE FOR 5½" NAME TAG IS LOCATED 4" TO 9%" FROM MARKED O AND Y4" FROM & ON 5 SIDE AND 15 - ELDED TO CHARGING PUMP PULSATION DAMPENER 18
REDUCED CODE COVERAGE 8.6%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C Bukes DATE: October 23, 2001
AUTHORIZED NUCLEAR



Charging Pump Pulsation Dampener 1B Weld APD-1B-W4

SPare Charbing Pung Pulsation SYSTEM OR COMPONENT: Dampener Slu GH 1 1846 DRAWING NO.: M-1210	
COMPONENT IDENTIFICATION: Tot weld: Bottom Weld PROCEDURE: GCP 911 REVISION: DRIG.	
ULTRASONIC: _ X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:	
EXAMINER: Philips C. Bukes II DATE: Feb. 3, 24, 1995 LEVEL EXAMINER: James ERettmann 111 DATE: Jan 19.324, 1995 LEVEL	
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. Welded Name Plate Located approximately 316" Fram Centerline of each weld. Welded Name Plate Limits 0° Li 45°5, 45°L	
SSCAN SSCAN SSCAN SSCAN SCAN SC	
* WELD LOCATION PER VENDOR (GREER HYDRAULICS) INFORMATION RECEIVED FEDRUARY 241995.	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Sold DATE: 2/28/95 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Hogo Mary Date: 2/28/95	



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-6

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Pressurizer Head Circumferential Welds P-W3 and P-W5
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-B; Item No. B2.11
- 4. Impracticality of Compliance: Approximately 2.0% of the scheduled examination of 100% of Weld P-W3 and 2.0% of the scheduled examination of 100% of Weld P-W5 were inaccessible due to Integral Welded Attachment Pads, 1" and ¾" Instrumentation Lines, Curvature of the Pressurizer Head, or a 3" ground area in close proximity to the Pressurizer Circumferential Head Welds thus restricting the Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide access to the 2.0% of PW-W3 and P-W5 weld scan area between the Integral Welded Attachment Pads, 1" and 3/4" Instrument Line, Curvature of the Pressurizer Head or 3" ground area would require modification of the original design of the Pressurizer.

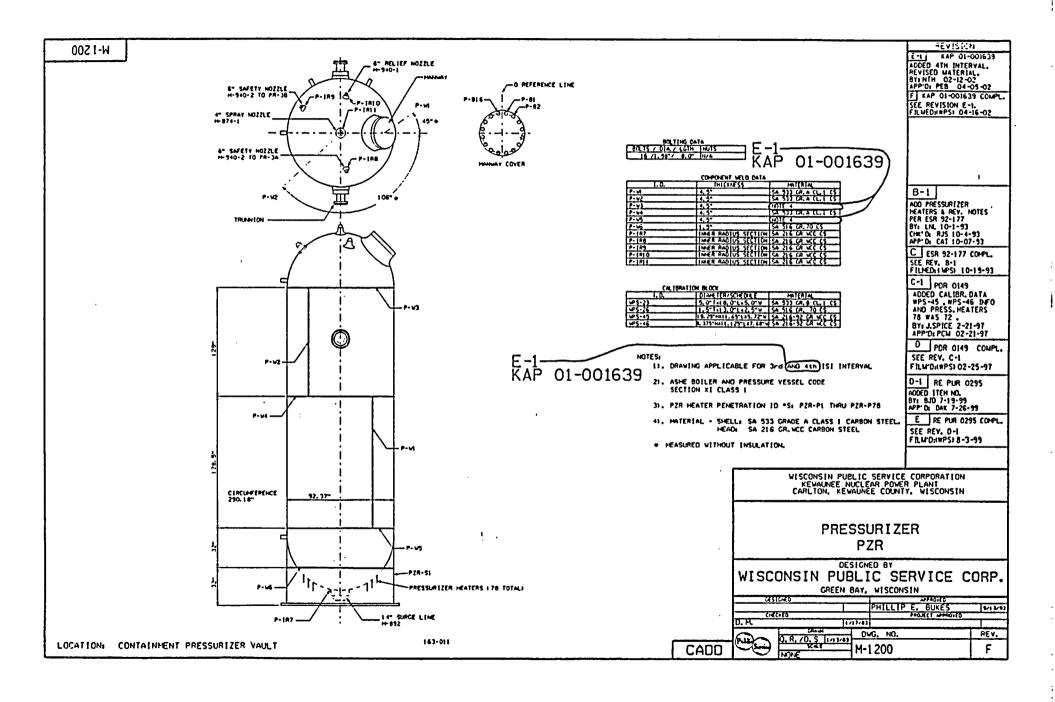
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-6

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each 3 1/3 Year Period for evidence of leakage.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



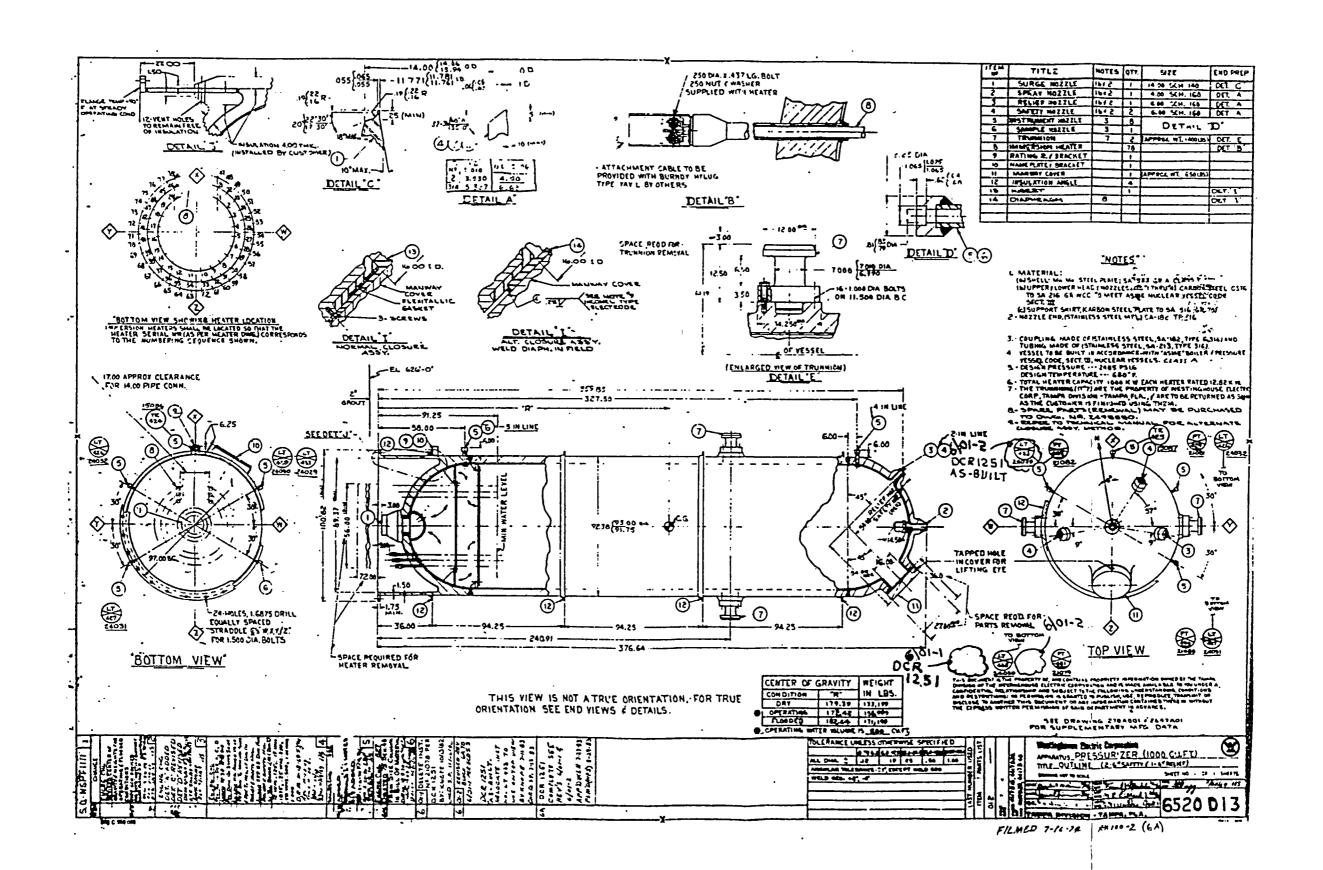
SYSTEM OR COMPONENT: PRESSURIZER PZR		
DRAWING NO.: M-1200		
COMPONENT IDENTIFICATION: P-W3 / P-W5 PROCEDURE: NEP No. 15.9 REVISION: OF		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: Jell Owes II DATE: 10/31/98 LEVEL		
EXAMINER: NA LEVEL		
LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
WELD CROWN WINTH 3.5" LENGTH OF EXAM 97.0"		
F		
NOTE: 0°, 45°, \$ 60° EXAMINATIONS LIMITED IN EACH SCAN DIRECTION DUE TO INTEGRAL WELDED ATTACHMENTS AT 145.5" CLOCKWISE WITH DIMENSIONS SHOWN ABOUE.		
(1) DINEUSIONS ILOCATIONS ARE THE SAME FOR WELD D.W. AUD WELD D.W.S.		
PERLENTAGE OF PROCEDURALICODE LIMITATION: 1.4% OF 97.0"		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 3,1998		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roya Profiter DATE: 11-3-98		

SYSTEM OR COMPONENT: PRESSURTZER	DRAWING NO.: 14 - 1200
COMPONENT IDENTIFICATION: P- w3	PROCEDURE: (DEP 904) REVISION: DRIG
ULTRASONIC: X LIQUID PENETRANT:	MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jeffy W. John	DATE: 4-13-75 4.19-95 LEVEL
EXAMINER: Muld floor	DATE: 4-18-95 4-19-95 LEVEL
	EATION, ORIENTATION, TYPE OF LIMITATION AND SED EXAMINATION COVERAGE. UATTO-U COVERAGE LESS THAN 2 %
ALEA GROUND 3' IN DIA. AGROY .3" DEEP 53'2	PW3
	73 ty
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. 13	Bukes DATE: april 23,1995
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	Montan DATE: 4/24/95

SYSTEM OR COMPONENT: PRESSURIZER PZR		
DRAWING NO.: <u>M-1200</u>		
COMPONENT IDENTIFICATION: P-W3 PROCEDURE: NEP-15.09 REVISION: A		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: Brian A. Knott II DATE: 10-3-01 LEVEL		
EXAMINER: DATE: 10-3-01		
LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
$ \begin{array}{c ccccccccccccccccccccccccccccccccccc$		
$\frac{P-W3}{3}$		
<u>(1)</u> 2		
EXAM AREA: 194" TO 0" = 120" (1) 3) WELDED PADS: 2"x 2", 1" FROM EACH TOE OF WELD. LIMITED: 0" /45" (SCANS 2,5,7+8) / 60" (SCANS 2,5,7+8)		
(2)(4) INSTRUMENTATION LINES: 1° DIAM, 3.5° FROM TOE OF WELD. LINITED: 45° (SCAN 5) / 60° (SCAN 5)		
(5) CURVATURE OF HEAD: 9° FROM TOE OF WELD. LIMITED: 60° (SCAN 5)		
REDUCED PROCEDURE COVERAGE = 3%		
REWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes DATE: October 8,2001		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ange Marine DATE: 10-9-01		

<u></u>	· · · · · · · · · · · · · · · · · · ·
SYSTEM OR COMPONENT: PRESSURTZER COMPONENT IDENTIFICATION: P-W5	PROCEDURE: ACP- 904 REVISION: OREG
ULTRASONIC: X LIQUID PENETRANT:	
EXAMINER: White was the second of the second	DATE: <u>4-18-95</u> 4-19-95
EXAMINER: Muli filler	DATE: 4-18-95 4-19-85 LEVEL
•	ATION, ORIENTATION, TYPE OF LIMITATION AND ED EXAMINATION COVERAGE. ON COVERAGE LESS THAN 2%
INSTRUMENTATION	2ω P-ω5
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bu	Res DATE: April 23, 1995 Postjum DATE: 4/24/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	12 DATE: 4/24/95

SYSTEM OR COMPONENT: PRESSURIZER PZR		
DRAWING NO.: M-1200		
COMPONENT IDENTIFICATION: P-W5 PROCEDURE: NEP-15.09 REVISION: A		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: Bream D. Knott II DATE: 10-3-01 LEVEL		
EXAMINER: Seen S. Knott II DATE: 10-3-01 LEVEL EXAMINER: DATE: 10-3-01 LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND		
PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
5 7		
P-W5		
2422 219 194"		
EXAM AREA: 194" TO 0" 120"		
13 WELDED PADS: 2" X 2" 1" FROM EACH TOE OF WELD.		
LIMITED: 0. / +5. (SCANS 2,5,7 + 8) / 60. (SCANS 2,5,7 + 8)		
(2)(4) INSTRUMENTATION LINES: 1° DIAM, 3.5" FROM TOE OF WELD. LIMITED: 45° (SCAN 2) / 60° (SCAN 2)		
REDUCED PROCEDURE COVERAGE = 2%		
REDUCED PROCEDURE COVERAGE = 27.		
,		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Buked DATE: October 8,2001		
REWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Buked DATE: October 8,2001 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogar Majuri DATE: 10-9-01		



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-7

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

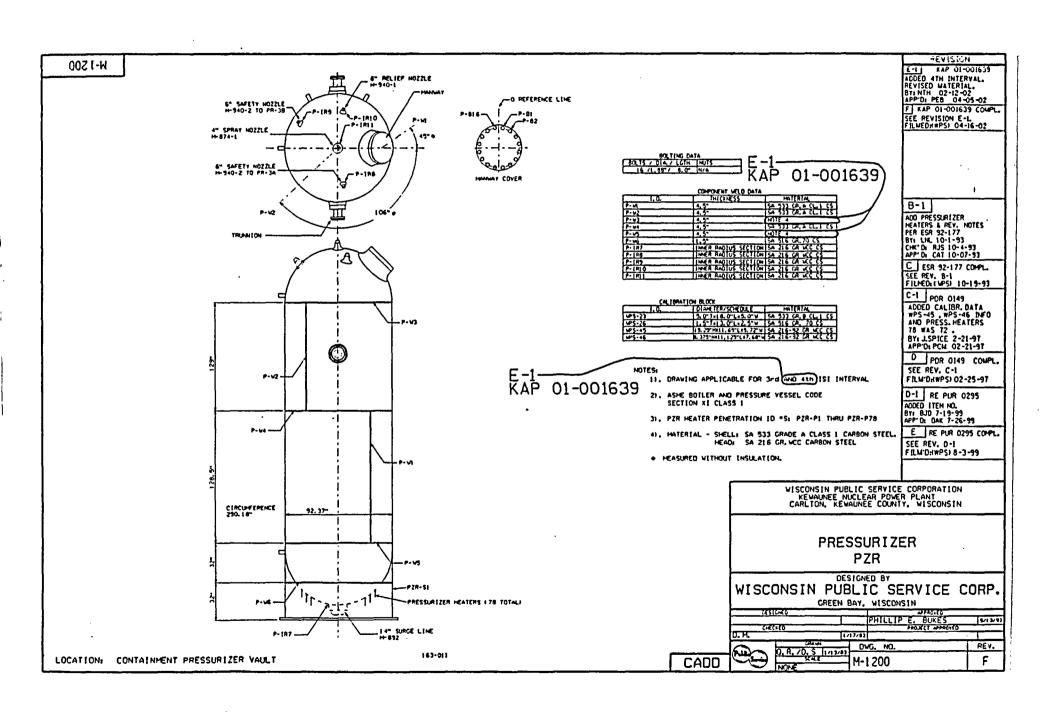
- 1. ASME Code Component Affected: Pressurizer Head Longitudinal Weld P-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-B; Item No. B2.12
- 4. Impracticality of Compliance: 3.0% of the scheduled examination of 12" of Weld P-W2 was inaccessible due to an insulation support ring in close proximity to the Pressurizer Longitudinal Weld thus restricting the Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide access to the 3.0% of PW-W2 would require removal of the insulation support ring and the insulation panels being supported on the ring. Removal of the insulation support ring and the insulation panels could cause a unsafe condition and an increase in personnel exposure due to the amount of time requiring Kewaunee Nuclear Power Plant to construct a temporary support to maintain the remaining Pressurizer Insulation Panels and the difficulty in removing and reinstalling the Insulation Support Ring and Insulation Panels in the correct location.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

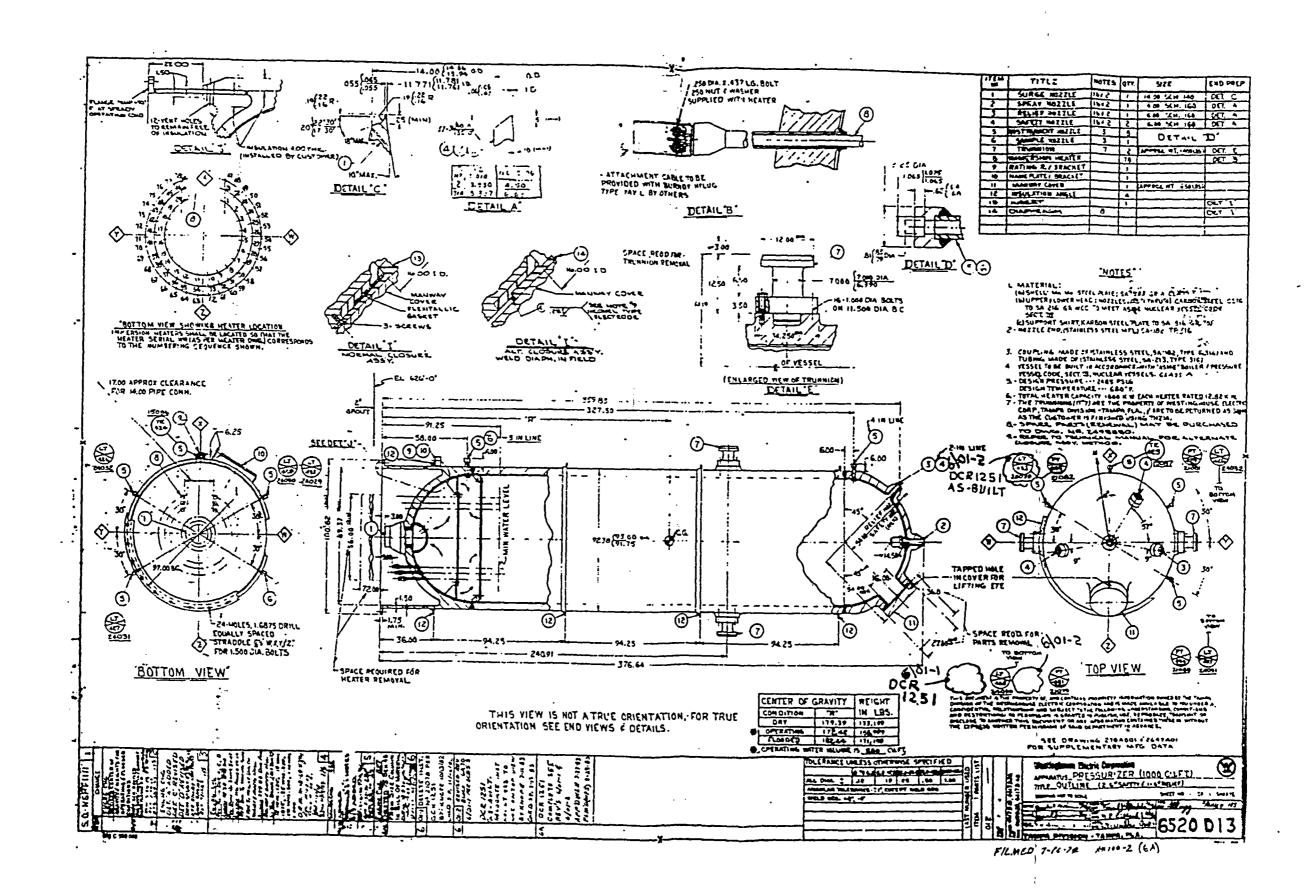
RELIEF REQUEST NO: RR-G-7-7

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



205566121270
SYSTEM OR COMPONENT: PRESSURIZER PZR
DRAWING NO.: M-1200
COMPONENT IDENTIFICATION: P-W2 PROCEDURE: NEP-15.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Limon Crothers I DATE: 10-5-01.
EXAMINER: Brian D. KNOW TT DATE: 10/5/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
OREF. 5
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes DATE: October 8,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Fry Majuri DATE: 10-7-01



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-8

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

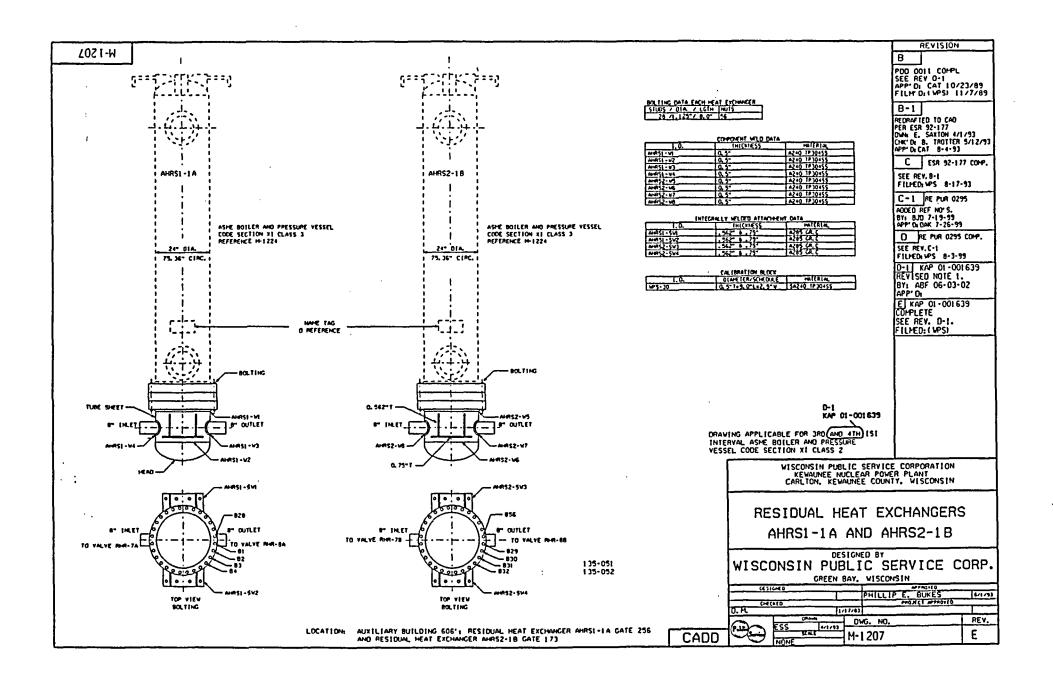
- 1. ASME Code Component Affected: Residual Heat Exchanger AHRS1-1A
 Shell Circumferential Weld AHRS1-W1
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.10
- 4. Impracticality of Compliance: 77.0% of the Residual Heat Exchanger Shell Circumferential Weld AHRS1-W1 was inaccessible due to configuration of the Residual Heat Exchanger Flange to Shell, 2 Welded Supports and the 8" Inlet Nozzle and the 8" Outlet Nozzle thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 77.0% of the Shell Circumferential Weld AHRS1-W1 would require modification from the Original Design of the Residual Heat Exchanger.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-8

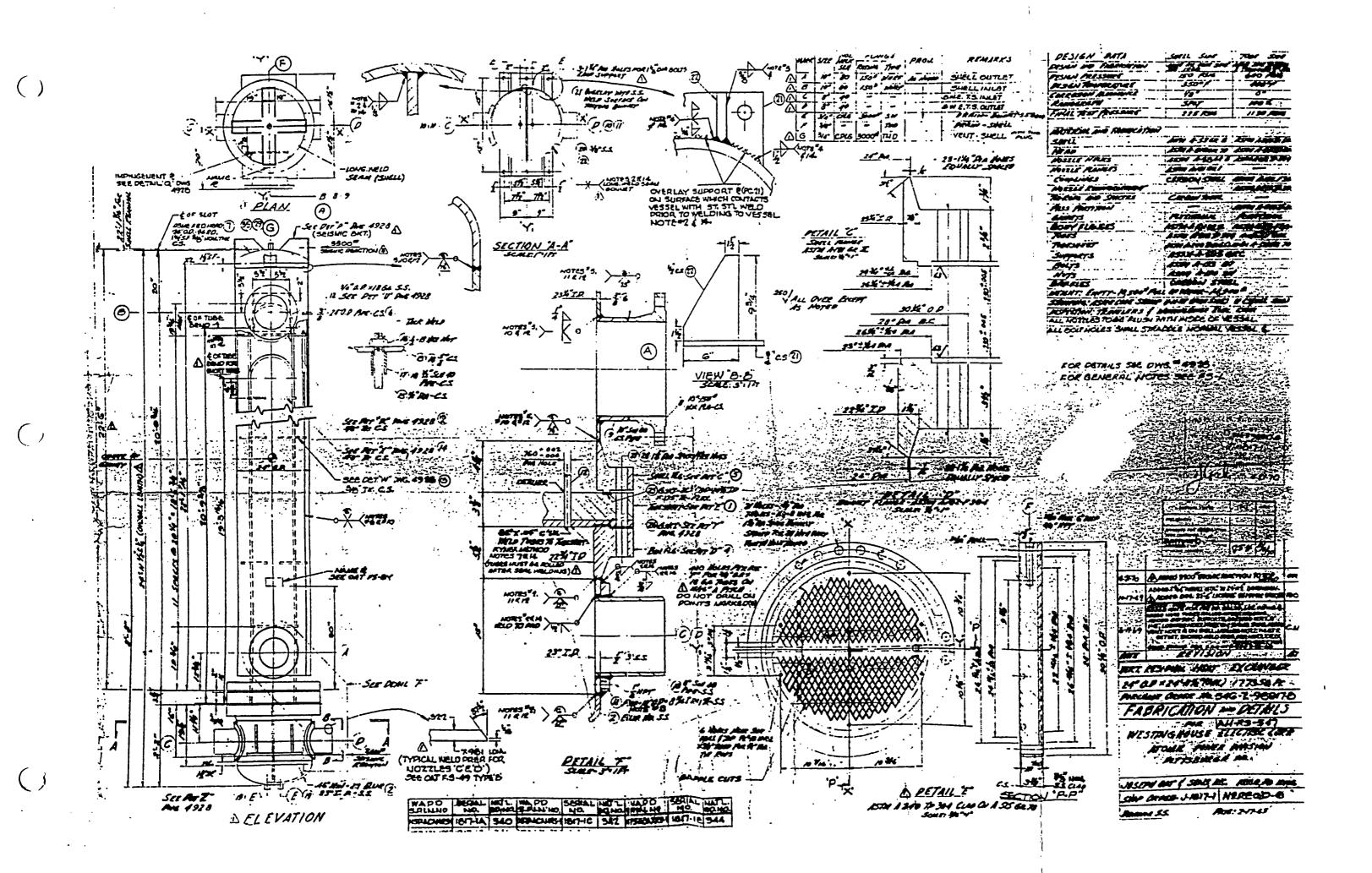
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

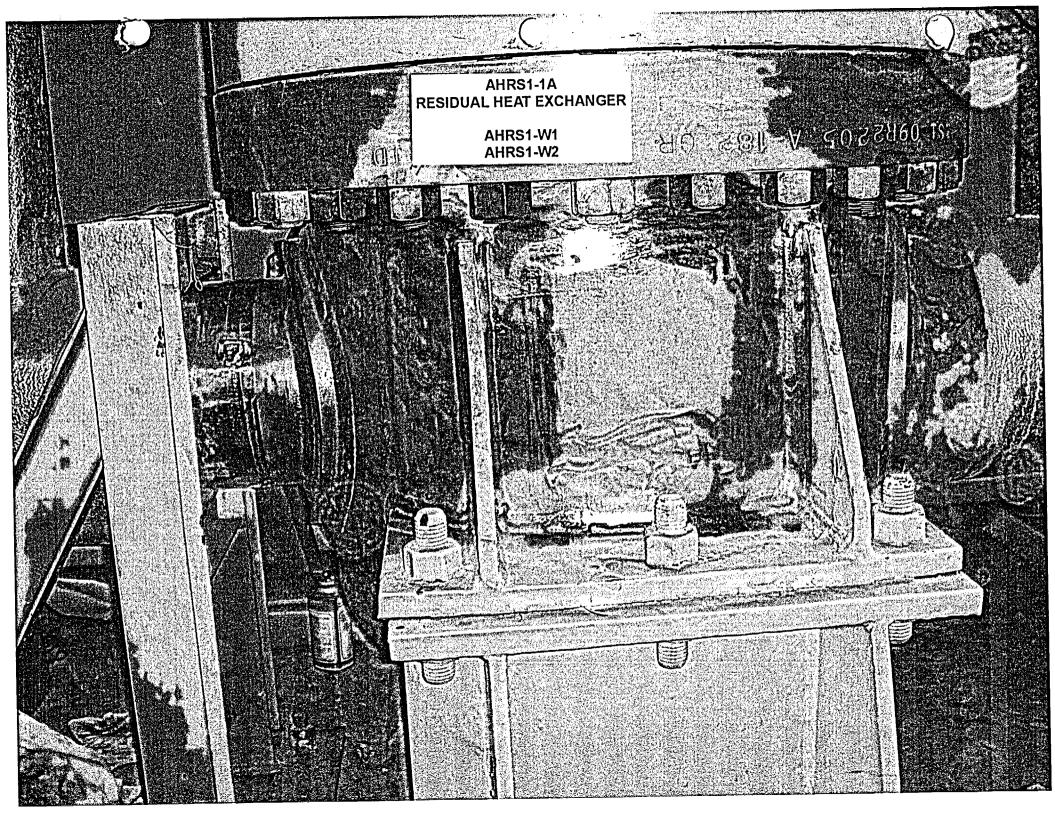
8. Precedents: Not Applicable



THORE EXTENSIVE ENTERING TO EXAMINATION TREGOTED
RESTOUAL HEAT EXCHANGERS SYSTEM OR COMPONENT: AHRS1-1A AND AHRS2-1B DRAWING NO.: M-1207
COMPONENT IDENTIFICATION: AHRS1 . W1 PROCEDURE: QP-911 REVISION: ORIG.
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: NIA. By DATE: 4-25-95
EXAMINER: NA NA DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
FLANGE 44.25'cw 33.5'cw AHRS1WI INLET WELDON SUPPORT AHRS1SWI Limits 2,748 SCAN FOR 45'S AND 60'S, AND 0° SCAN. Sheet I of 3
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Sind A Balala DATE: 4/27/95 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ray May DATE: 4/29/95

RESIDUAL HEAT EXCHANGERS
SYSTEM OR COMPONENT: AHRS1-1A AND AHRS2-1B DRAWING NO.: M-1207
COMPONENT IDENTIFICATION: AHRS 1 - W1 PROCEDURE: QCP-911 REVISION: ORIG.
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: NAA-BY II DATE: 4-25-95
EXAMINER: NA DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
FLANGE 6.25cm 69.86cm Well AHRSI.WI
INLET OUTLET AHRS1-SW 2 FLANGE CONF. GUTANOM
Limits 5,748 Scan For Welded Support Scan. MRS1 - SW2 Limits
INLET AND OUTLET NOTELES ITT 60'S AND 0'SCAM.
SCAN. PCRCENTAGE OF REDUCED EXAMINATION COVERAGE. 77%
Sheet 2 of 3
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Sind A. Balala DATE: 4/27/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Korga Photogram DATE: 4/27/55





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-9

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

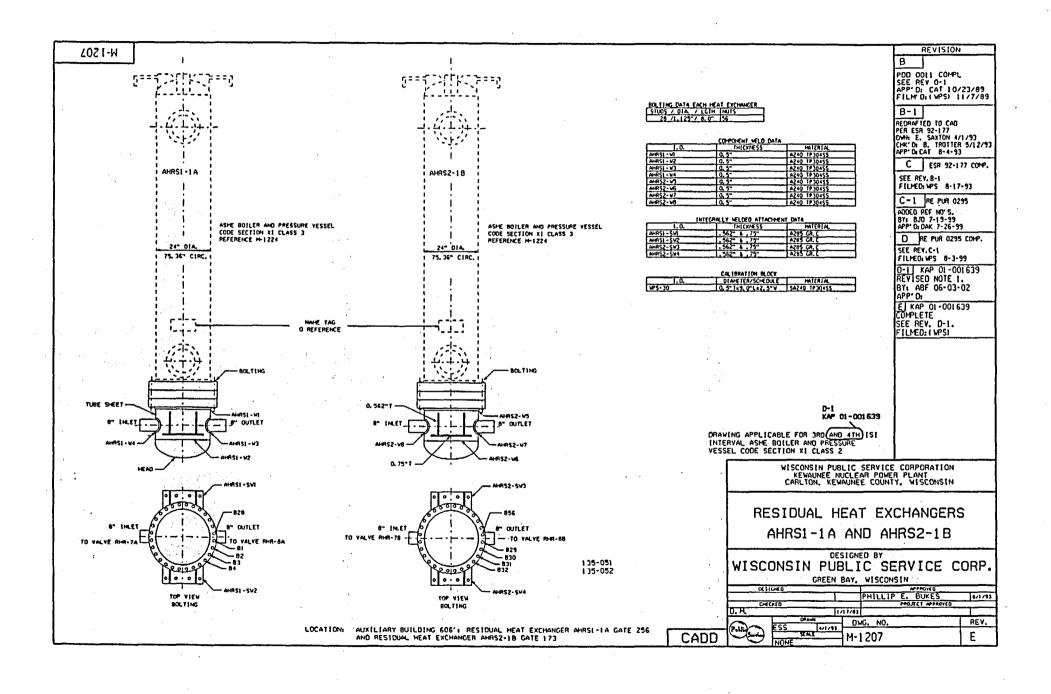
- 1. ASME Code Component Affected: Residual Heat Exchanger AHRS1-1A
 Head Circumferential Weld AHRS1-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 37.6% of the Residual Heat Exchanger Head Circumferential Weld AHRS1-W2 was inaccessible due to configuration of the Residual Heat Exchanger Welded Support thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 37.6% of the Head Circumferential Weld AHRS1-W2 would require modification from the Original Design of the Residual Heat Exchanger.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

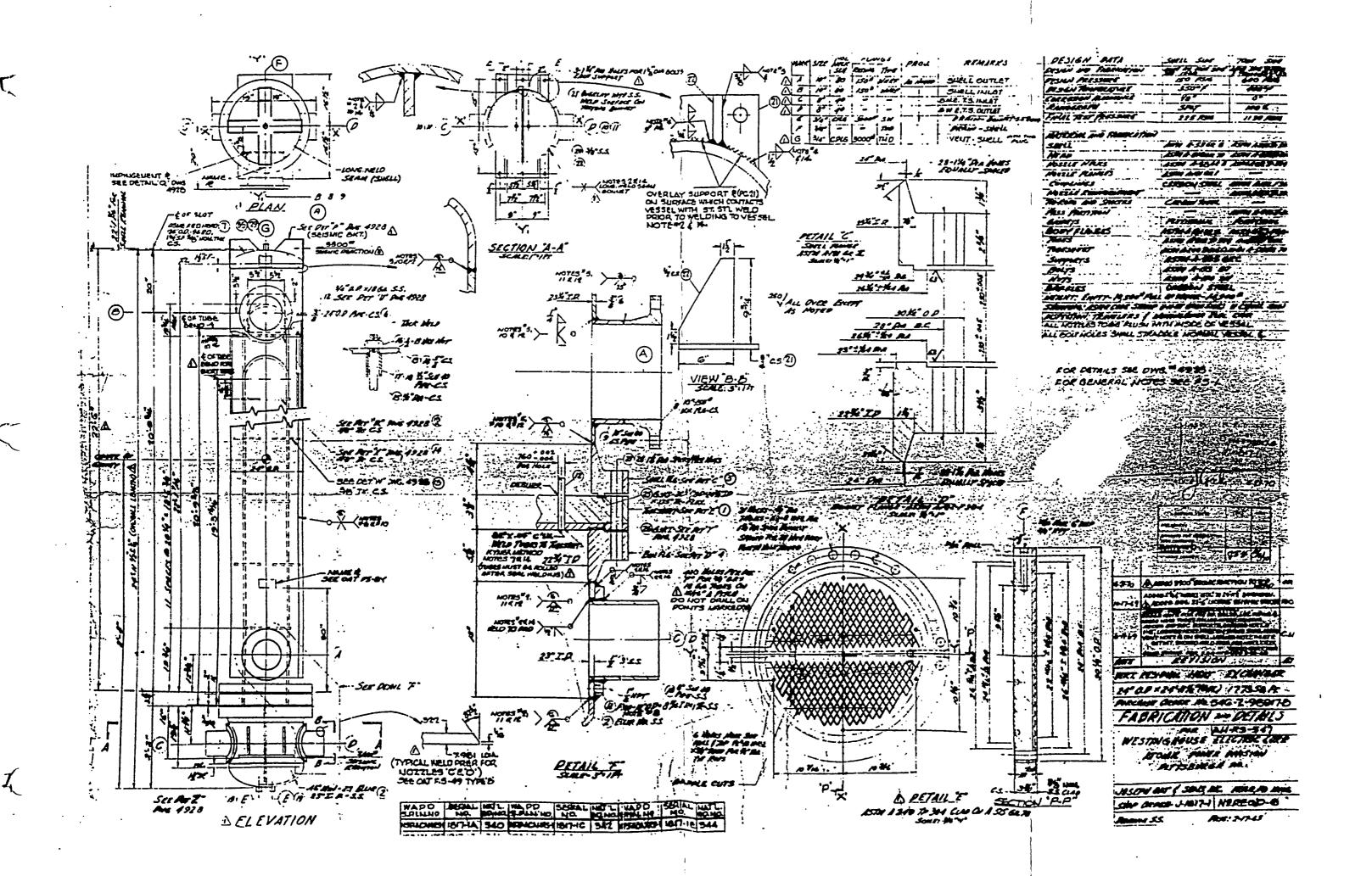
RELIEF REQUEST NO: RR-G-7-9

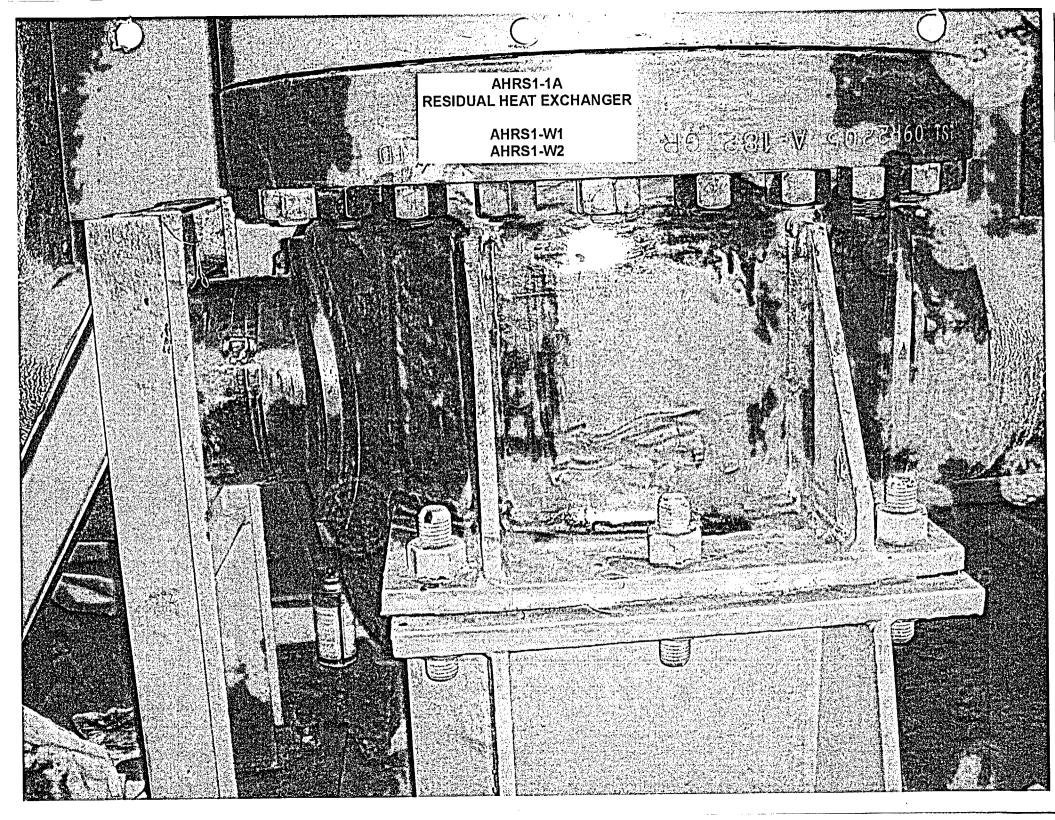
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: RESIDUAL HEAT EXCHANGERS AHESINA AND	AHQ52 -1B	
DRAWING NO.: M . 1267		
COMPONENT IDENTIFICATION: AHRS 1 . W 2 PROCEDURE: MED NO. 15.	يك REVISION: مريخ	
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE:	VISUAL:	
EXAMINER: DATE: LEVEL EXAMINER: DATE: LEVEL		
LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
AHRSI.W2	1	
NOTE: SLAN 2.5.7.8 ; DO ETAMBLETION LIMITED TO ALES NOTED ABOUT DUE TO SUPPORTS I SACOLE WELD INTERFERENCE.		
REDULED DEDLEARL CONTENDE: 30.2%. ZEDULED CODE, CONTENDE: 37.6%.		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes	DATE: October 30,1998	
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roga Instrum	DATE: 10-30-98	





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-10

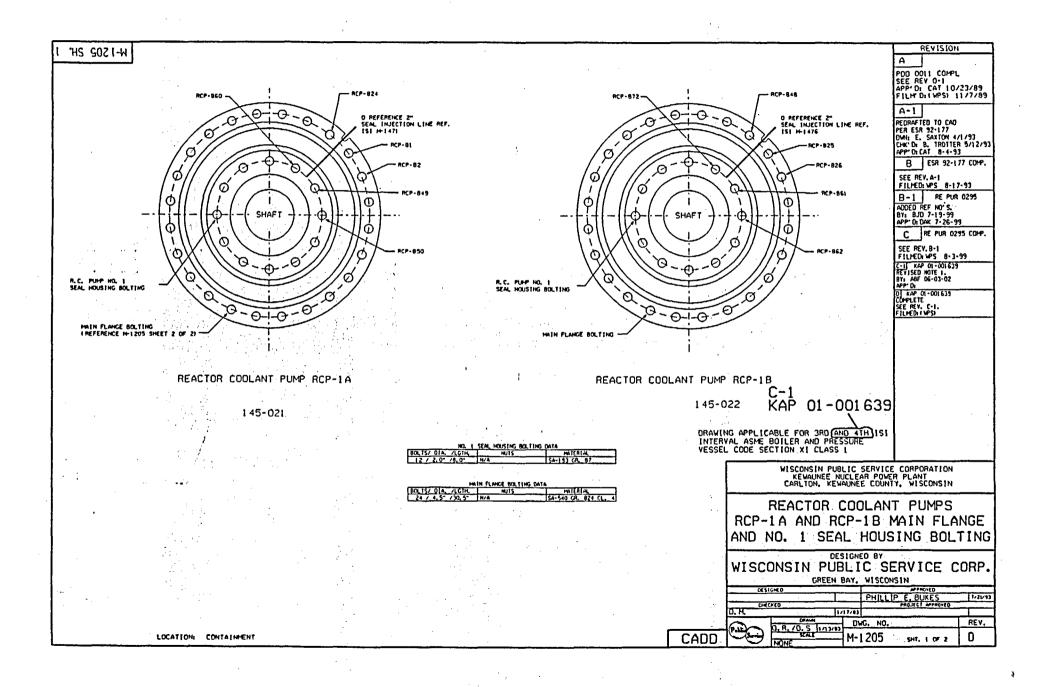
PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

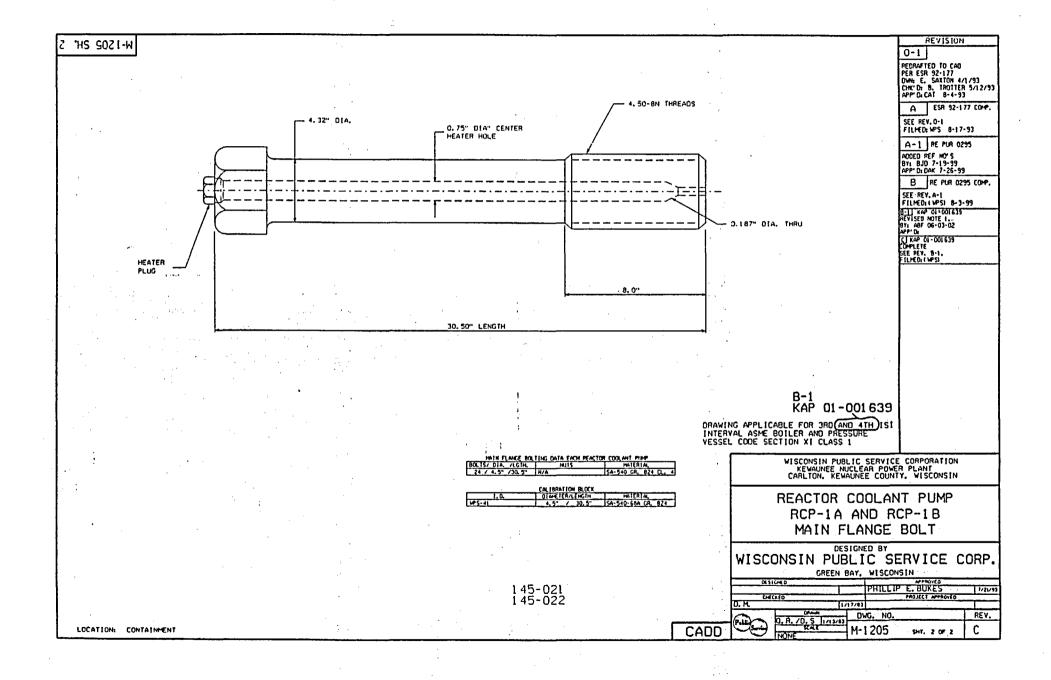
- 1. ASME Code Component Affected: Reactor Coolant Pump 1A Main Flange Bolting RCP-B1 though RCP-B8, RCP-B9 and RCP-B11 through RCP-17
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-G-1; Item No. B6.180
- 4. Impracticality of Compliance: 7.3% of each Reactor Coolant Pump 1A Main Flange Bolt was inaccessible due to configuration of Main Flange Bolt thus restricting Bore Hole Probe Ultrasonic Examination during the 1st and 2nd Period.
- 5. Burden Caused by Compliance: To provide for access to the 7.3% of the Main Flange Bolt would require modification from the Original Design of the Reactor Coolant Pump Main Flange Bolt.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-10

- 6. Proposed Alternative and Basis for Use: Kewaunee Nuclear Power Plant Bore Hole Probe examinations required a 70° Forward, 70° Reverse and 90° Surface Scan to cover 100% of the required length of the Main Flange Bolt during the 1st and 2nd Examination Periods. Configuration of the Main Flange Bolt limited by 7.3% the 70° Forward and 90° Surface Scans. Alternative Code required 0°Straight Beam Examination per ASME Boiler and Pressure Vessel Code Section XI 1995 Edition Appendix VIII Supplement 8 was performed on the remaining 8 Main Flange Bolts during the 3rd Examination Period. The 0° Straight Beam Examination although less restrictive in number of transducers requires less access and thus No Limitations were recorded. If KNPP would have continued use of the Ultrasonic Probe technique as permitted by ASME Boiler and Pressure Vessel Code Section XI 1995 Edition Appendix VIII Supplement 8 during the 3rd Period limit to examinations would have been encountered and additional identical Relief Requests would require submittal. 4TH Interval Examinations will be performed utilizing the approved Appendix VIII Supplement 8 - 0° Straight Beam to eliminate the Limitation on the Reactor Coolant Pump Main Flange Bolting and the need for Relief Request Submittal.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004
- 8. Precedents: Not Applicable
- 9. References: Not Applicable





TIOCAL EXAMINATION FINITATION TO EXAMINATION TIEGOTED
SYSTEM OR COMPONENT: R.C. PUMP BOLT DRAWING NO.: M1205
COMPONENT IDENTIFICATION: WPS - 41 PROCEDURE: 910 REVISION: ORIG
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: DATE: 4-21-95 LEVEL
EXAMINER: DATE: 4-21.95 LEVEL DATE: 4-21.95 LEVEL
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND
PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
→ 3" ←
AREA OF REQUIRED VOLUME NOT EXAMINED
OUE TO BOLT CONFIGURATION
FOR 90° SULFACE EXAM
PERCENTAGE OF REDUCED EXAMINATION COVERAGE 92.7% EXAMINED 7.3% NOT EXAMINED
7.5 18 700 85141100
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: april 23, 1995
AUTHORIZED NUCLEAR Authorized Nuclear
INSERVICE INSPECTOR REVIEW: The Marie DATE: 4/24/95

SYSTEM OR COMPONENT: RC PUMPS RCP-IA AND RCP-IB MAIN FLANGE AND No. I SEAL HOUSING BOLTING
DRAWING NO.: M-1205 SHT 1 of Z
RCP-B9, RCP - BII COMPONENT IDENTIFICATION: THRU RCP - BIT PROCEDURE: NEP No. 15.15 REVISION: Orig.
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER:
EXAMINER: Jug 6/stome I DATE: 10 - 30 - 98 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
LITOLITIAL OF TILBOOLD EXAMINATION GOVERNAL.
AREA OF REQUIRED VOLUME NOT EXAMINED DUE TO BOLT CONFIGURATION FOR 90° SURFACE EXAM AND 70° FORWARD EXAM (PER STUD)
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 5, 1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ryn Material DATE: 11-4-98

KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-11

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

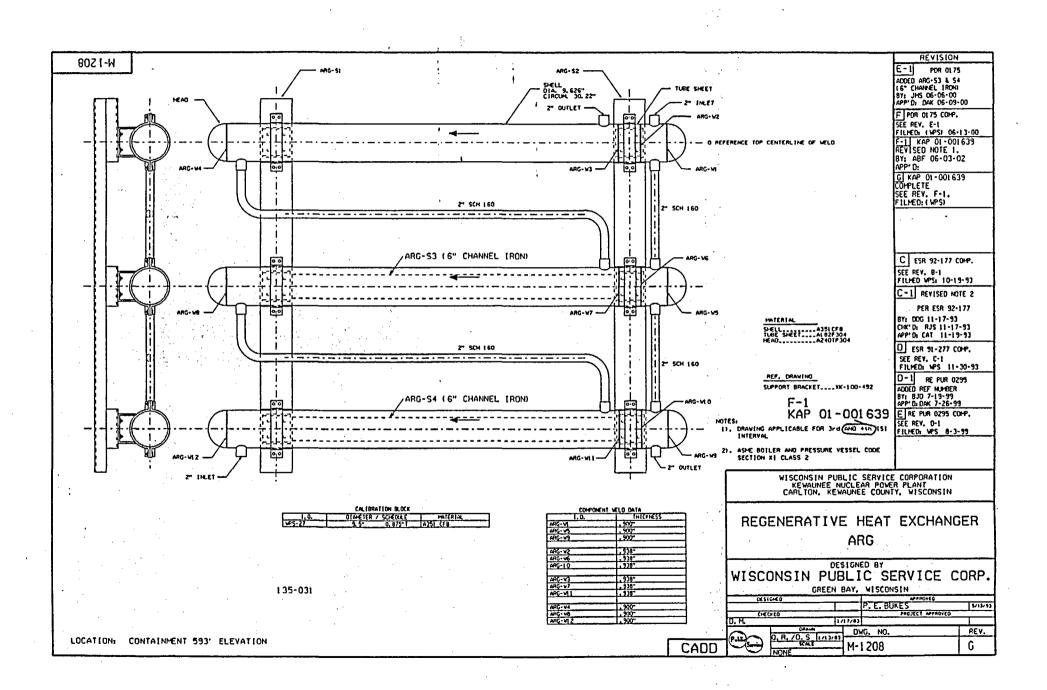
- 1. ASME Code Component Affected: Regenerative Heat Exchanger
 Head Circumferential Weld ARG-W9
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 1.5% of the Regenerative Heat Exchanger Head Circumferential Weld ARG-W9 was inaccessible due to configuration of the Regenerative Heat Exchanger 2" Nozzle thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 1.5% of the Head Circumferential Weld ARG-W9 would require modification from the Original Design of the Regenerative Heat Exchanger.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

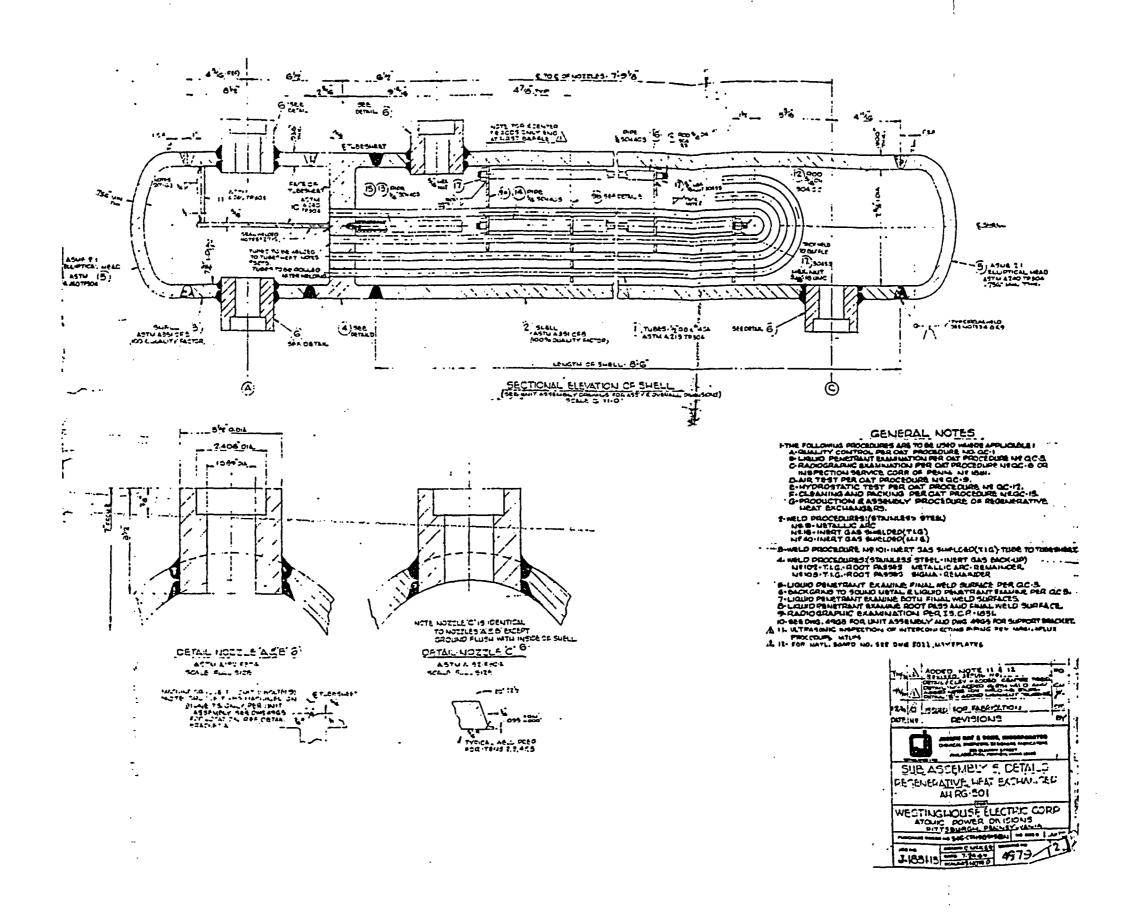
RELIEF REQUEST NO: RR-G-7-11

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

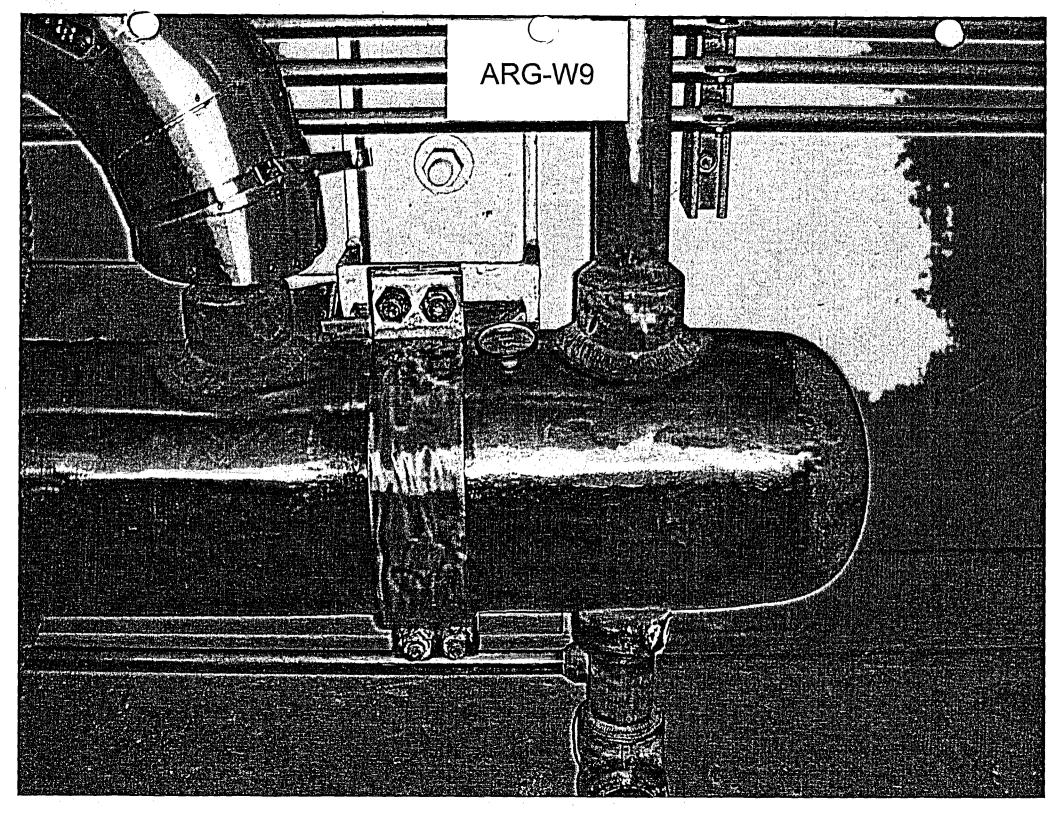
8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REGENERATIVE HEAT EXCHANGER ARG		
DRAWING NO.: _ M-1208		
COMPONENT IDENTIFICATION: ARG-W9 PROCEDURE: NEP NO.15.17 REVISION: ORIG		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: DATE: 11-11-98 LEVEL DATE: 11-11-98		
EXAMINER: NA LEVEL DATE: NA		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
WELO CROWN 1.0"		
k-4"->		
1.25"		
45° SCAN 2 LIMITED DUE TO NOZZLE CONFIGURATIONS ON TOP AND BOTTOM. REDUCED CODE COVERAGE 1.5% REDUCED PRIEDURAL COVERAGE8%		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 11,1998		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royan Marin DATE: 11-12-98		



(_,



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-12

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

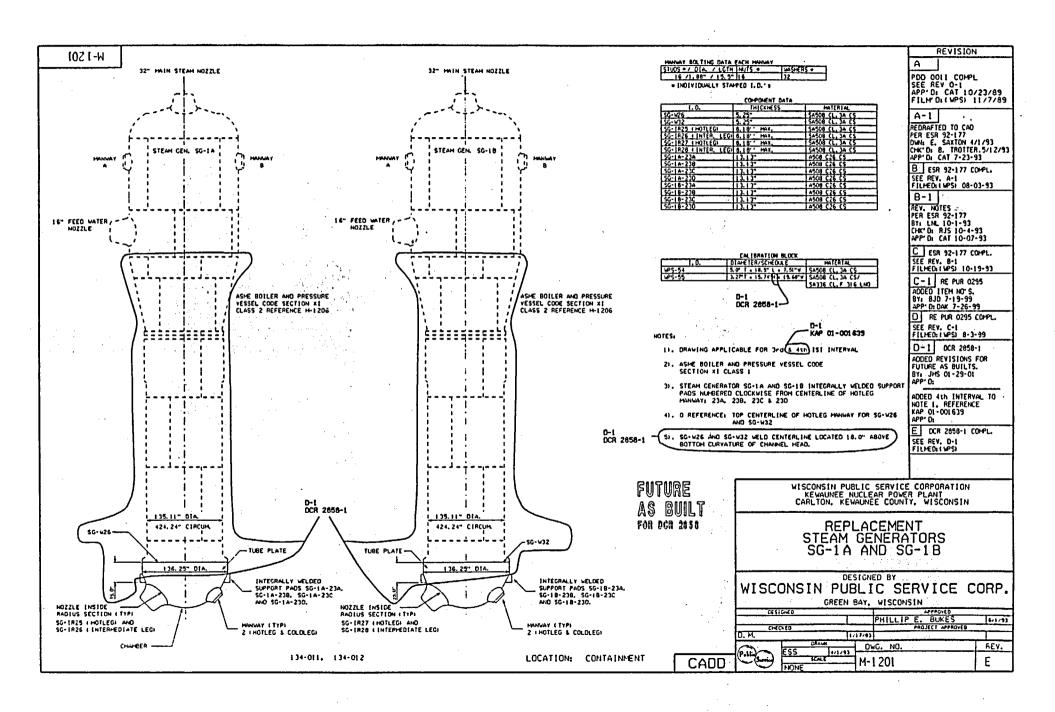
- 1. ASME Code Component Affected: Steam Generator 1A Nozzle Inside
 Radius Sections SG-IR25 and SG-IR26
 and Steam Generator 1B Nozzle Inside
 Radius Section SG-IR27 and SG-IR28
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-D; Item No. B3.140
- 4. Impracticality of Compliance: 6.3% of each of the 4 Nozzle Inside Radius Sections SG-IR25, SG-IR26, SG-IR27 and SG-IR28 were inaccessible due to the configuration of the Steam Generator Nozzle Inside Radius Section Integrally Welded Attachment thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 6.3% of each of the Steam Generator 1A and 1B Nozzle Inside Radius Sections would require modification from the Original Design of the Steam Generators and the Integrally Welded Attachments supporting the Steam Generators.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-12

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: VT-2 Table IWB-2500-1; Examination Category B-P; Item B15.10 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

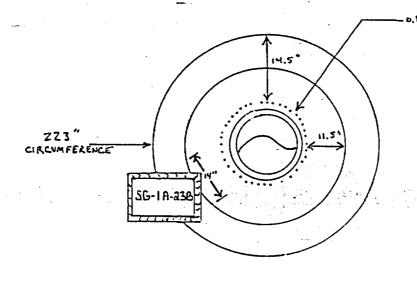


SYSTEM OR COMPONENT: REPLACEMENT STEAM GENERATORS SG-1A AND SG-1B
DRAWING NO.:
COMPONENT IDENTIFICATION: SG-IR 25 PROCEDURE: NEP-15.44 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: VISUAL:
EXAMINER: JEFF JOHNSON II DATE: 6-22-01 LEVEL
EXAMINER: TRAUIS THOMAS II DATE: 6-22-01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
14.0" OF INTEGRALLY WELDED ATTACHMENT SG-1A-23 D LIMITED 60° SCANS WITH CALIBRATION FOR NOTCH B AND NOTCH F
14.5
CIRCUMFERENCE CIRCUMFERENCE (11.5) REQUIRED VOLUME NOT EXAMINED 6.3 %
SG-1A-23 D
KEWAUNEE NUCLEAR POWER BLANT REVIEW: Phillips C. Bukes DATE: Numer 25 2001

SYSTEM OR COMPONENT: REPLACEMENT STEAM GENERATORS SG-1A AND SG-1B
DRAWING NO.:
COMPONENT IDENTIFICATION: SG-IR 26 PROCEDURE: NEP-15.44 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: JEFF JOHNSON II DATE: 6-22-01 EXAMINER: TRAVIS THOMAS II DATE: 6-22-01
EXAMINER: TRAVIS THOMAS II DATE: 6-22-01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND

PERCENTAGE OF REDUCED EXAMINATION COVERAGE.

14.0" OF INTEGRALLY WELDED ATTACHMENT SG-1A-23 B LIMITED 60° SCANS WITH CALIBRATION FOR NOTCH B AND NOTCH F



REQUIRED VOLUME NOT EXAMINED - 6.3%.

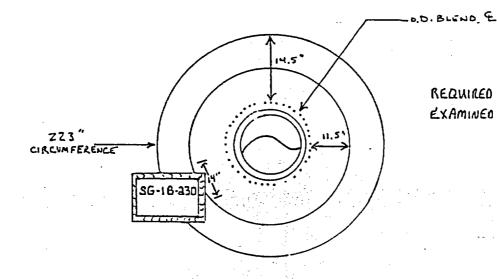
AUTHORIZED NUCLEAR
INSERVICE INSPECTOR REVIEW: Roga Mynin DATE: 6-25-0/

SYSTEM OR COMPONENT: REPLACEMENT STEAM GENERAT	ORS SG-IA AND SG-IB
DRAWING NO.: M- 1201	
COMPONENT IDENTIFICATION: SG-IR 27 PROCEE	DURE: <u>NEP-15.44</u> REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNET	FIC PARTICLE: VISUAL:
f m fol	
EXAMINER: JEFF JOHNSON II	DATE: <u>6-23-01</u>
EXAMINER: JEFF JOHNSON II LEVEL	
EXAMINER: TRAUS THOMAS II	DATE: <u>6-23-01</u>
LEVE	L

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.

14.0" OF INTEGRALLY WELDED ATTACHMENT SG-16-23 D LIMITED

60° SCANS WITH CALIBLATION FOR NOTCH B AND F.



REQUIRED VOLUME NOT EXAMINED - 6.3 1.

AUTHORIZED NUCLEAR
INSERVICE INSPECTOR REVIEW: Rogn Within

DATE: June 25,2001

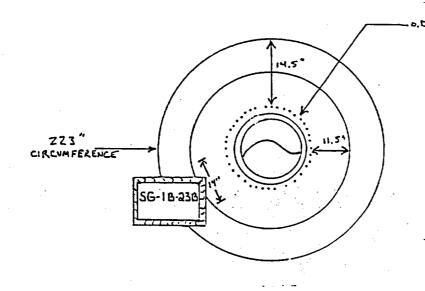
DATE: June 25,2001

DATE: 6-25-01

SYSTEM OR COMPONENT: REPLACEMENT STEAM GENERATORS SG-IA AND SG-IB
DRAWING NO.:
COMPONENT IDENTIFICATION: SG-IR 28 PROCEDURE: NEP- 15.44 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: JEFF JOHNSON II DATE: 6-23-01
Trans Thomas LEVEL
EXAMINER: TRAVIS THOMAS IT DATE: 6-23-01 LEVEL

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.

14.0" OF INTEGRALLY WELDED ATTACHMENT SG-1B- 23 B LIMITED 60° SCANS WITH CALIBRATION FOR NOTCH B AND NOTCH F



REQUIRED VOLUME NOT EXAMINED 6.3%

KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phil	les C. Bukes	DATE: June 25,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: _	Roan Mynen	DATE: 6-25-0/

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-13

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

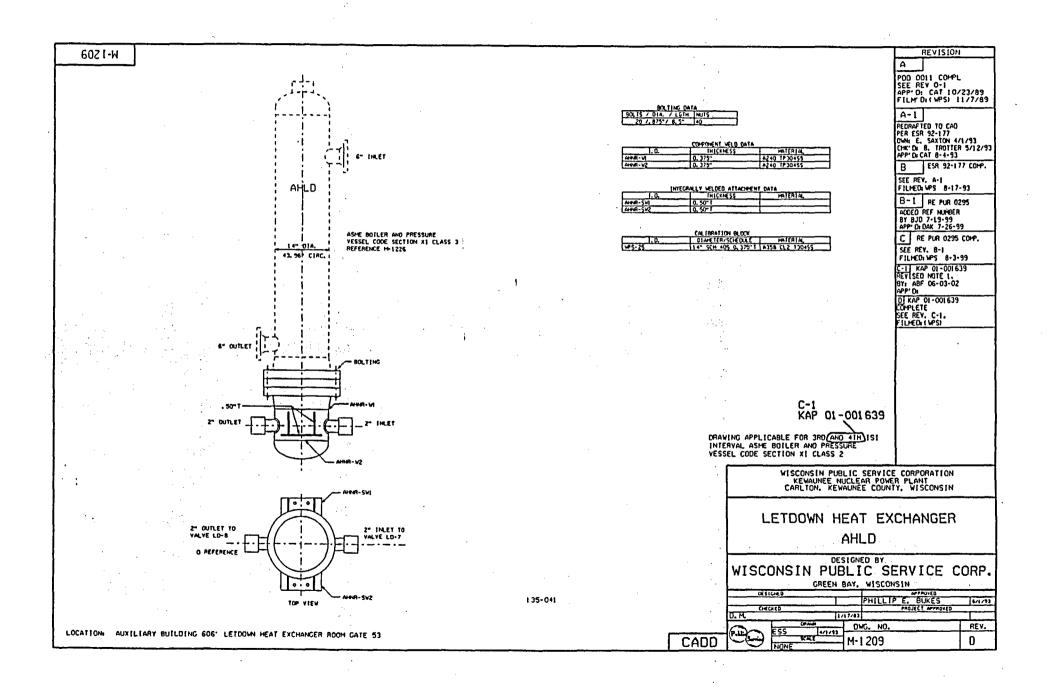
- 1. ASME Code Component Affected: Letdown Heat Exchanger
 Head Circumferential Weld AHNR-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 43.0% of the Letdown Heat Exchanger Head Circumferential Weld AHNR-W2 was inaccessible due to configuration of the Letdown Heat Exchanger 2 Welded Supports and the 2" Inlet Nozzle and the 2" Outlet Nozzle thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 43.0% of the Head Circumferential Weld AHNR-W2 would require modification from the Original Design of the Letdown Heat Exchanger.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

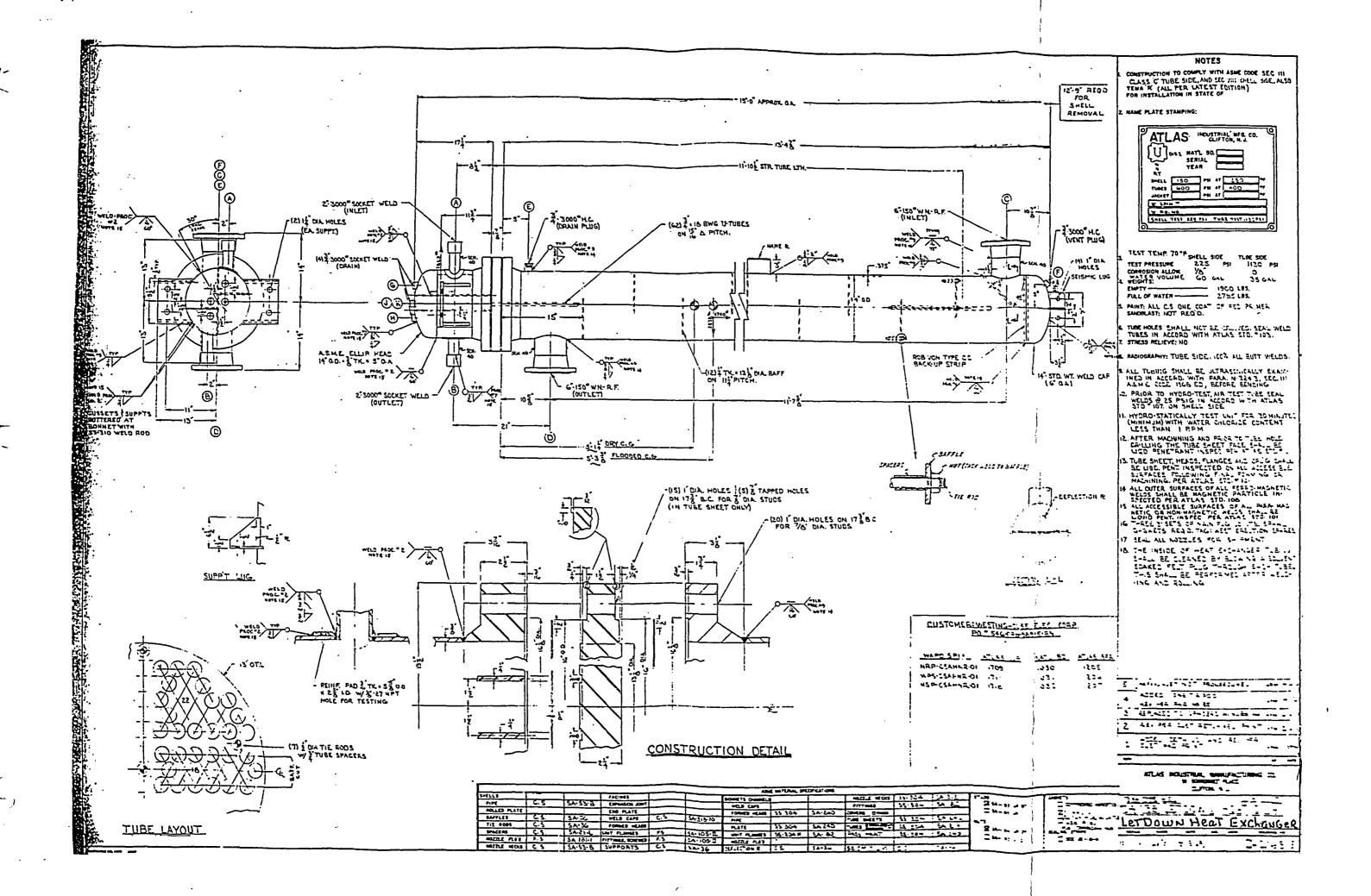
RELIEF REQUEST NO: RR-G-7-13

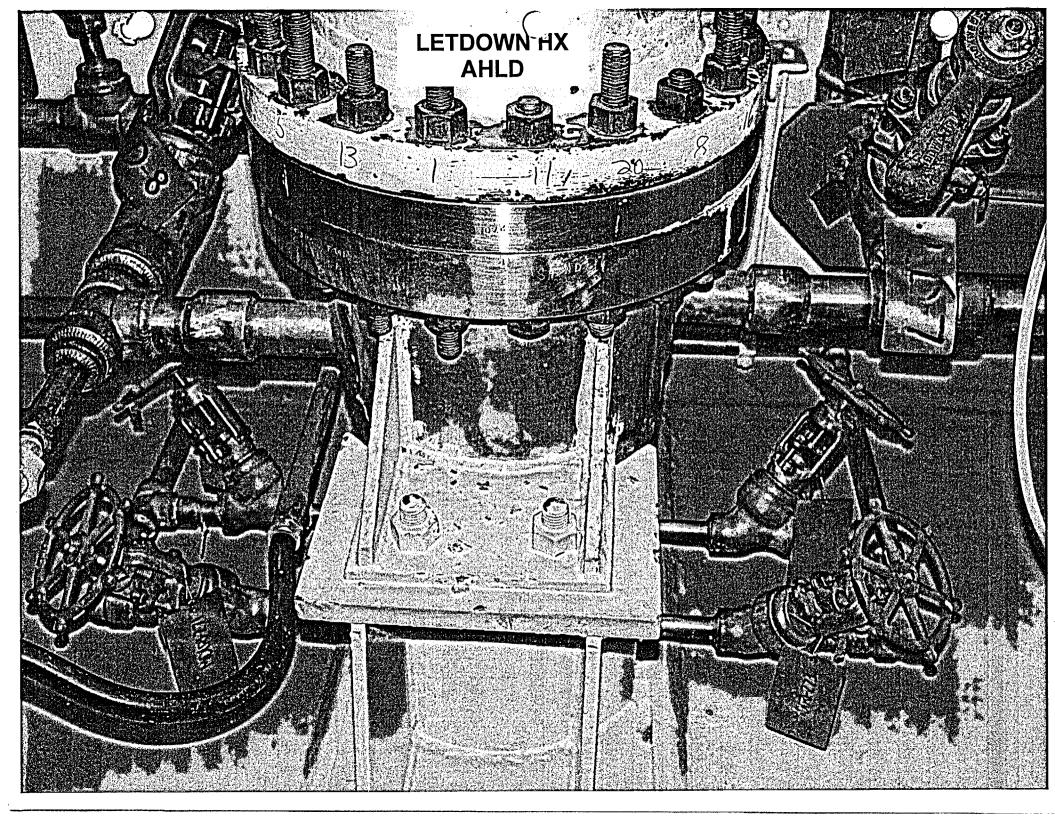
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: LET DOWN HEAT EXCHANGER AHLD		
DRAWING NO.: _M-1209		
COMPONENT IDENTIFICATION: AHNR - W2 PROCEDURE: NEP-15.41 REVISION: A		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: TIM COBURN AM IL DATE: 10/24/01 LEVEL		
EXAMINER: NA NA DATE: NA LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. 2" OUTLET LIMITS WELD AHNR - W2 FROM 42" - 44". WELDED SUPPORT LIMITS WELD AHNR - W2 FROM 15" - 20" AND 30 - 42". 45" - RESTRICTED SCAN 2.5.7.8 DUE TO WELDED SUPPORTS.		
45 - RESTRICTED SCAN 5 DUE TO INLET AND OUTLET 2" LINES.		
REDUCED CODE COVERAGE BY 43%.		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes DATE: Otoler 29,2001 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royn Whymin DATE: 10/29/01		





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-14

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

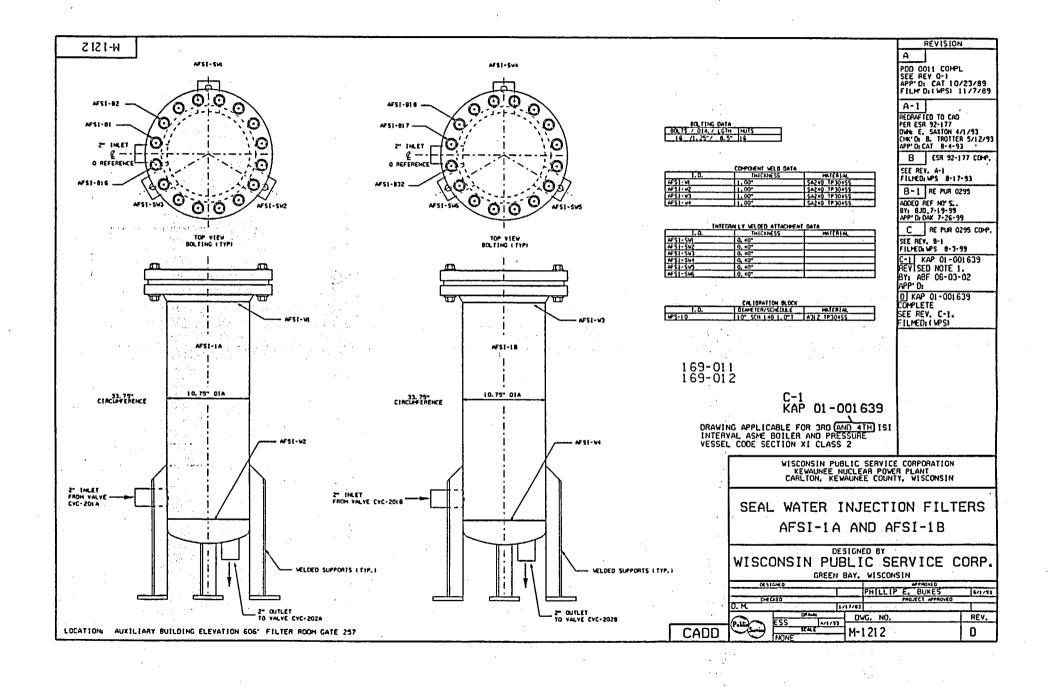
- 1. ASME Code Component Affected: Seal Water Injection Filter 1A
 Shell Circumferential Weld AFS1-W1
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.10
- 4. Impracticality of Compliance: 67.06% of the Seal Water Injection Filter 1A Shell Circumferential Weld AFSI-W1 was inaccessible due to configuration of the Seal Water Injection Filter 1A Flange to Shell Configuration, Flange Cover Hinge Plate, Weld Crown and Welded Name Plate thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 67.06% of the Shell Circumferential Weld AFSI-W1 would require modification from the Original Design of the Seal Water Injection Filter 1A.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

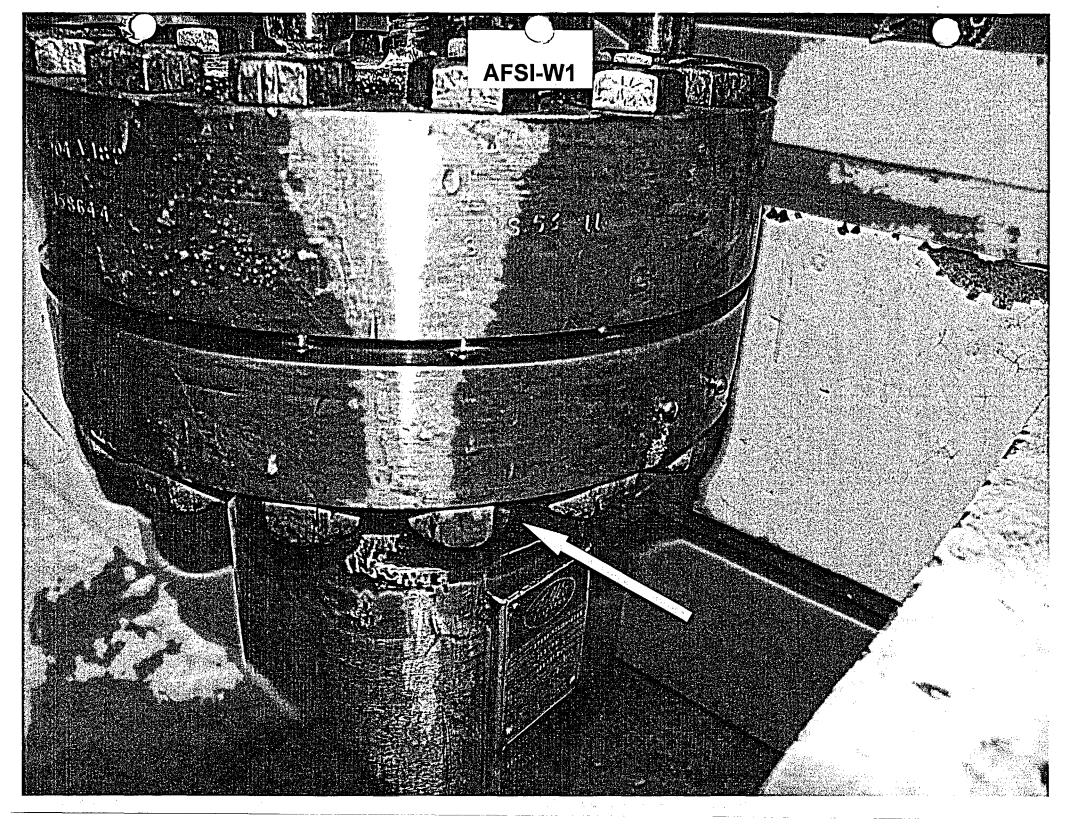
RELIEF REQUEST NO: RR-G-7-14

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. However, during the Kewaunee Nuclear Power Plant 2nd Interval 1984-1994; 100% Radiography in 1988, 100% Visual From the I.D. in 1988 and 100% Liquid Penetrant From the I.D. in 1988 were performed on Seal Water Injection Filter 1A Shell Circumferential Weld AFSI-W1. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



OVOTEM OF COMPONENT Communication of the Arch	LA AVE AFOT IS
SYSTEM OR COMPONENT: SEAL WATER INJECTION FILTERS AFSI:	- IA AND AFST-IR
DRAWING NO.:	
COMPONENT IDENTIFICATION: AFST-WI PROCEDURE: NEP-15	5.41 REVISION: A
ULTRASONIC:X LIQUID PENETRANT: MAGNETIC PARTICLE:	VISUAL:
EXAMINER: Bria B. Kustt II DATE:	
EXAMINER: <u>VA</u> DATE:	<i>NA</i>
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TY PERCENTAGE OF REDUCED EXAMINATION COVER	PE OF LIMITATION AND IAGE.
FLANCE COVER HINGE PLATE CONFIGURATION AFSI-WI - FLANGE CONFI SCANS FOR 45° RL. WELD C SCANS FOR 45° RL. FLANGE LOCATED FROM 16.0" CW REFERENCE LIMITS 2,748 NAME PLATE LOCATED FROM FROM O REFERENCE 0.9° E AFSI-WI LIMITS 2 SCAN 67.06% PROCEDURE CODE OREF.	CROWN LIMITS 25,7+8 COUER HINGE PLATE TO 19.0" CW FROM O SCANS FOR 45°RL 24.0" CW TO 18.0" CW TROM TAE OF WELD FOR 45°RL
REWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes	DATE: November 1, 2001
AUTHORIZED NUCLEAR NOSPINIOS INSPECTOR DEVIEW	



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-15

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Seal Water Injection Filter 1A

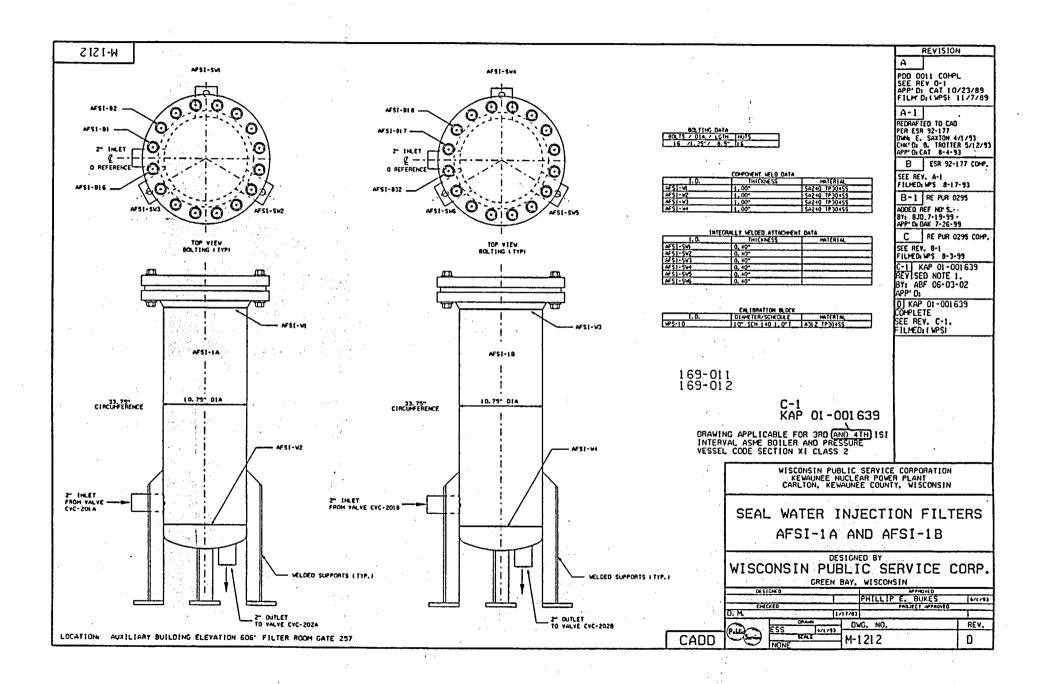
 Head Circumferential Weld AFS1-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 35.8% of the Seal Water Injection Filter 1A Head Circumferential Weld AFSI-W2 was inaccessible due to configuration of the Seal Water Injection Filter 1A 3 Welded Supports and the 2" Inlet Nozzle thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 35.8% of the Head Circumferential Weld AFSI-W2 would require modification from the Original Design of the Seal Water Injection Filter 1A.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-15

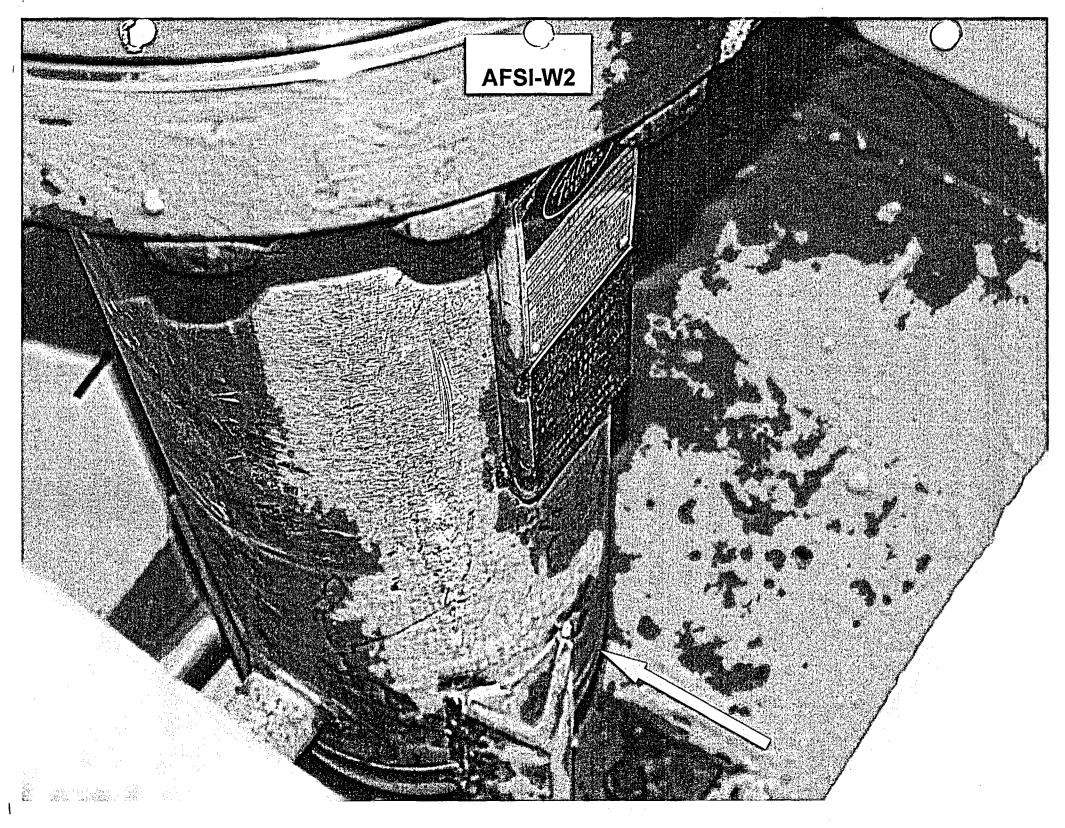
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. However, during the Kewaunee Nuclear Power Plant 2nd Interval 1984-1994; 100% Radiography in 1988 and 100% Visual From the I.D. in 1988 were performed on Seal Water Injection Filter 1A Head Circumferential Weld AFSI-2. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD
SYSTEM OR COMPONENT: SEALWATER INJECTION FILTERS AFSI-IA AND AFSI-18
DRAWING NO.: <u>M - 1212</u>
COMPONENT IDENTIFICATION: AFSI-WZ PROCEDURE: NEPNo. 15. 16 REVISION: Orig.
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: VISUAL:
EXAMINER:
EXAMINER:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
PERCENT OF LIMITED CODE COVERAGE: 35.8%
PERCENT OF LIMITED PROCEDURE COVERAGE: 35.9%
45° AND 60° SCANS LIMITED BY 2" INLET NOZZLE 1.3" FROM TOE OF IA-2 (AFSI-W2).
O° 45° AND 60° SCANS LIMITED BY
4.25 INCHES FROM AFSI-SWI, AFSI-SWZ
AFSI-WZ AND AFSI-SW3 FOR A TOTAL OF
12.75"
AFSI-SW2
AFSI-SWI
AFSI-SW3
POWER PLANT REVIEW: Phillip C. Bukes DATE: November 16,1998

AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-16

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

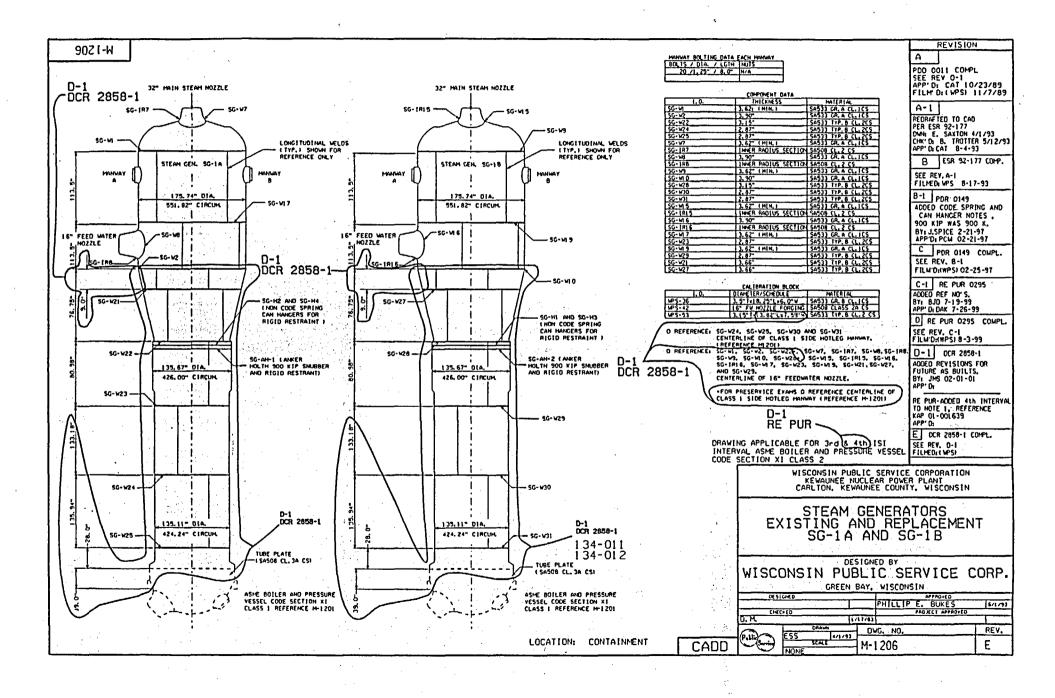
- 1. ASME Code Component Affected: Steam Generator 1A
 Shell Circumferential Weld SG-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.10
- 4. Impracticality of Compliance: 11.1% of the Steam Generator 1A Shell Circumferential Weld SG-W2 was inaccessible due to configuration of the Steam Generator 1A 4 Welded Pads and the configuration of the Weld Crown thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 11.1% of the Shell Circumferential Weld SG-W2 would require modification from the Original Design of Steam Generator 1A.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-16

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage. Magnetic Particle Examination of 100% of the I.D. was performed during the 2001 Steam Generator Replacement for evidence of I.D. Cracking.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: STEAM GENERATORS EXISTING AND REPLACEMENT 36-1A AND 56-1B
DRAWING NO :
COMPONENT IDENTIFICATION: 56-W2 PROCEDURE: NEP-15.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TIM COBURN AMM I DATE: 11/10/01
EXAMINER: MILO DATE: 11/10/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
O REF.
1. 28'3.5' 1. 16'9.5" 1. 5'4" ///////////////////////////////////
2 SIDE
1. FOUR WELDED PADS 7.75" X ID.75" MEASURED INTHE CW DIRECTION FROM DREF. LOCATED ON THE WELD TOE ON THE 5 SIDE OF THE WELD LIMITS 57 AND 9 SCANS.
2. WELD CROWN LIMITS SCANS 2,5,7 AND 8 FOR THE 60° 3. REDUCED CODE / PROCEDURE COVERAGE BY 0.5% FOR SCANS 7 AND 8 DUE TO WELDED PADS.
4. REDUCED CODE / PROCEDURE COVERAGE BY 0.5% FOR SCAN 5.
5. REDUCED CODE / PROCEDURE COVERAGE BY 0.17. FOR SCAN 2.
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 17,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ann Material DATE: 11-20-01

Existing and REPLACEMENT SYSTEM OR COMPONENT: STEAM GENERATORS SG-1A AND SG-1	D.
DRAWING NO.:	·
COMPONENT IDENTIFICATION: SG-W2 PROCEDURE: NEP-1	5.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE	: VISUAL:
EXAMINER: Rear d. Knott II DATE LEVEL EXAMINER: II DATE LEVEL	: <u>11/10/01</u> : <u>11/4/01</u> : <u>11/10/01</u>
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TO PERCENTAGE OF REDUCED EXAMINATION COVE	
1. <u>29</u> '9.5" 1. <u>28'3.5</u> 1. <u>16'9.5"</u> 1.	5 SIDE
Four Welded Pads 7.75" X ID.75" MEASURED INTHE CW DIRECTI LOCATED ON THE WELD TOE ON THE 5 SIDE OF THE WELD LI	ON FROM OREF.
	MITS 57 ANDS SCANS.
2. WELD CROWN LIMITS SCANS 2,5,7 AND 8 FOR THE 45° 3. 45°- REDUCED CODE/PROCEDURE COVERAGE BY 7%	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes	DATE: November 17,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Kogn Manue	DATE: 11-20-01

SYSTEM OR COMPONENT: STEAM GENERATORS EXISTING AND REPLACEMENT SG-IA AND SG-IB
DRAWING NO:
COMPONENT IDENTIFICATION: <u>\$6-ω2</u> PROCEDURE: <u>NEP-15.09</u> REVISION: <u>A</u>
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TIM COBURN Mu. M II DATE: 11/10/01
EXAMINER: My DATE: 11 10 01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
O REF.
1. 39'9.5" 1. 28'3.5" 1. 16'9.5" 1. 5"4" //////////////////////////////////
2 SIDE
Four Welded Pads 7.75" X 10.75" MEASURED INTHE CW DIRECTION FROM OREF. LOCATED ON THE WELD TOE ON THE 5 SIDE OF THE WELD
2. 0° - REDUCED CODE / PROCEDURE COVERAGE BY 3% ON 5 SIDE.
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: November 17, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Mynum DATE: 11-30-01

KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-17

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

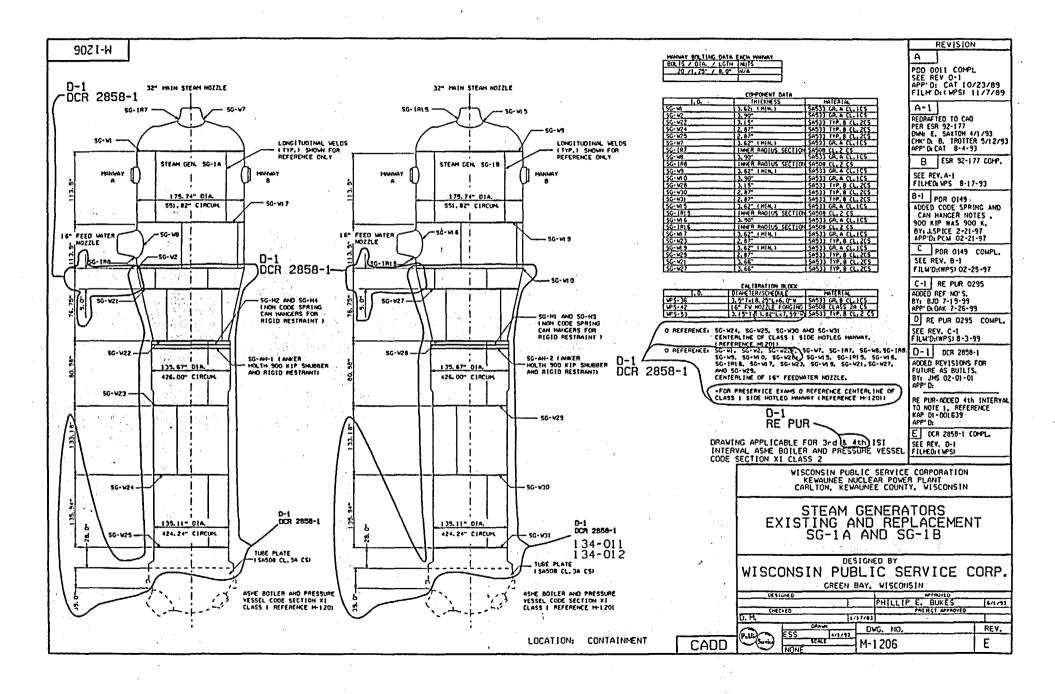
- 1. ASME Code Component Affected: Steam Generator 1B Shell Circumferential Weld SG-W10
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.10
- 4. Impracticality of Compliance: 7.9% of the Steam Generator 1B Shell Circumferential Weld SG-W10 was inaccessible due to configuration of the Steam Generator 1B 4 Welded Pads, 3 2" Nozzles and the configuration of the Weld Crown thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 7.9% of the Shell Circumferential Weld SG-W10 would require modification from the Original Design of Steam Generator 1B.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-17

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage. Magnetic Particle Examination of 100% of the I.D. was performed during the 2001 Steam Generator Replacement for evidence of I.D. Cracking.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: STEAM GENERATORS Existing AND REPLACE	LEMENT SG-1A ANDSG-B
DRAWING NO .: M - 120 6	
COMPONENT IDENTIFICATION: SG-ω10 PROCEDURE: NEP-1	15.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE	:: VISUAL:
EXAMINER: Brian B. Knoth II DATE LEVEL DATE	
EXAMINER: My LEVEL DATE	<u> </u>
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, T PERCENTAGE OF REDUCED EXAMINATION COVE	YPE OF LIMITATION AND PAGE. O REFERENCE TED AT CENTER OF FEEDWARD ON NOTHER
5 SIDE (5) (4) 7 (6) (7) (3) (4) (4) (4) (4) (4) (4) (4) (4) (4) (4	2
2 SIDE	
1. WELDED PAD 73/4" x 103/4" LOCATED AT 5'4" CW FROM O PO WELD LIMITS 0°, 45° + 60° SCANS * 2. 2" NOTZLE LOCATED AT 13'4"2" CW FROM O REFERENCE 3. WELDED PAD 72/4" x 103/4" LOCATED AT 16' 9'2" CW FROM 5 SIDE OF WELD LIMITS 0°, 45° + 60° SCANS * 4. 2" NOTZLE LOCATED AT ZI'1" CW FROM O REFERENCE WELD OU CONTENT AT ZI'1" CW FROM O REFERENCE	m O REFERENCE ON
WELD ON S SIDE LIMITS GOO SCAN 5. 2" NOTTLE LOCATED AT 24' 11" CW FROM O REFERENCE WELD ON S SIDE LIMITS GOO SCAN	6314" FROM TOE OF
6. WELDED PAD 73/4" x 103/4" LOCATED ATZ8' 3"2" CW FROM SIDE OF WELD LIMITS 0", 450 + 600 SCANS *	
7. WELPED PAD 7314" x 10214" LOCATED AT 39' 91/2" CW F S SIDE DE WELD LIMITS 00, 450 + 600 SCANS *	From O REFERENCE ON
8. WELD CROWN LIMITS 00, 45, 4 60° SCANS	7.9% OF REQUIRED VOLUME NOT EXAMINED
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes	DATE: NW.20,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Logs Whym	_ DATE:

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-18

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

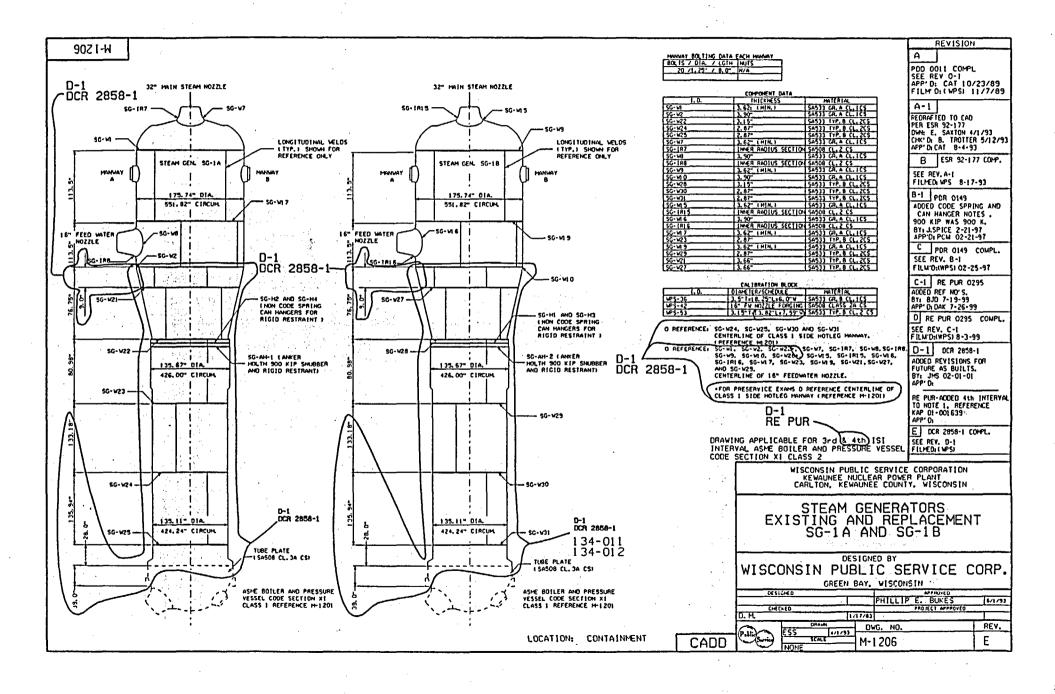
- 1. ASME Code Component Affected: Steam Generator 1B Head Circumferential Weld SG-W9
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.20
- 4. Impracticality of Compliance: 0.113% of the Steam Generator 1B Head Circumferential Weld SG-W9 was inaccessible due to configuration of the Steam Generator 1B 4 Welded Pads thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 0.113% of the Head Circumferential Weld SG-W9 would require modification from the Original Design of Steam Generator 1B.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-18

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



EXISTING AND REPLACEMENT SYSTEM OR COMPONENT: STEAM GENERATORS SG-(A AND SG- IB
DRAWING NO .: _ M-1206
COMPONENT IDENTIFICATION: 36-W9 PROCEDURE: NEP-15:09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: VISUAL:
EXAMINER: July Tim COBURN II DATE: 10/6/01 EXAMINER: DATE: 10/6/01 LEVEL DATE: 10/6/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
JSG-W9 SCAN 2 SIDE O REFERENCE LOCATED AT CENTER OF FEEDWATER NO22LE I. WELDED PAD 2.7" x 2.7" LOCATED 60° FROM O REFERENCE ON S SIDE. 2. WELDED PAD 2.7" x 2.7" LOCATED 150° FROM O REFERENCE ON S SIDE. 3. WELDED PAD 2.7" x 2.7" LOCATED 240° FROM O REFERENCE ON S SIDE. 4. WELDED PAD 2.7" x 2.7" LOCATED 330° FROM O REFERENCE ON S SIDE. ALL PADS 1.6" FROM TDE OF WELD ON S SIDE. SCAN S LIMITED ON THE 45° AND LO° DUE TO WELDED PADS. REDUCED CODE COVERAGE BY 0.113%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: October 16, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: American DATE: 10-18-01

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-19

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

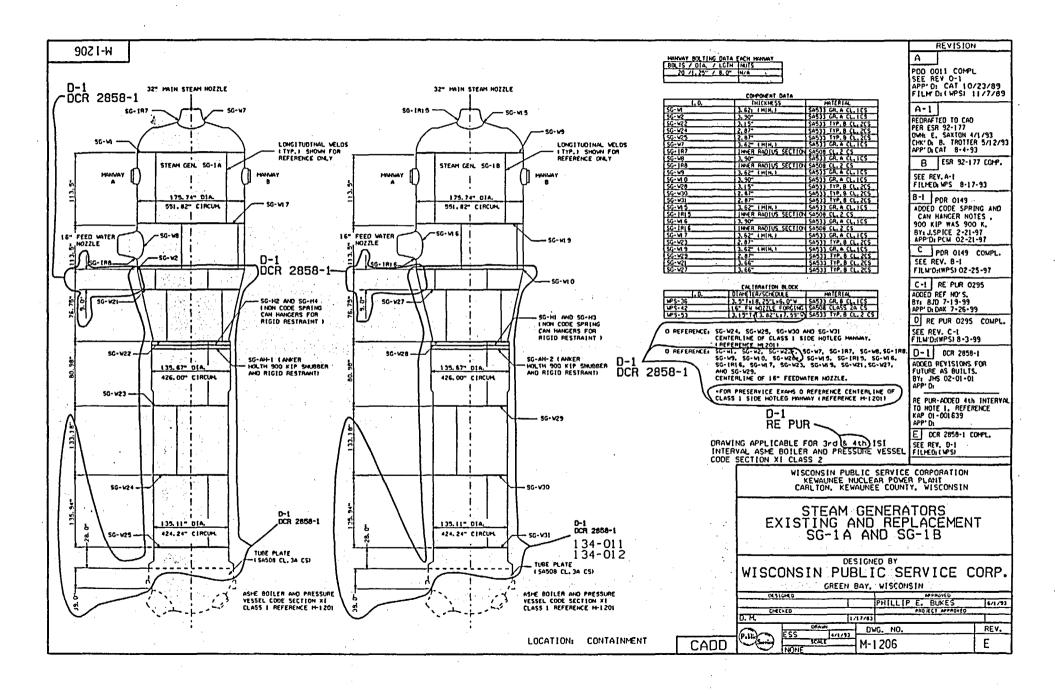
- 1. ASME Code Component Affected: Steam Generator 1A Tubesheet -To-Shell Circumferential Weld SG-W25 and Steam Generator 1B Tubesheet -To-Shell Circumferential Weld SG-W31
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.30
- 4. Impracticality of Compliance: 9.0% of the Steam Generator 1A Tubesheet-To-Shell Circumferential Weld SG-W25 and 9.0% of the Steam Generator 1B Tubesheet -To-Shell Circumferential Weld SG-W31 were inaccessible due to each weld possessing 3 Handholes, 2-4" Nozzles, 1-2 1/4" Nozzle and 4-2" Nozzles thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 9.0% of the Steam Generator 1A Tubesheet-To-Shell Circumferential Weld SG-W25 and 9.0% of Steam Generator 1B Tubesheet-To-Shell Circumferential Weld SG-W31 would require modification of the design of Steam Generator 1A and Steam Generator 1B.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-19

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage. ASME Boiler and Pressure Vessel Code Section III Radiography was performed as part of Kewaunee Nuclear Power Plant Steam Generator Replacement during the 2001 Refueling Outage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT

KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: STEAM GENERATORS EXISTING AND REPLACEMENT SG-1A & SG-1B
DRAWING NO.:
COMPONENT IDENTIFICATION: SG-WAS PROCEDURE: NEP-15.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: VISUAL:
EXAMINER: TRAVIS THOMAS IL DATE: 6-8-01 LEVEL EXAMINER: DEFF JOHNSON II DATE: 6-8-01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. 5
I HANDHOLE 38"-48", 2.5" FROM TOE OF WELD 5 SIDE. 2 4" NOZZLE 35"-42", 2.5" FROM TOE OF WELD 2 SIDE. 3 HANDHOLE 135"-156", 2.0" FROM TDE OF WELD 5 SIDE. 4 2" NOZZLE 179"-183", @ TOE OF WELD 5 SIDE. 5 2" NOZZLE 214"-218", @ TOE OF WELD 5 SIDE. 6 4" NOZZLE 249"-255", 2.5" FROM TOE OF WELD 2 SIDE. 8. 2" NOZZLE 249"-255", 2.5" FROM TOE OF WELD 5 SIDE. 8. 2" NOZZLE 285", 287", @ TOE OF WELD 5 SIDE. 9. 4" NOZZLE 285"-287", @ TOE OF WELD 5 SIDE. 10. 2" NOZZLE 386"-391". 2.5" FROM TOE OF WELD 5 SIDE. 10. 2" NOZZLE 386"-391". 2.5" FROM TOE OF WELD 5 SIDE. 10. 2" NOZZLE 386"-391". 2.5" FROM TOE OF WELD 5 SIDE. 10. 2" NOZZLE 386"-391". 2.5" FROM TOE OF WELD 5 SIDE. 10. 2" NOZZLE 386"-391". 2.5" FROM TOE OF WELD 5 SIDE. 3.5" FROM TOE OF WELD 5 SIDE. 2.5" FROM TOE OF WELD 5 SIDE. 3.5" FROM TOE OF WELD 5 SIDE. 4
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Proprint DATE: 6-13-01

REV.: ORIG.

SYSTEM OR COMPONENT: STEAM GENERATORS EXISTING AND REPLACEMENT SG-IA AND SG-18
DRAWING NO.: _ M - 1206
COMPONENT IDENTIFICATION: SG-W31 PROCEDURE: NEP-15.09 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TW THOMAS Inflorm II DATE: 6-14-01 EXAMINER: TW THOMAS Inflorm I DATE: 6-14-01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. 5
HANDHOLE 28"-48", 2.5" FROM TOE OF WELD 5 SIDE. 4" NOZZLE 35"-42", 2.5" FROM TOE OF WELD 2 SIDE. 3 HANDHOLE 135"-156", 2.0" FROM TOE OF WELD 5 SIDE. 4" NOZZLE 179"-183", Q TOE OF WELD 5 SIDE. 5" NOZZLE 219"-215", Q TOE OF WELD 5 SIDE. FROM TOE OF WELD 2 SIDE OF WELD 5 SIDE. THANDHOLE 2411"-215", Q TOE OF WELD 5 SIDE. THANDHOLE 2411"-364", 2.5" FROM TOE OF WELD 2 SIDE OF WELD 5 SIDE. A" NOZZLE 285"-289", Q TOE OF WELD 5 SIDE. O
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: June 16,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-20

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

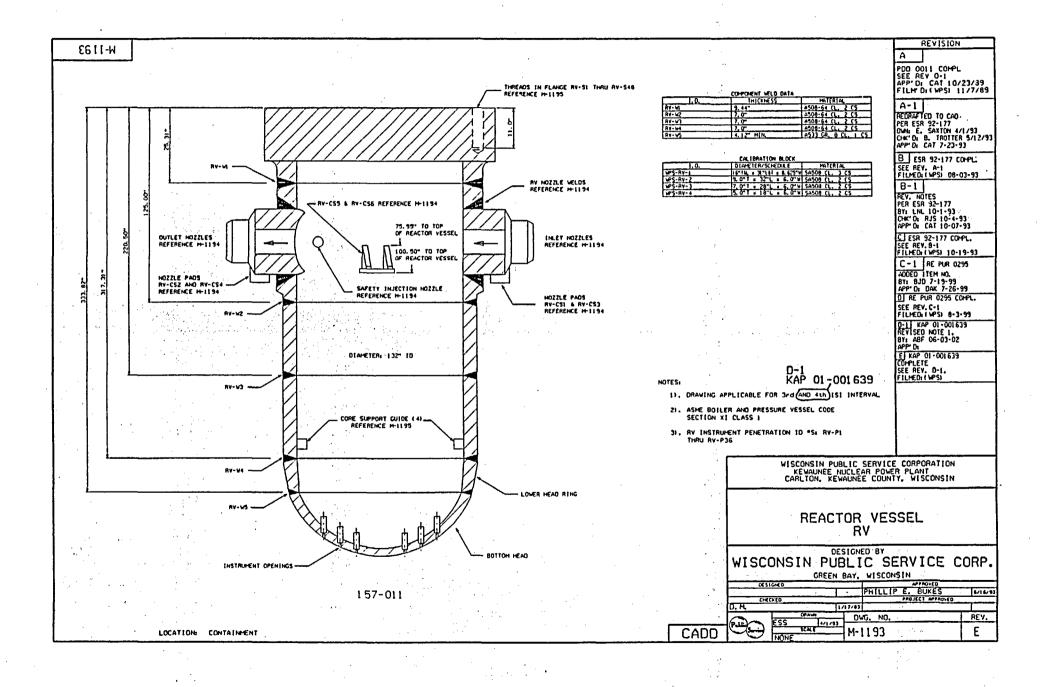
- 1. ASME Code Component Affected: Reactor Vessel Outlet Nozzle To Vessel Weld RV-W7
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-D; Item No. B3.90
- 4. Impracticality of Compliance: 2.84% of the Remote Ultrasonic Perpendicular Scan and 56.76% of the Remote Ultrasonic Tangential Scan of the Reactor Vessel Outlet Nozzle to Vessel Weld RV-W7 were inaccessible due to the Outlet Nozzle Boss Radius and the Outlet Nozzle Boss thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 2.84% Perpendicular Scan and 56.76% of the Tangential Scan of the Reactor Vessel Outlet Nozzle to Vessel Weld RV-W7 would require modification of the original design of Reactor Vessel Outlet Nozzle.

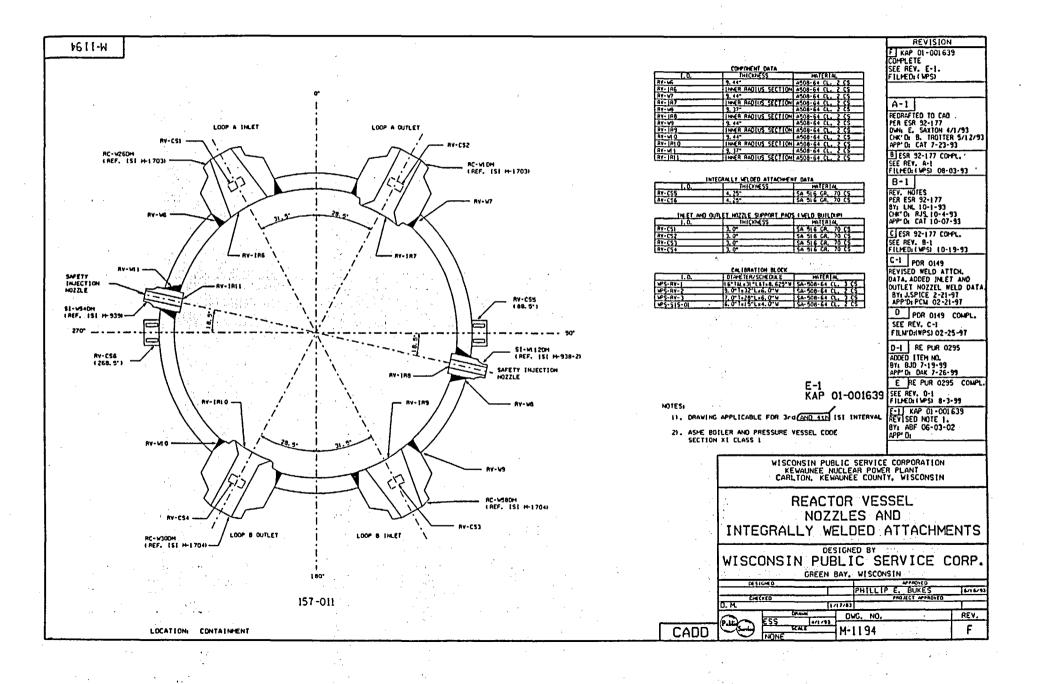
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

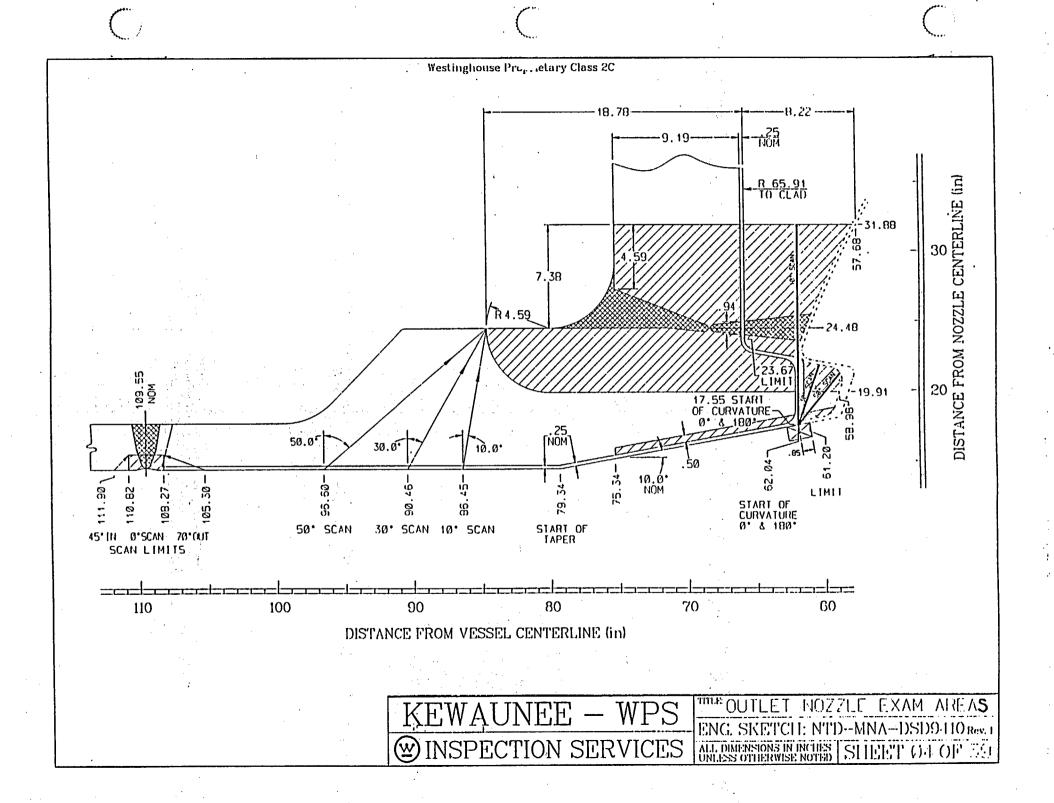
RELIEF REQUEST NO: RR-G-7-20

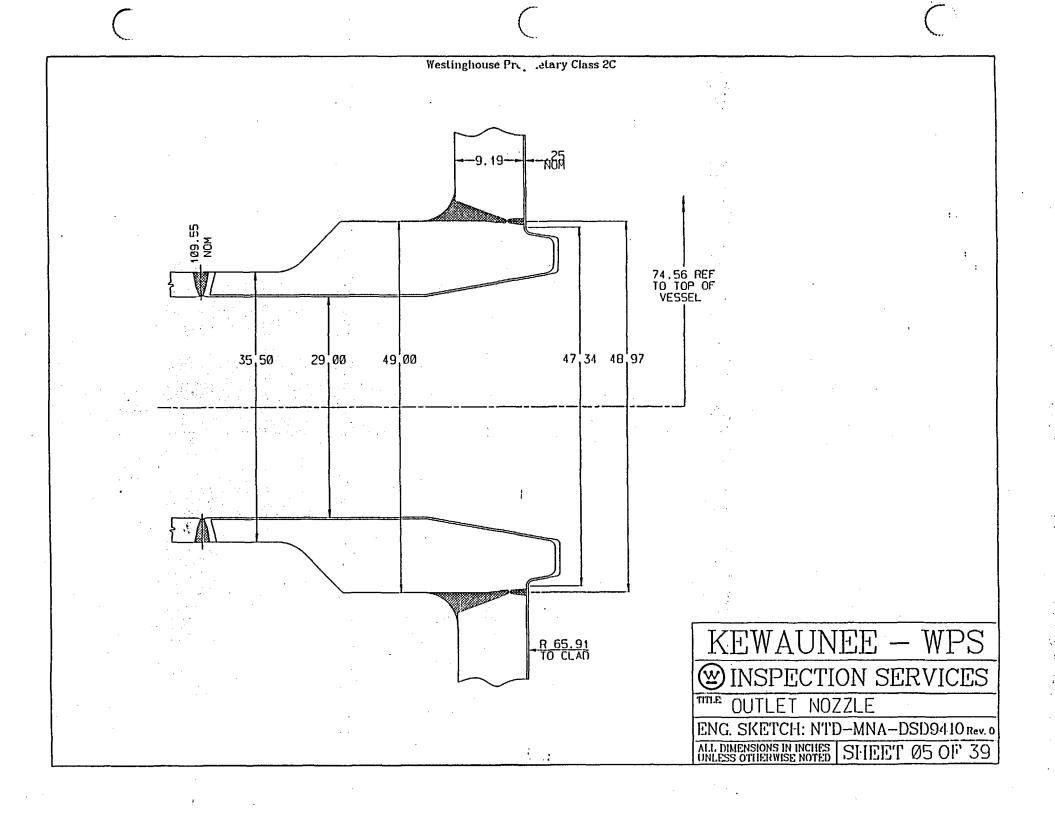
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable









Wisconsin Public Service Corporation Kewaunee Nuclear Power Plant 1st Outage; 1st Period; 3rd Interval Automated Reactor Vessel Tool Examination Coverage

Program Item	Weld Number	Description	Direction	% Coverage	Limitations
2	RV-W7	Loop A Outlet Nozzle to Vessel Weld	Perpendicular Tangential		Nozzle Boss Radius Nozzle Boss
3	RV-W10	Loop B Outlet Nozzle to Vessel Weld	Perpendicular Tangential		Nozzle Boss Radius Nozzle Boss
4	RV-IR7	Loop A Outlet Nozzle Inside Radius Section	Circumferential	100.00%	
5	RV-IR10	Loop B Outlet Nozzle Inside Radius Section	Circumferential	100.00%	
6	RC-W1DM		Parallel Perpendicular	100.00% 100.00%	
7	RC-W30DM		Parallel Perpendicular	100.00% 100.00%	

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-21

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

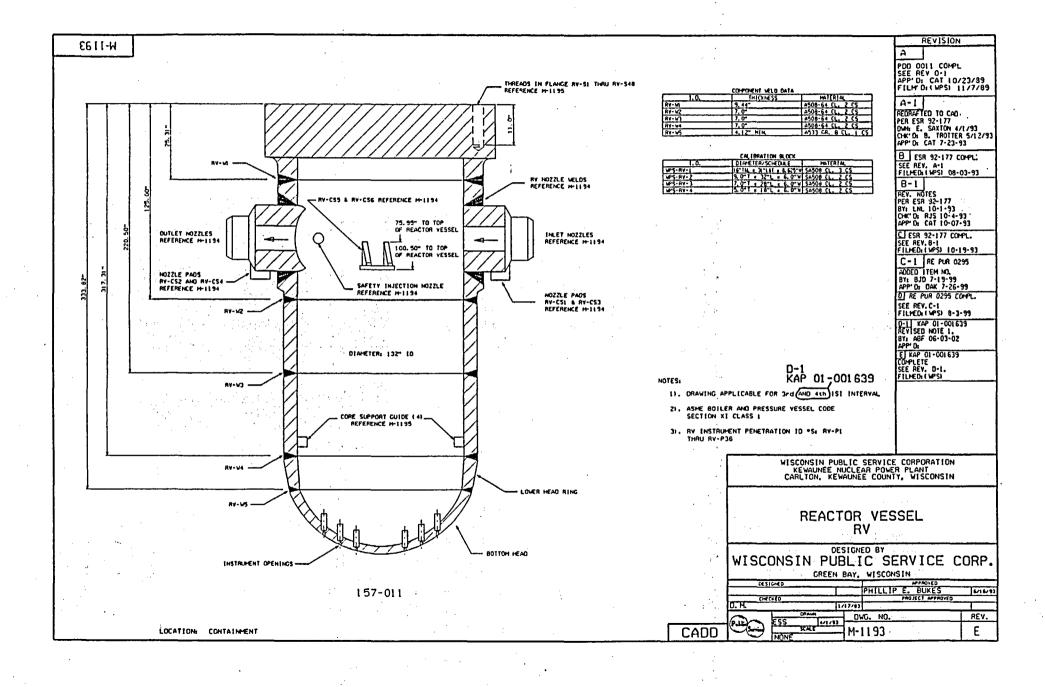
- 1. ASME Code Component Affected: Reactor Vessel Outlet Nozzle To Vessel Weld RV-W10
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-D; Item No. B3.90
- 4. Impracticality of Compliance: 2.84% of the Remote Ultrasonic Perpendicular Scan and 56.76% of the Remote Ultrasonic Tangential Scan of the Reactor Vessel Outlet Nozzle to Vessel Weld RV-W10 were inaccessible due to the Outlet Nozzle Boss Radius and the Outlet Nozzle Boss thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 2.84% Perpendicular Scan and 56.76% of the Tangential Scan of the Reactor Vessel Outlet Nozzle to Vessel Weld RV-W10 would require modification of the original design of Reactor Vessel Outlet Nozzle.

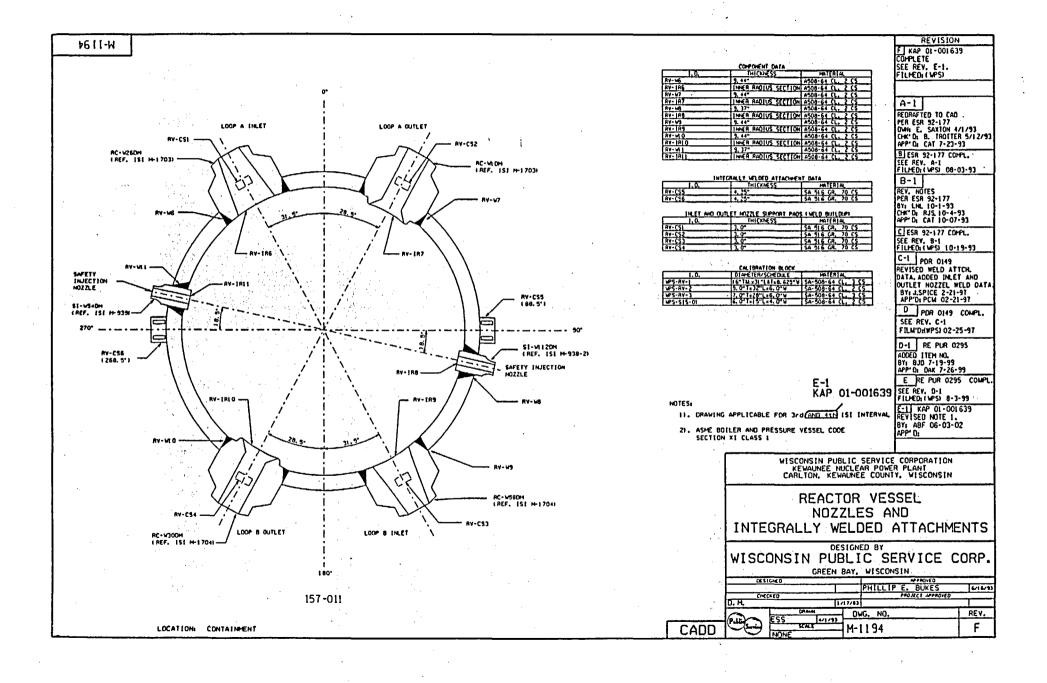
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

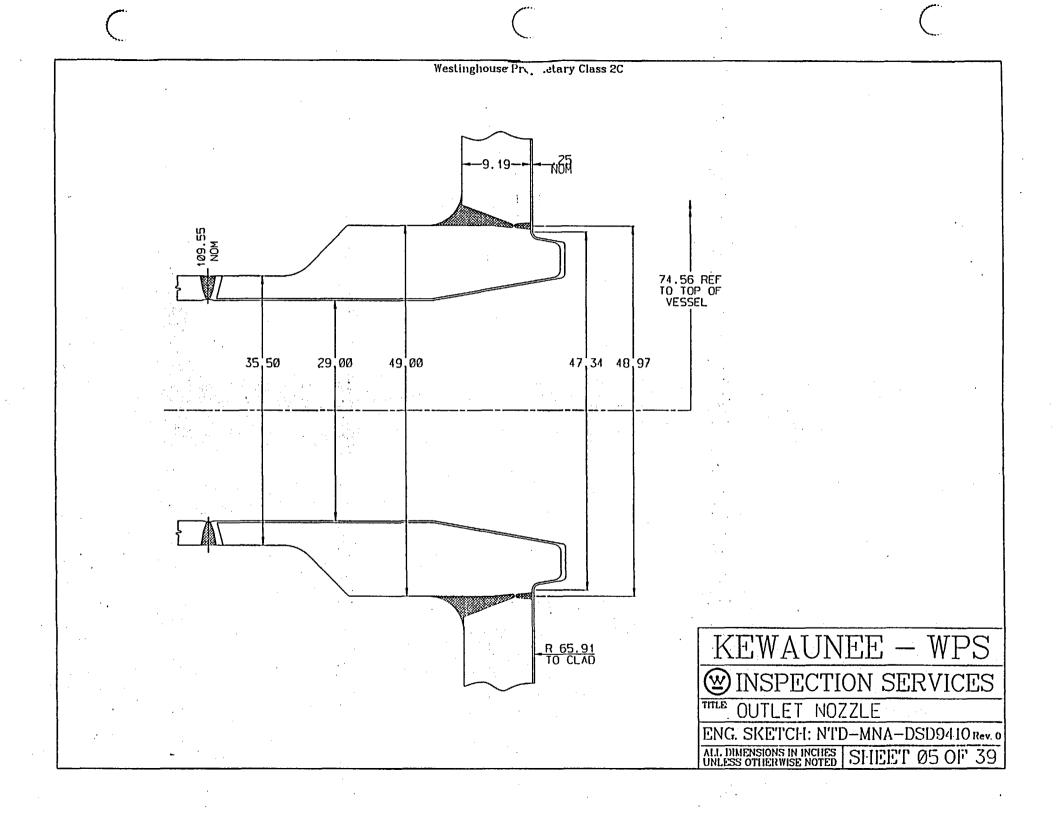
RELIEF REQUEST NO: RR-G-7-21

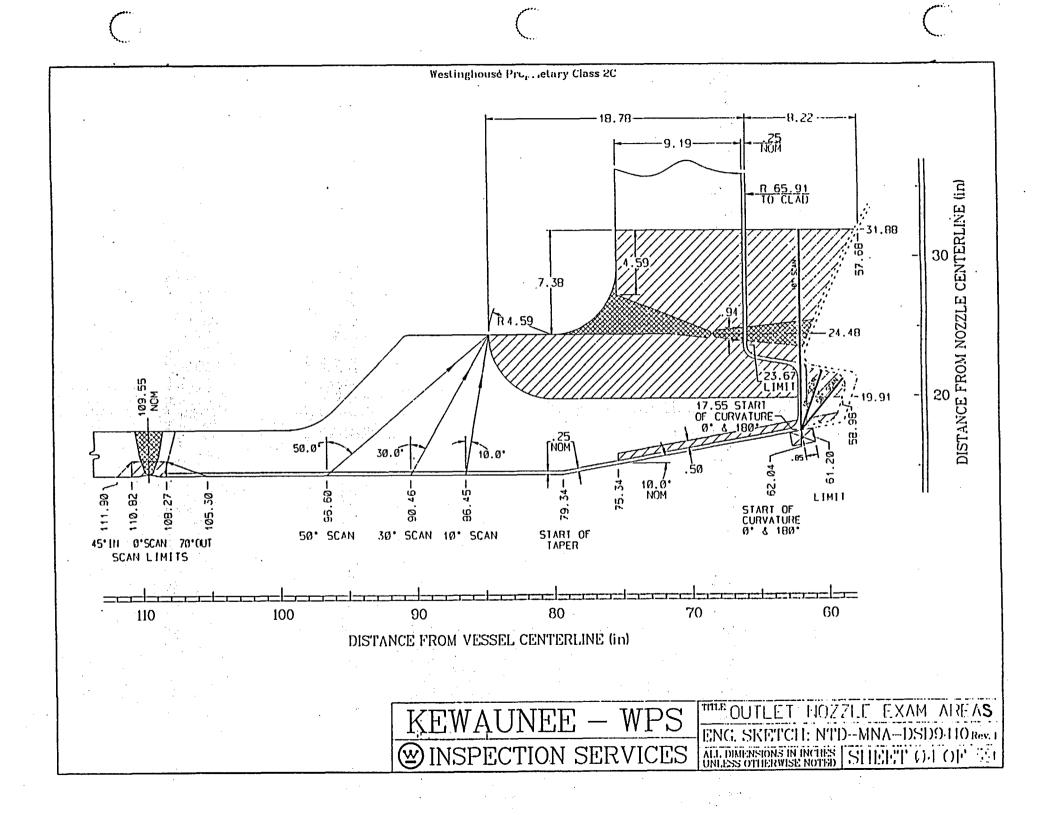
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable









Wisconsin Public Service Corporation Kewaunee Nuclear Power Plant 1st Outage; 1st Period; 3rd Interval Automated Reactor Vessel Tool Examination Coverage

Program Item	Weld Number	Description	Direction	% Coverage	Limitations
2	RV-W7	Loop A Outlet Nozzle to Vessel Weld	Perpendicular Tangential		Nozzle Boss Radius Nozzle Boss
3	RV-W10	Loop B Outlet Nozzle to Vessel Weld	Perpendicular Tangential		Nozzle Boss Radius Nozzle Boss
4	RV-IR7	Loop A Outlet Nozzle Inside Radius Section	Circumferential	100.00%	
5	RV-IR10	Loop B Outlet Nozzle Inside Radius Section	Circumferential	100.00%	
6	RC-W1DM		Parallel Perpendicular	100.00% 100.00%	
7	RC-W30DM		Parallel Perpendicular	100.00% 100.00%	

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-22

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

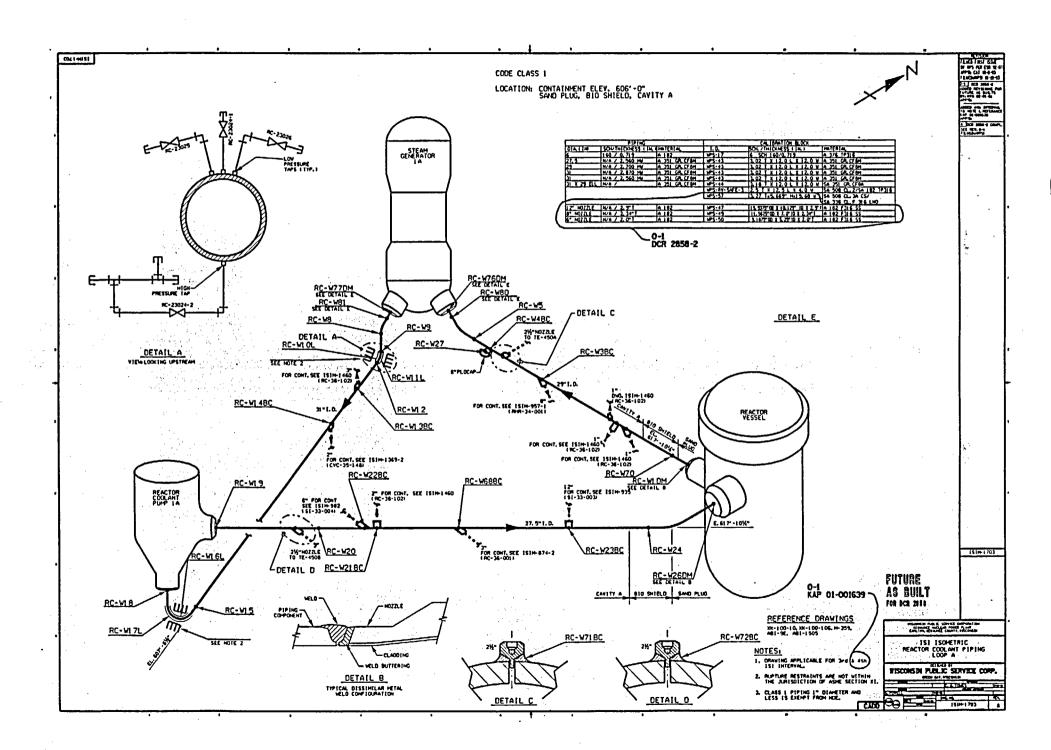
- 1. ASME Code Component Affected: Steam Generator 1A Nozzle To Safe End Butt Welds RC-W76DM and RC-77DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.70
- 4. Impracticality of Compliance: 38.1% of the Steam Generator 1A Nozzle To Safe End Butt Welds RC-W76DM and RC-W77DM were inaccessible due to the Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 38.1% of the Steam Generator 1A Nozzle To Safe End Butt Welds RC-W76DM and RC-W77DM would require modification of the design of Steam Generator 1A Nozzles and Reactor Coolant Pipe.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-22

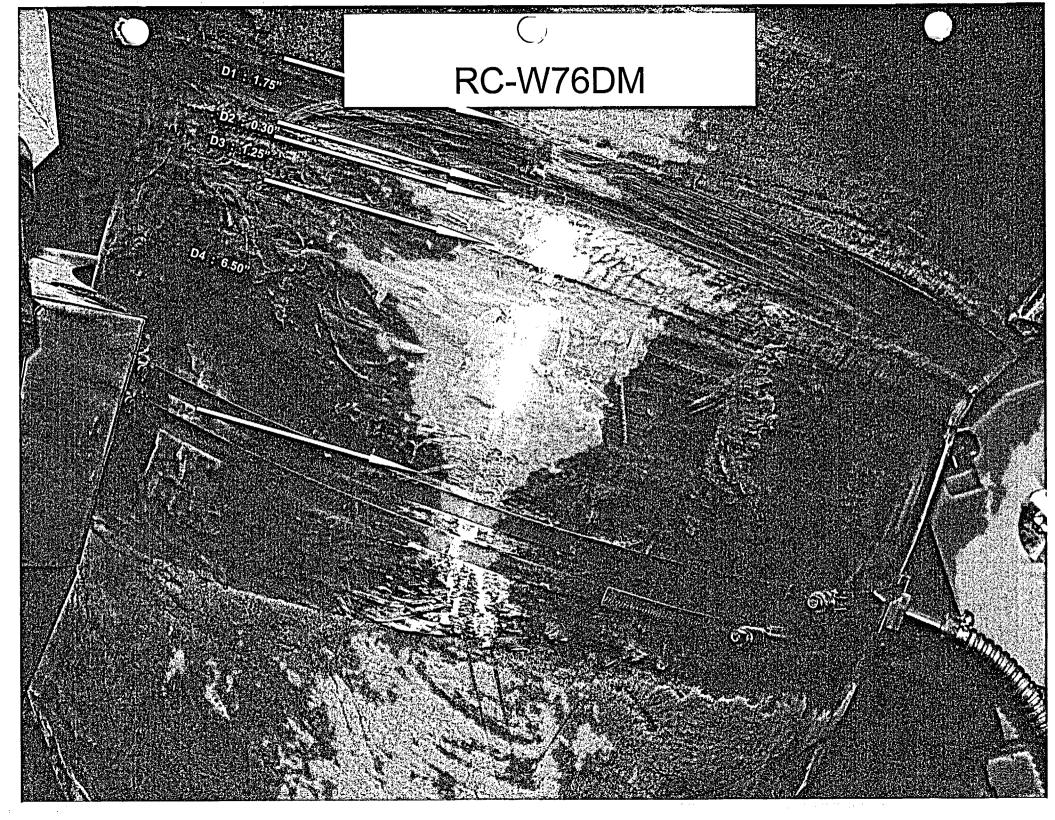
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.30 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Steam Generator 1A Nozzle to Safe End Butt Welds RC-W76DM and RC-W77DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.70. ASME Boiler and Pressure Vessel Code Section III Radiography was performed as part of Kewaunee Nuclear Power Plant Steam Generator Replacement during the 2001 Refueling Outage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REACTOR COOLART PIPING LOOP A
DRAWING NO.: ISIM-1703
COMPONENT IDENTIFICATION: 2C-W76 DM PROCEDURE: NEP-15.45 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: JUL BY TI DATE: 6-19-01 LEVEL
EXAMINER: Jelb Q III DATE: 06/19/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
Nozzle CONFIGURATION LIMITS COVERNOE WITH 0°, 45° SCAN 2.5.7, AND 8; 60° SCM 2,5.7, AND 8
SAFE END SIDE CONERAGE NOT OBTAINED: 17.4%, DUE TO MOZZLE CONFILURATION. NOZZLE SIDE COVERAGE NOT OBTAINED: 100%,
DUE TO NOZZLE CONFIDURATION.
TOTAL CONERAGE NOT OBTAINED: 38.1%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bulas DATE: June 19,200 J AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Imp. Mynin DATE: 6-20-01

SYSTEM OR COMPONENT: REACTOR COOLANT DIPING LOOP A
DRAWING NO.:ISIM -1703
COMPONENT IDENTIFICATION: RC-W770M PROCEDURE: NEP-15.45 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: J. LEVEL DATE: 6-19-01
EXAMINER: Jelb DATE: 00/19/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
NOZZLE CONFIGURATION LIMITS CONGRAGE WITH 0°.45° SCAN 2.5.7, AND 8° 60° SCAN 2,5,7, AND 8 SAFE END SIDE CONERAGE NOT DETAINED: 17.4%, DUE TO NOZZLE CONFIGURATION. NOZZLE SIDE CONERAGE NOT OBTAINED: 100%, DUE TO NOZZLE CONFIGURATION. TOTAL CONERAGE NOT OBTAINED: 38.1%
KEWAUNEE NUCLEAR PROPERTY AND
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogu Mogram DATE: 6-20-01



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-23

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

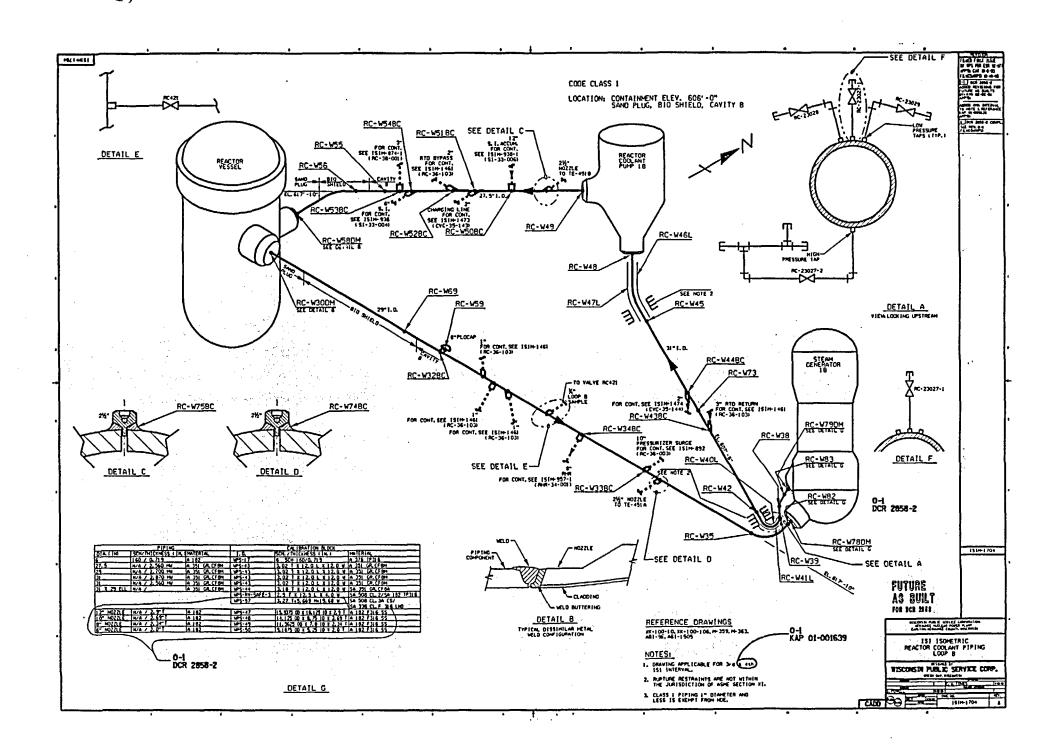
- 1. ASME Code Component Affected: Steam Generator 1B Nozzle To Safe End Butt Welds RC-W78DM and RC-79DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.70
- 4. Impracticality of Compliance: 40.25% of the Steam Generator 1B Nozzle To Safe End Butt Welds RC-W78DM and RC-W79DM were inaccessible due to the Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 40.25% of the Steam Generator 1B Nozzle To Safe End Butt Welds RC-W78DM and RC-W79DM would require modification of the design of Steam Generator 1B Nozzles and Reactor Coolant Pipe.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-23

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.30 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Steam Generator 1B Nozzle to Safe End Butt Welds RC-W78DM and RC-W79DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.70. ASME Boiler and Pressure Vessel Code Section III Radiography was performed as part of Kewaunee Nuclear Power Plant Steam Generator Replacement during the 2001 Refueling Outage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REACTOR COOLANT PIPING LOOP B
DRAWING NO.:
COMPONENT IDENTIFICATION: RC-W780M PROCEDURE: NEP-15.45 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: VISUAL:
EXAMINER: Joll P. RUT DATE: 6-19-01 LEVEL
EXAMINER: Jello DATE: 00/19/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
NOZZLE CONFIGURATION LIMITS COVECAGE WITH DO, 45° SCAN 2,5,7, AND 8; 60° SCAN 2,5,7, AND 8 SAFE END SIDE COVERAGE NOT OBTAINED: 20.3%, DUE TO NOZZLE CONFILLICATION. NOZZLE SIDE CEVERAGE NOT OBTAINED: 100%, DUE TO NOZZLE CONFIGURATION. TOTAL COVERAGE NOT OBTAINED: 40.25%
REWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: June 19, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ry DATE: 6-20-01

SYSTEM OR COMPONENT: REACTOR COOLOUT PIPING LOOP B
DRAWING NO.:
COMPONENT IDENTIFICATION: RC-W75 DM PROCEDURE: NEP-15.45 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Tell Blf TIL DATE: 6-19-01 LEVEL
EXAMINER: Jello Date: 06/19/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
Q Nozz(E
NORRE CONFIGURATION LIMITS CONFLABE WITH DO. 45° SCAN 2.5.7. AND 8; 60° SCAN 2.5.7. AND 8
SAFE END SIDE COVERAGE NOT OBTAINED: 20.3%, DUE TO NOZZIE CONFILURATION.
NOZZLE SIDE CONFIGURATION: 100%,
INE TO MOZZUE CONFIGURATION.
TOTAL COVERAGE NOT OSTAWED: 40.25%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: June 19, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roy Majuri DATE: 6-20-01

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-24

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

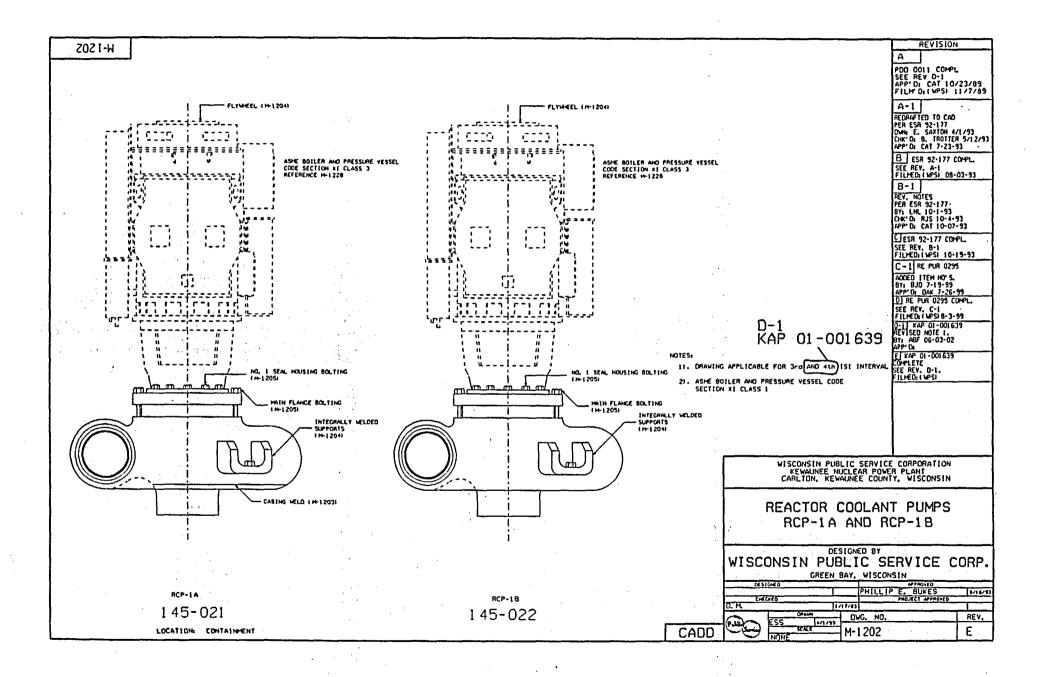
- 1. ASME Code Component Affected: Reactor Coolant Pump 1A Flywheel RCP-1A-FLY
- 2. Applicable Code Edition and Addenda: Not Applicable Nuclear Regulatory Commission Regulatory Guide 1.14 Reactor Coolant Pump Flywheel Integrity
- 3. Applicable Code Requirement: Not Applicable
- 4. Impracticality of Compliance: 9.0% of the Reactor Coolant Pump 1A Flywheel RCP-1A-FLY was inaccessible due to the 5 Anti Rotation Paws and the Flywheel In Place Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 9.0% of the Reactor Coolant Pump 1A Flywheel would require removal of the 5 Anti Rotation Paws which would affect the balance of the Reactor Coolant Pump Motor and would also require modification of the original design of Reactor Coolant Pump Flywheel Motor.

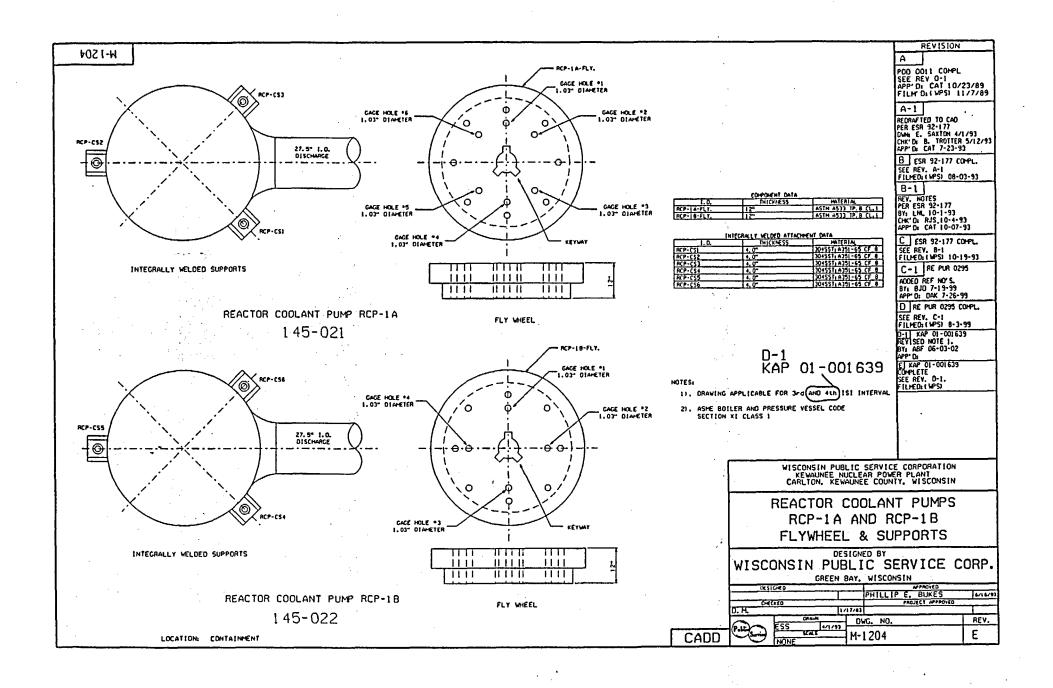
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-24

- 6. Proposed Alternative and Basis for Use: No alternative Regulatory Guide required Ultrasonic examination is available due to the limited access.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable





SYSTEM OR COMPONENT: REACTOR COOLANT PYMP 14
DRAWING NO.:
COMPONENT IDENTIFICATION: RCP- IA - FLY PROCEDURE: NEPROSECUES ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: 1015/96 LEVEL DATE: 1015/96
EXAMINER: July w John II DATE: 10/5/96 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
GAGE HOLE
ANTI-ROTATION PAWS J
FLYWHEEL IN PLACE RESTRICTS BOTTOM 1" OF LOWER PLATE OF KEYWAY AND RADIAL GAGE HOLE EYAMS. DIAMETER CHANGE FROM 1.5" TO 1.03" IN TOP 1.5" OF UPPER PLATE RESTRICTS KEYWAY AND RADIAL GAGE HOLE EXAMS. 5 - 3" X 7" ANTI - ROTATION PAWS RESTRICT PERIPHERY EXAM ON LOWER PLATE
9% 25 8
9% of Required Volume. Not Examined
POWER PLANT REVIEW: Philip C. Bukes DATE: October 5, 1996
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Avgn Thoughing DATE: 10/5/96

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-25

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

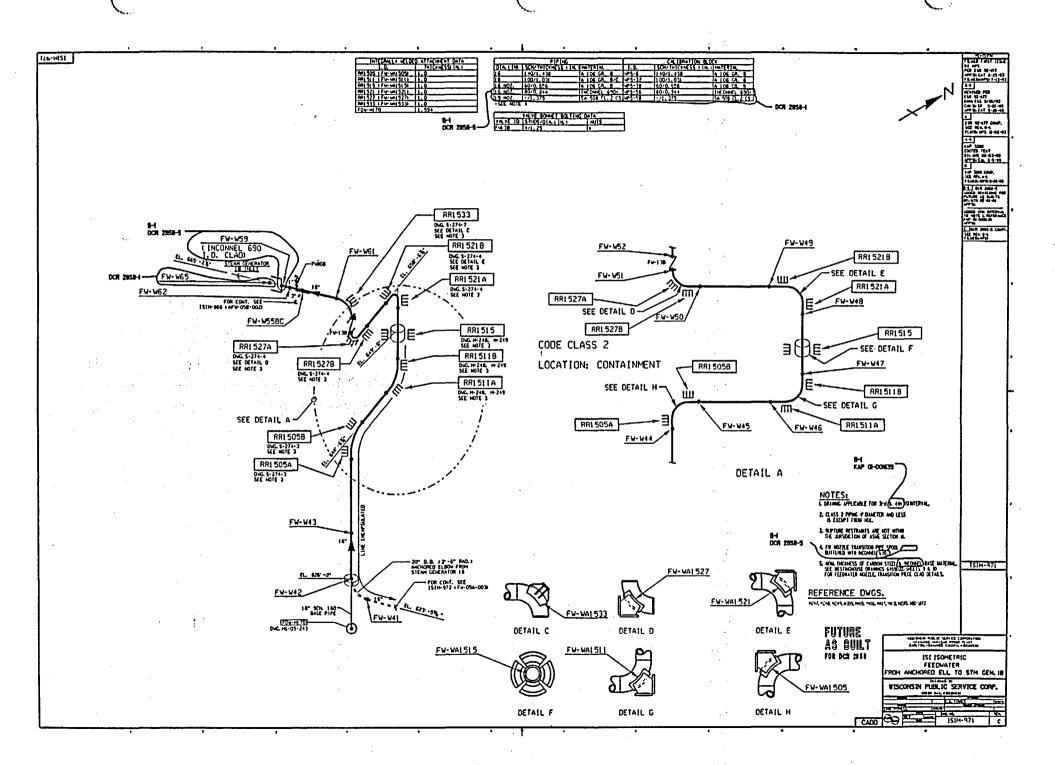
- 1. ASME Code Component Affected: 16" Feedwater Integrally Welded
 Attachment FDW-H170
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-C; Item No. C3.20 and Table IWF-2500-1 Examination Category F-A; Item No. F1.20B
- 4. Impracticality of Compliance: 100% of the Integrally Welded Attachment Saddle Weld Portion of 16" Feedwater Support FDW-H170 is inaccessible for Surface Examination as required by IWC-2500-1, Category C-C, Item No. 3.20 and Visual examination as required by IWF-2500-1 Examination Category F-A, Item No. F1.20B due to Encapsulation surrounding the 16" Feedwater Piping. Visual Examination of the remaining portion of FDW-H170 support is available for examination.
- 5. Burden Caused by Compliance: To provide for access to perform Surface Examination and Visual examination on the Integrally Welded Attachment Saddle Weld for FDW-H170 would require removal of the 16" Feedwater Piping Encapsulation.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

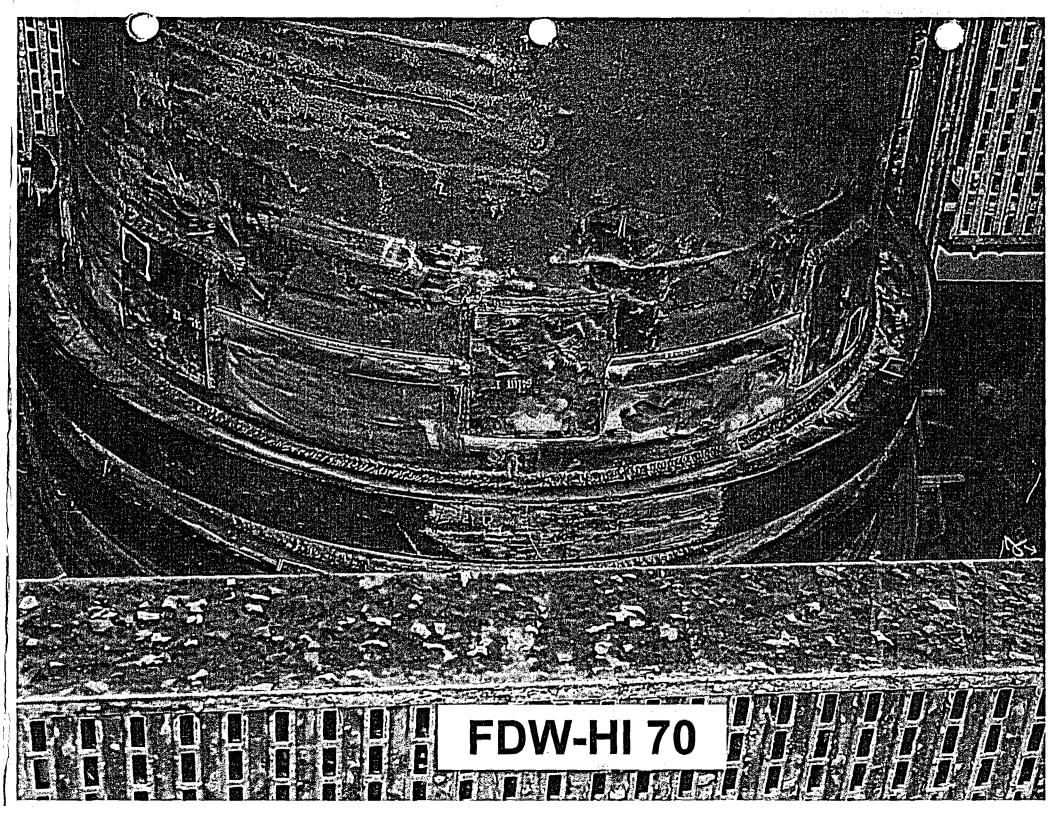
RELIEF REQUEST NO: RR-G-7-25

- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface examination is available of the FDW-H170 Saddle Weld due to the restricted access caused by the encapsulation. VT-3 Examination of the accessible portion of the Support was performed during the 3rd Interval.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: FEEDWATER FROM ANCHORED ELL TO STEAM GEN. 18				
DRAWING NO.: ISTM-971				
COMPONENT IDENTIFICATION: FDW-H170 PROCEDURE: NEPNO.15.5 REVISION: ORIG				
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL: _X				
EXAMINER:				
EXAMINER:				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
VT-3 Exam Limitation Due To EN CAPSULATION PIPE AROUND THE FEED WATER LINE. THE SADDLE WELD CANNOT BE SEEN				
REWAUNEE NUCLEAR POWER PLANT REVIEW: October 18, 2001 Phillip C. Buken DATE: October 18, 2001				
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:				



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-26

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

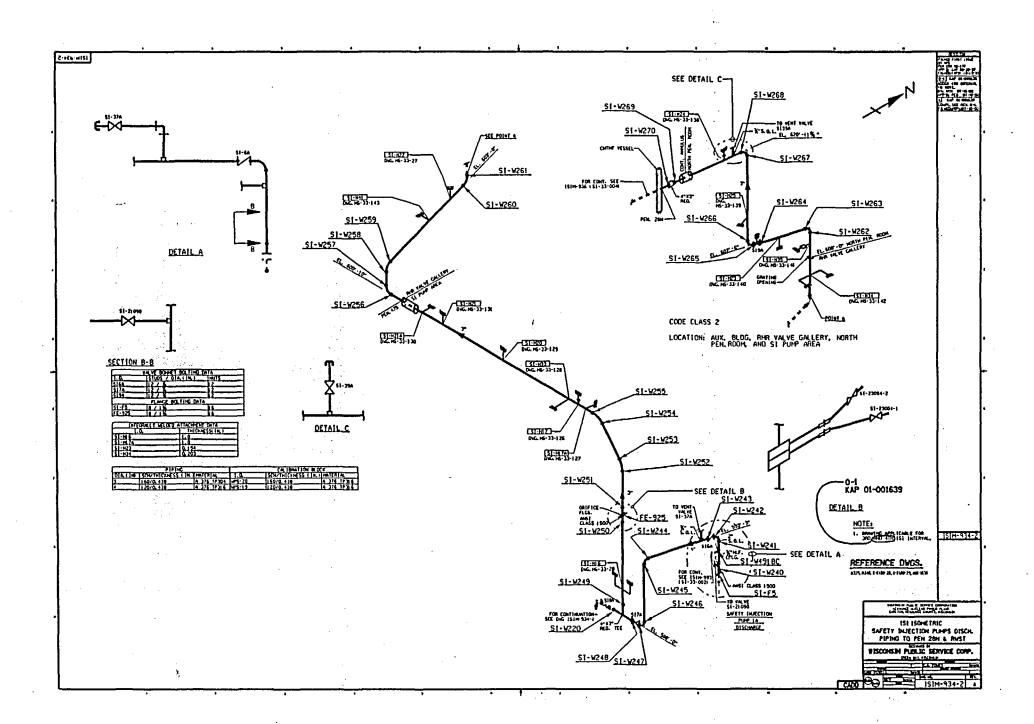
- 1. ASME Code Component Affected: 3" Safety Injection Integrally Welded
 Attachment SI-H17A
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-C; Item No. C3.20
- 4. Impracticality of Compliance: 1% of Integrally Welded Attachment SI-H17A was inaccessible for Surface Liquid Penetrant Examination due to Welded Name Plate located next to 2 of the 4 Welded Lugs thus restricting Surface Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Surface Examination for SI-H17A would require removal and then rewelding of the Name Plate located on the 3" Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-26

- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface examination is available for the SI-H17A Welded Lug Area due to restricted access caused by the Welded Name Plate. VT-3 Examination of the accessible portion of SI-H17A Support and Welded Lugs were performed.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SAFETY INJECTION PUMPS DISCH. SYSTEM OR COMPONENT: PIPING TO PEN 28 N & RWST
DRAWING NO.: ISIM-934-2
COMPONENT IDENTIFICATION: ST-HITA PROCEDURE: NEPND. 15.6 REVISION: DRIG.
ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jrain Zhoman III DATE: 10-26=98 LEVEL
EXAMINER: N A DATE: N A
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. SI-HIT
SI-HITA WELDED NAME PLATE LUGS O
NAME PLATE .20" FRDM TOE DF WELD DN 2 LUGS
PERCENTAGE OF REDUCED EXAMINATION COVERAGE = 1 %
REWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rom Motion DATE: 10-28-98
INSERVICE INSPECTOR REVIEW: Roan Mothum DATE: 10-28-98

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-27

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

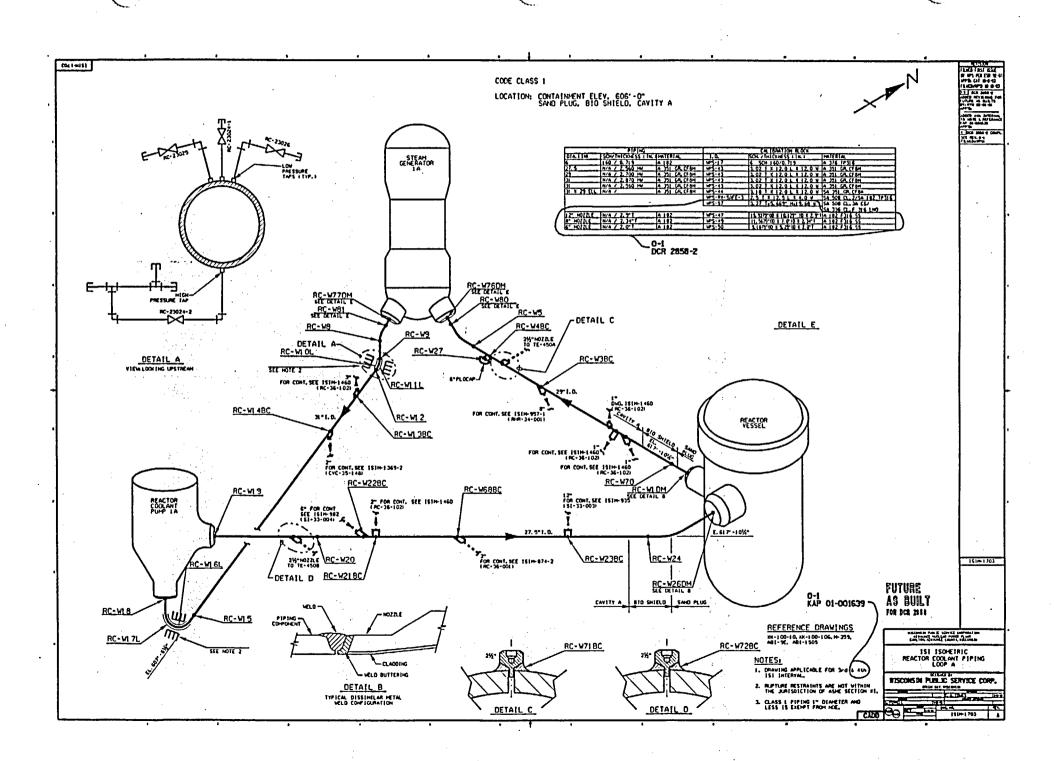
- 1. ASME Code Component Affected: 8" Reactor Coolant Pipe Branch
 Connection Weld RC-W3BC
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.31
- 4. Impracticality of Compliance: 42.4% of Reactor Coolant Pipe 8" Branch Connection Weld RC-W3BC was inaccessible for Ultrasonic Examination due to the Branch Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Ultrasonic Examination for Reactor Coolant Pipe 8" Branch Connection Weld RC-W3BC would require modification of the original design of the Reactor Coolant Pipe.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

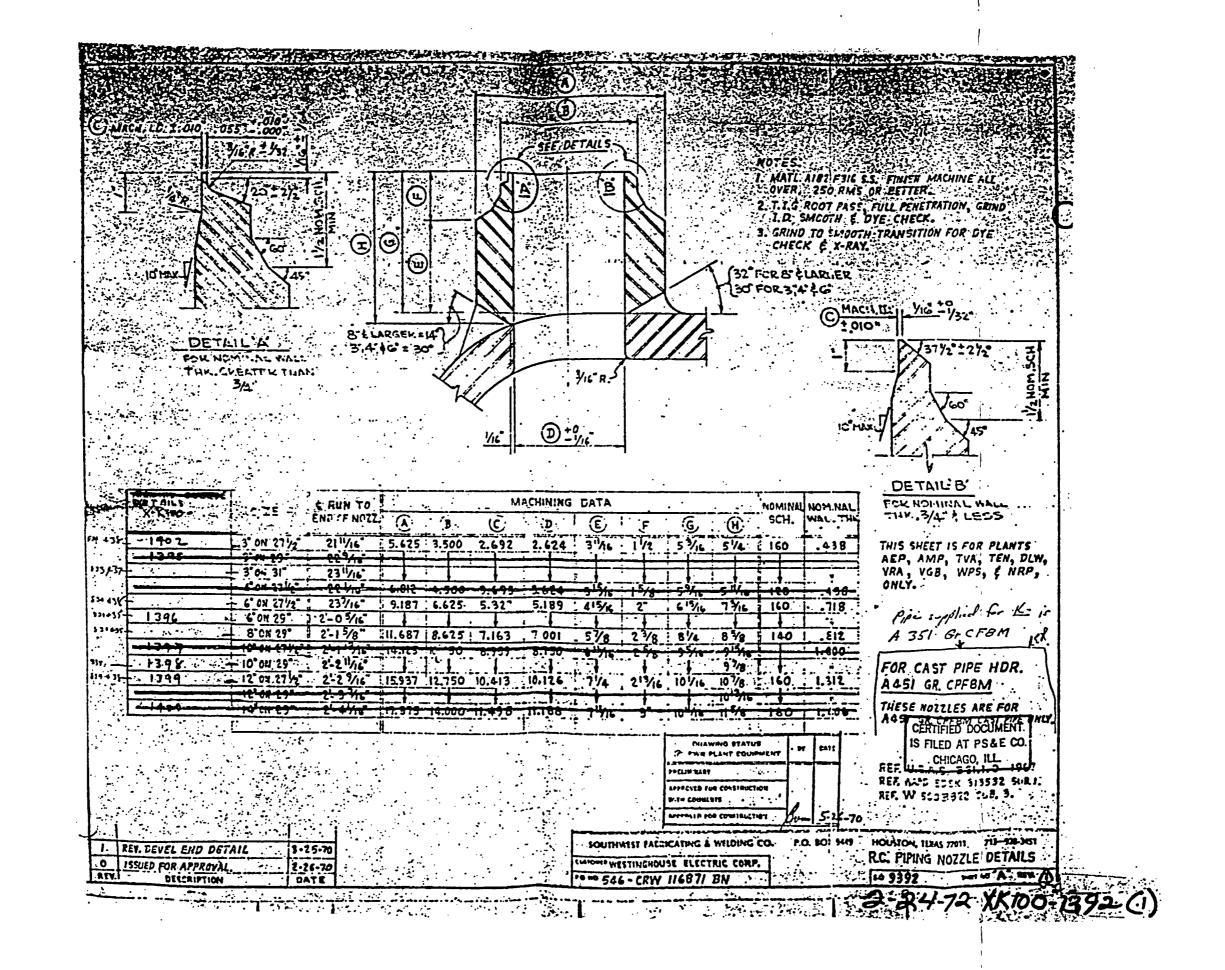
RELIEF REQUEST NO: RR-G-7-27

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic Examination is available for Reactor Coolant Pipe Branch Connection RC-W3BC due to the Nozzle Configuration. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item No. B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Reactor Coolant Pipe 8" Branch Connection Weld RC-W3BC during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.31.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: PEACTOR COOLAUT PIPIUG LOOP A				
DRAWING NO.: 151M-1703				
COMPONENT IDENTIFICATION: RC-W3BC PROCEDURE: NEP No 15.39 REVISION: OF 15				
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:				
EXAMINER: Jell Ones II DATE: 11/03/98 LEVEL				
EXAMINER: DATE: 11-03-98 LEVEL				
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.				
8 REACTOR WOLANT PIPE RC-W3BL				
NOTE: NO SCAN 5, SCAN 7 & 8 LIMITED TO WELD & DOWNSTREAM BASE METAL ONLY DUE TO CONFIGURATION OF BLANCH CONNECTION.				
PROCEDURE LIMITATION: 42.47				
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Buked AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roger Material DATE: 11-6-98				



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-28

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Reactor Coolant Pipe Branch Connection Weld RC-W4BC
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.31
- 4. Impracticality of Compliance: 65.0% of Reactor Coolant Pipe 6" Branch Connection Weld RC-W4BC was inaccessible for Ultrasonic Examination due to the Branch Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Ultrasonic Examination for Reactor Coolant Pipe 6"Branch Connection Weld RC-W4BC would require modification of the original design of the Reactor Coolant Pipe.

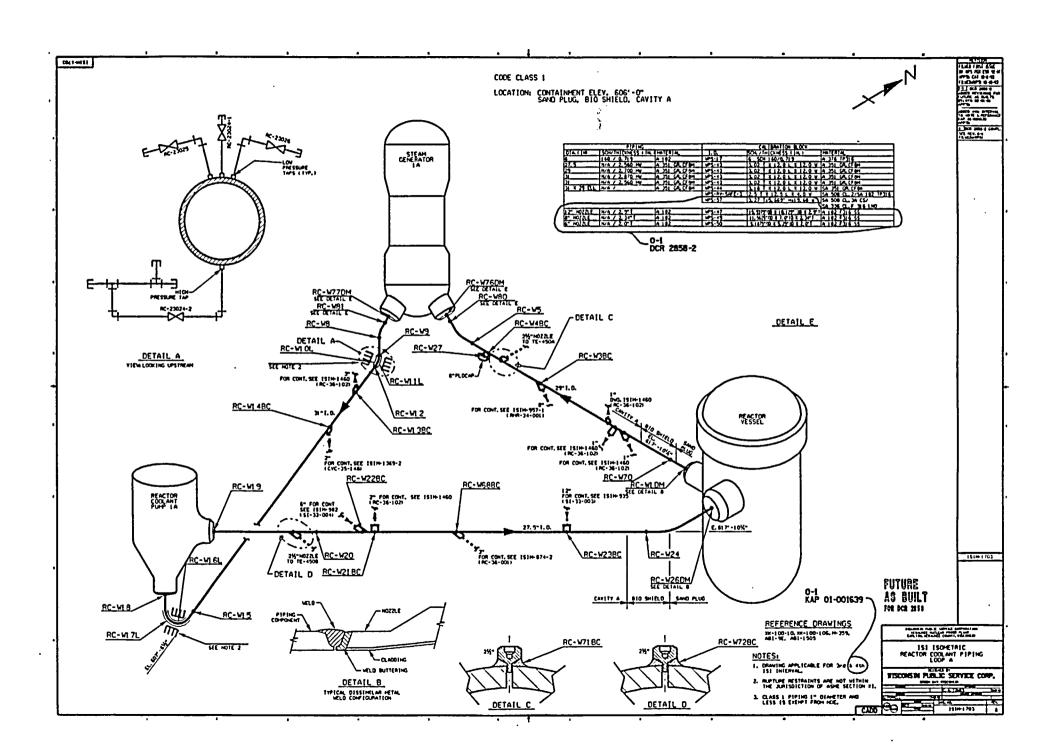
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-28

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic Examination is available for Reactor Coolant Pipe 6" Branch Connection RC-W4BC due to the Nozzle Configuration. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item No. B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Reactor Coolant Pipe 6" Branch Connection Weld RC-W4BC during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.31.

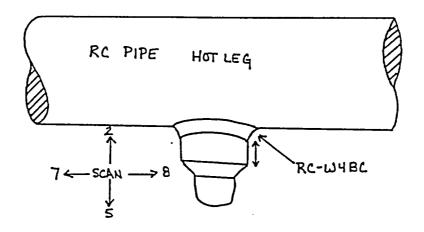
7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REACTOR COOLANT	- PIPING LOO	РА	
DRAWING NO.: ISTM-1703			
COMPONENT IDENTIFICATION: RC-W4BC	_PROCEDURE:	NEP NO.15.3	REVISION: DRIG.
ULTRASONIC: X LIQUID PENETRANT:	_ MAGNETIC PA	RTICLE:	VISUAL:
EXAMINER: Al Calm	LEVEL		11-7-98
EXAMINER: July	I LEVEL	_ DATE: _	11-7-98

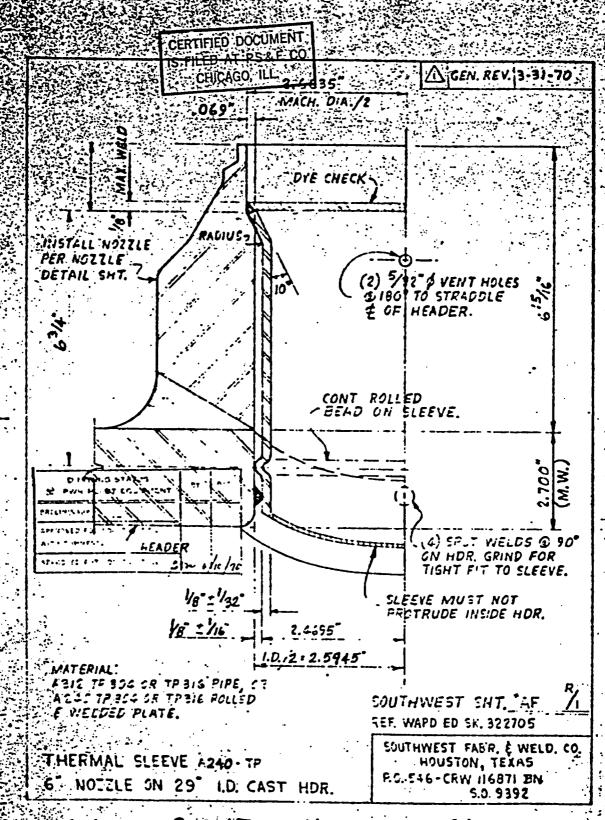
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.



X NO SCAN 5 DUE TO BRANCH CONFIGURATION. X SCANS 7: 8 LIMITED DUE TO BRANCH CONFIGURATION.

PROCEDURE COVERAGE REDUCED BY 65%.

POWER PLANT REVIEW: Philips C. Bukes	DATE: November 9,1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roga Majuri	DATE: 11-9-98



2-24-72 XK100-1396 (1)

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-29

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

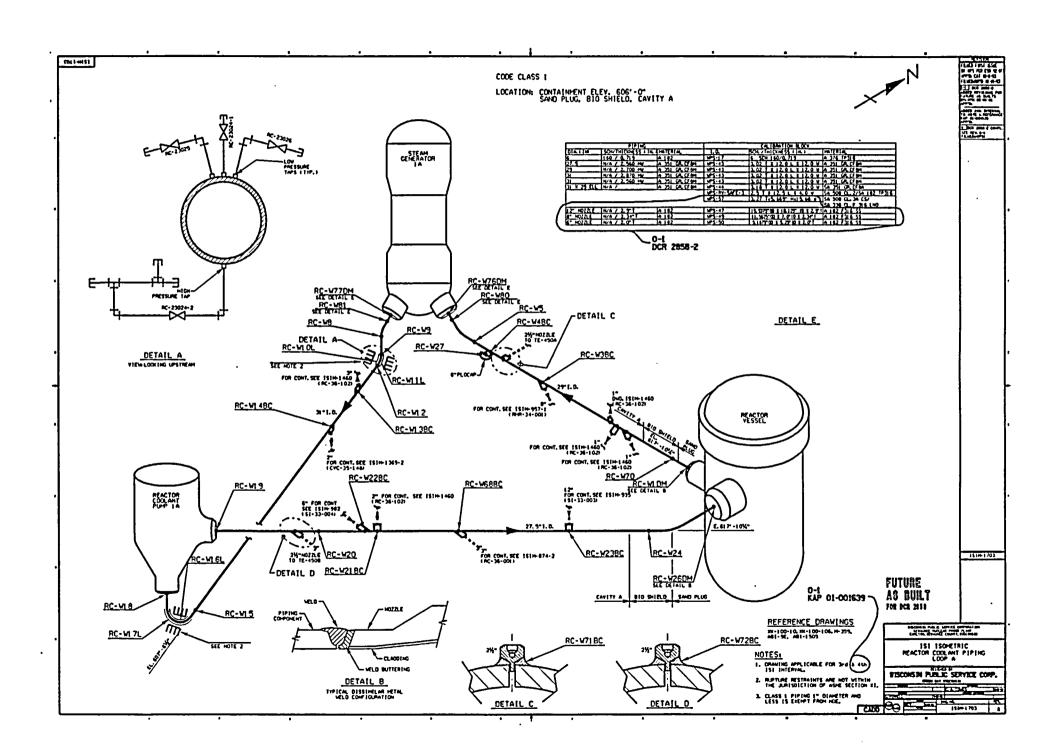
- 1. ASME Code Component Affected: 29" I.D. Reactor Coolant Pipe Circumferential Weld RC-W5
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 4.6% of Reactor Coolant Pipe 29" I.D. Circumferential Weld RC-W5 was inaccessible for Ultrasonic Examination due to the O.D. Taper on the Elbow thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Ultrasonic Examination for Reactor Coolant Pipe 29" I.D. Circumferential Weld RC-W5 would require modification of the original design of the Reactor Coolant Pipe.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-29

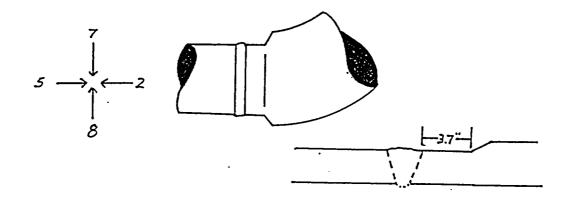
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic Examination is available for Reactor Coolant Pipe 29" I.D. Circumferential Weld RC-W5 due to the O.D. Taper on Elbow. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item No. B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Reactor Coolant Pipe 29" I.D. Circumferential Weld RC-W5 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REACTOR COOL	ANT PIPING	LOOP A
DRAWING NO.:	•	. •
COMPONENT IDENTIFICATION:	_PROCEDURE:	NEPNo. 15.13 REVISION: A
ULTRASONIC: X LIQUID PENETRANT:	_ MAGNETIC PA	RTICLE: VISUAL:
EXAMINER: Jeff Oeves	エ LEVEL	DATE:
EXAMINER: Luft Caul	LEVEL	DATE: <u>10-30 - 98</u>

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.



NOTE: SCAN 2 LIMITED ON UPSTREAM SIDE OF WELD DUE TO O.D. TAPER ON ELBOW - SEE DIMENSIONS ABOVE.

PERCENTAGE OF PROCEDURAL LIMITATION 4.6%

KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phil	lep C. Bukes	DATE: November 7, 1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	Roga Motenia	DATE:

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-30

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

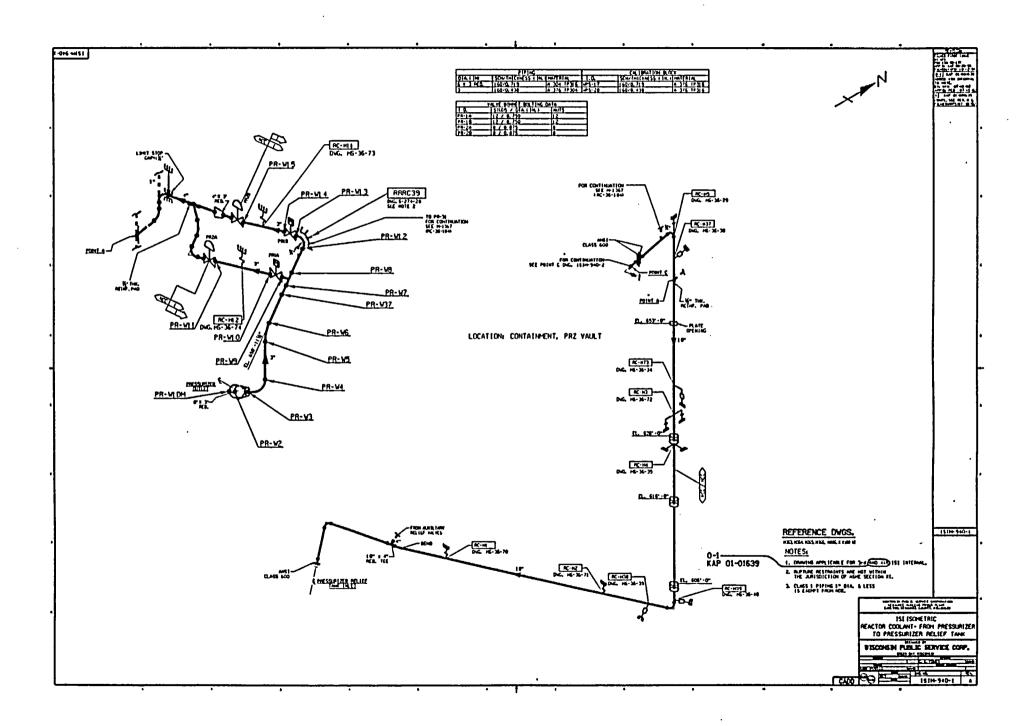
- 1. ASME Code Component Affected: 3" Pressurizer Relief Circumferential Weld PR-W12
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.21
- 4. Impracticality of Compliance: 74.0% of the 3" Pressurizer Relief Circumferential Weld PR-W12 was inaccessible due to a Whip Restraint RRRC39 thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 74.0% of the 3" Pressurizer Relief Circumferential Weld PR-W12 would require removal of the Whip Restraint RRRC39.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

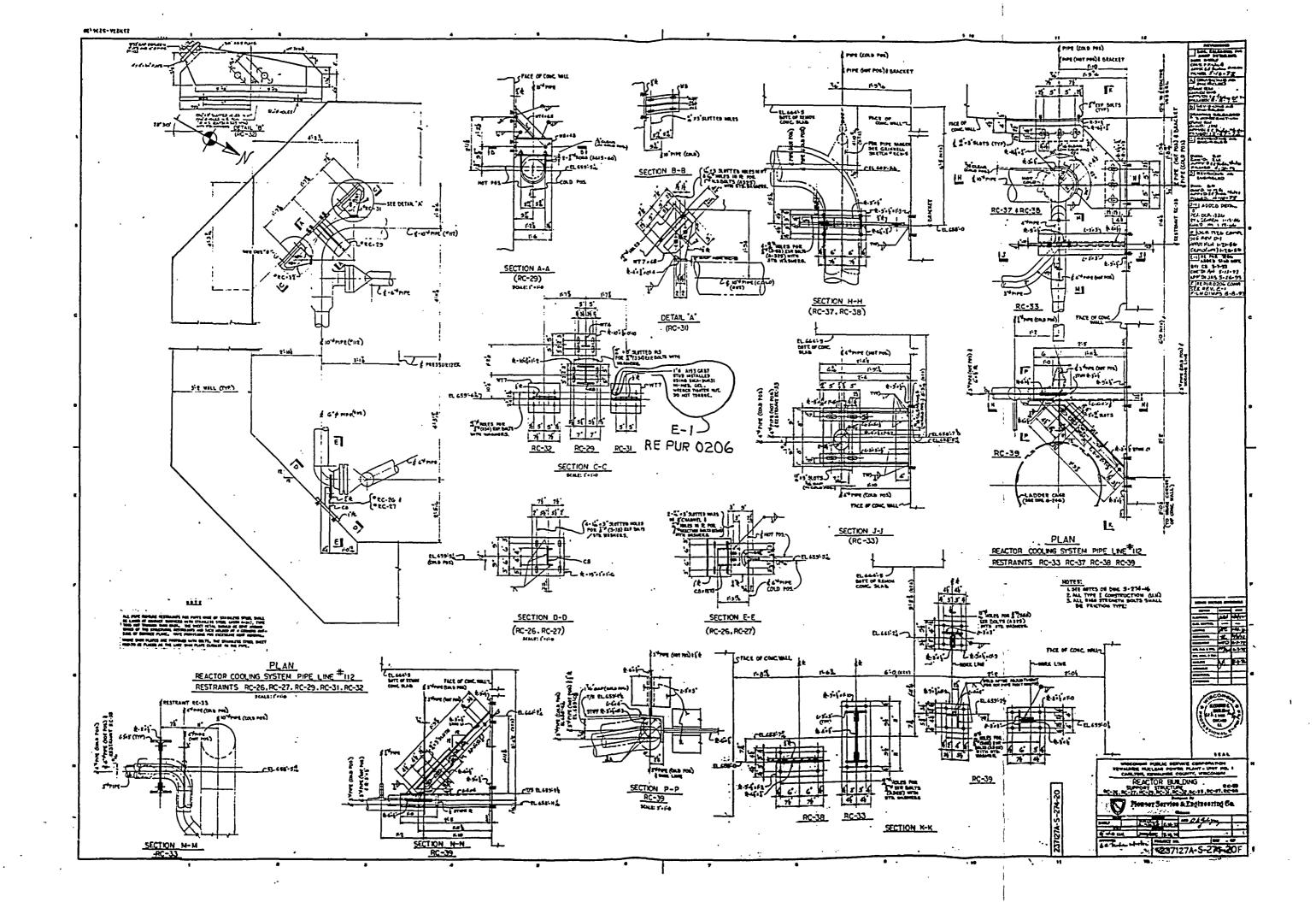
RELIEF REQUEST NO: RR-G-7-30

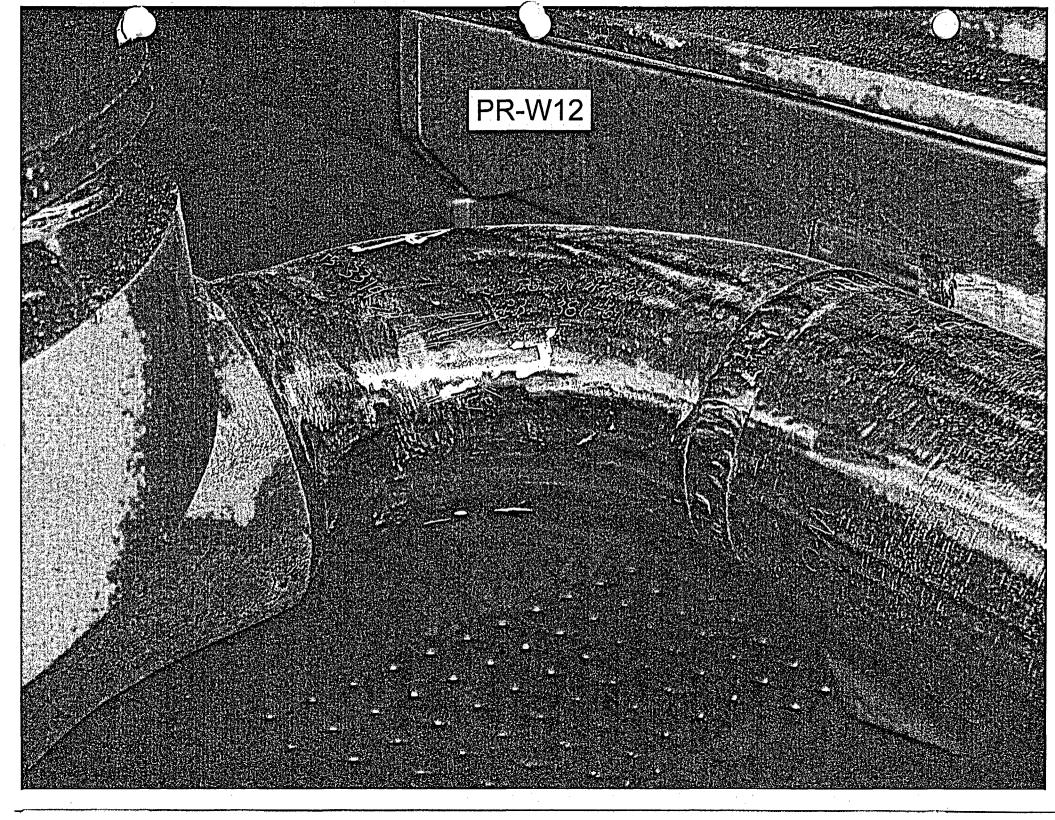
- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



	·	
Renoted Coolant-From F SYSTEM OR COMPONENT: To Press unizer Relie		RAWING NO.: 1S1 M - 940-1
COMPONENT IDENTIFICATION: PR-W 12	PROCEDU	RE:QCP-961 REVISION: ORIG.
ULTRASONIC: LIQUID PENETRANT: X	_ MAGNETIC PA	RTICLE: VISUAL:
EXAMINER: N.L.A. Bej.	LEVEL	_ DATE: 4-15-95
EXAMINER: NA	N A LEVEL	DATE: NA
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCA PERCENTAGE OF REDUC		
PR-WIN		whip Restraint Limited P. T. Exam. of weld PR- WIZ Percentage of Reduced Examination Coverage = 74%
KEWAUNEE NUCLEAR POWER PLANT REVIEW:	Balstad	DATE: 4/17/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	sta	DATE: <u> </u>





KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-31

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Pressurizer 6" Nozzle To Safe End Butt Weld PR-W1DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.40
- 4. Impracticality of Compliance: 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W1DM was inaccessible due to the Carbon Steel Nozzle Configuration and Nozzle O.D. Taper Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W1DM would require modification of the original design of Pressurizer Nozzle to Safe Ends.

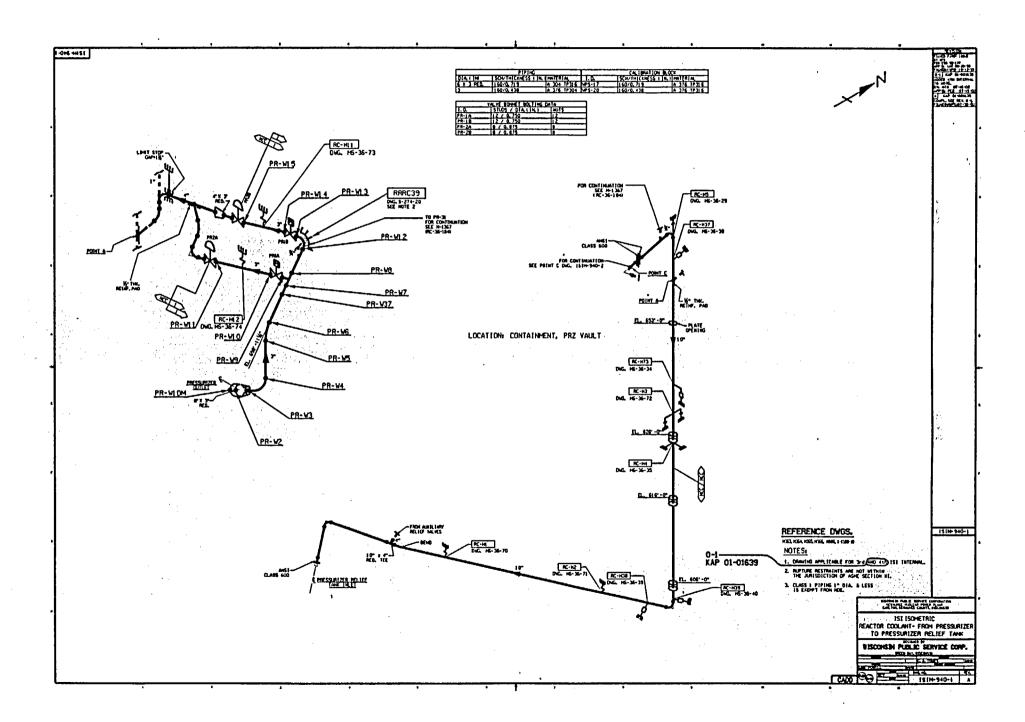
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-31

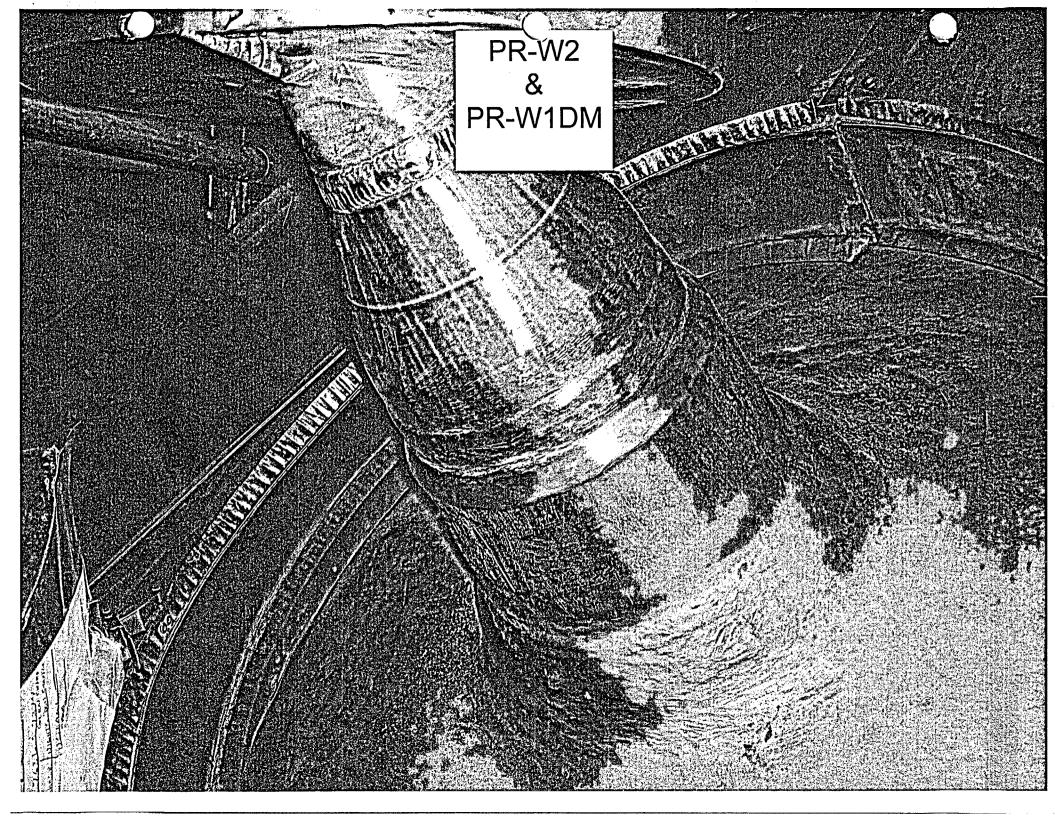
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Pressurizer 6" Nozzle To Safe End Butt Weld PR-W1DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.40.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Bogan Motymen	DATE://-6-98
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Bogan Motion	DATE: November 5,1998
60° 1.D. ROLL AT 8.1 DIVISIONS.	·
45° I.D. ROLL AT 6.2 DIVISIONS.	
ADELDUATE SCREEN RANGE ACHIEVED UTILIZING WPS-17	•
ACTUAL PART THICKNESS: 1.2" CAL. BLOCK (WPS 17) THICKNESS: 0.719"	
PERCENTAGE OF COOE/PROCEDURE LIMITATION: 50%	
SCAN 2 LIMITED DUE TO O.O. TAPER	
AND UIS BASE METAL OULY DUE TO C/S OF ONE !	† PR.WZLOM ARE TYDICAL WOTHER.
NOTE: NO SLAN 5. SLAN 7 ! 8 LIMITED TO WELD (1) PR. WIDM	
cls	
\$\\\ \\$\\\\	
7 5	
2	
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, PERCENTAGE OF REDUCED EXAMINATION COV	
EXAMINER: Juy Colom II DA'	1E: // 03-70
EXAMINER: Jell Juses I LEVEL DA	TE: 11/03/98
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICI	LE: VISUAL:
COMPONENT IDENTIFICATION: PROCEDURE: NEP N	L IE IA DEVISIONI ACIA
DRAWING NO.: _ 151M-940 - 1	
SYSTEM OR COMPONENT: REACTOR COOLANT FROM PRESSURIZER TO PE	lessurizer Relief Tank



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-32

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

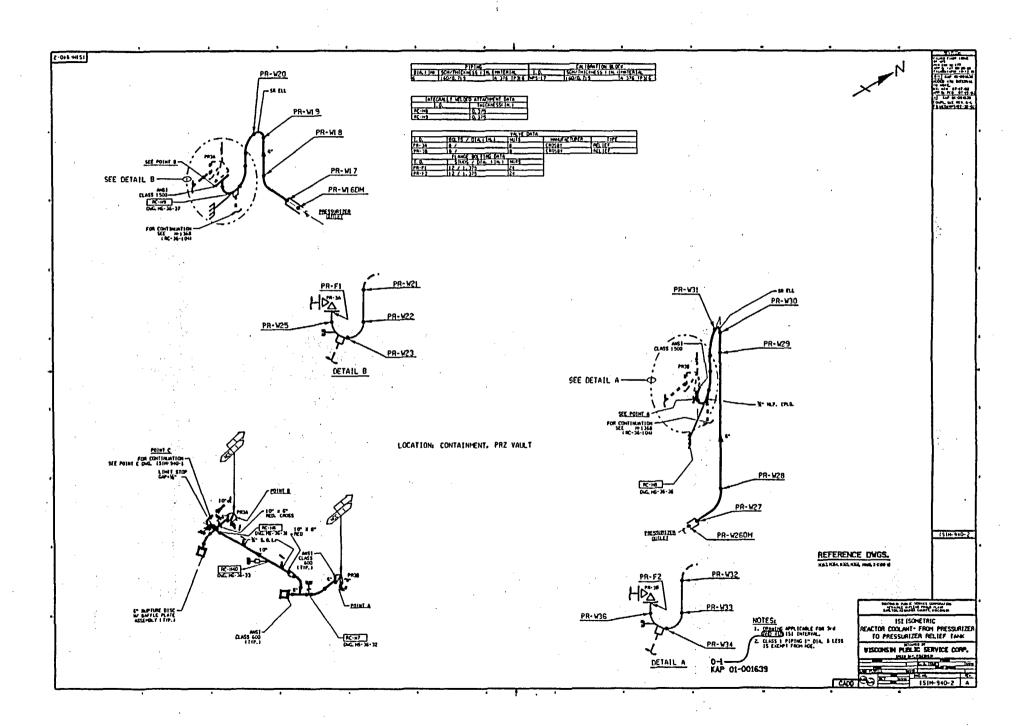
- 1. ASME Code Component Affected: Pressurizer 6" Nozzle To Safe End Butt Weld PR-W16DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.40
- 4. Impracticality of Compliance: 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W16DM was inaccessible due to the Carbon Steel Nozzle Configuration and Nozzle O.D. Taper Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W16DM would require modification of the original design of Pressurizer Nozzle to Safe Ends.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

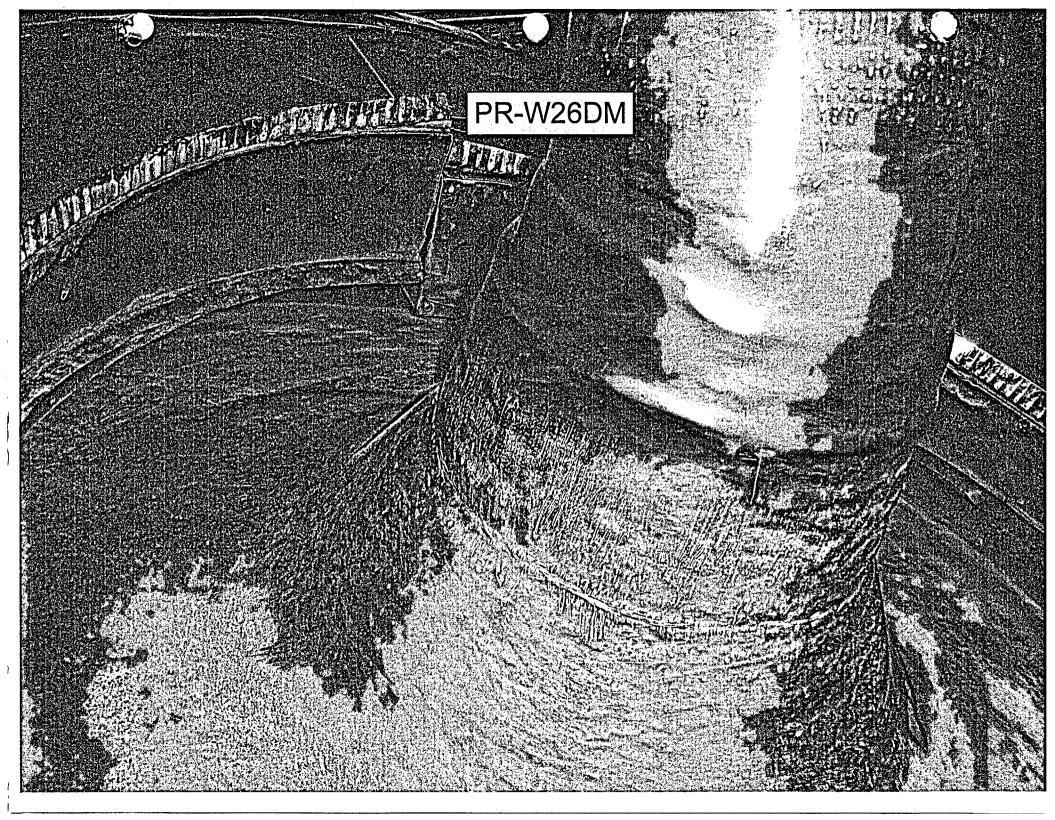
RELIEF REQUEST NO: RR-G-7-32

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Pressurizer 6" Nozzle To Safe End Butt Weld PR-W16DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.40.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



_	
	SYSTEM OR COMPONENT: REACTOR COOLANT - FLOW PRESSURIZER TO PRESSURIZER RELIEF TANK
	DRAWING NO.:
	COMPONENT IDENTIFICATION: PR - WILD DM PROCEDURE: NEP-15.14 REVISION: ORIG.
	ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
	EXAMINER: TIM COBURN WIM I DATE: 10/25/01 LEVEL
	EXAMINER: Milable DATE: 10-25-01 LEVEL
	SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
	sts
-	cls < Flow
	PERCENTAGE OF COSE/PROCEDURE LIMITATION: 50%
	ALTUAL PART THICKNESS: 1.2" CAL. BLOCK (WPS . 17) THICKNESS: 0.719"
	ADEQUATE SCREEN RANGE ACHIEVED UTILIZING WPS-17 45° 1.D. ROLL AT 6.2 DIVISIONS.
	60° 1.D. ROLL AT B.1 DIVISIONS.
	0°,45°,60° - NO SCAN 5 DUE TO MATERIAL CONFIGURATION. SCAN 2 LIMITED DUE TO DD TAPER. SCAN 7,8 LIMITED ON S SIDE DUE TO MATERIAL
L	CONFIGURATION.
	REWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: October 29, 2001
	AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rom Marin DATE: 10/29/01



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-33

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Pressurizer 6" Nozzle To Safe End Butt Weld PR-W26DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.40
- 4. Impracticality of Compliance: 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W26DM was inaccessible due to the Carbon Steel Nozzle Configuration and Nozzle O.D. Taper Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the Pressurizer 6" Nozzle To Safe End Butt Weld PR-W26DM would require modification of the original design of Pressurizer Nozzle to Safe Ends.

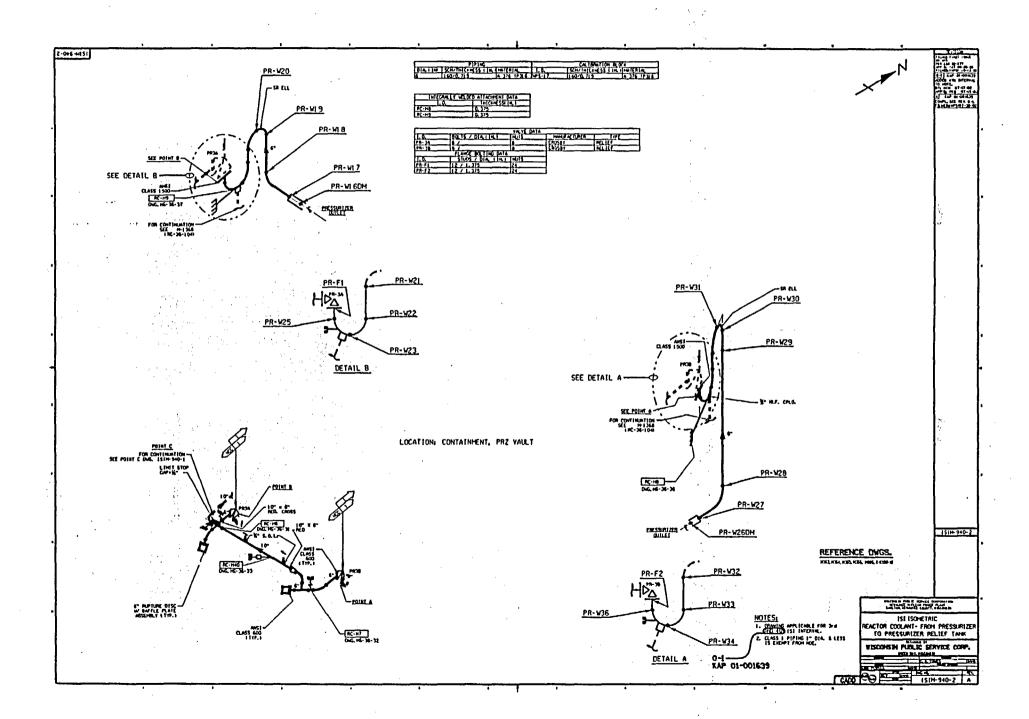
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-33

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Pressurizer 6" Nozzle To Safe End Butt Weld PR-W26DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.40.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REACTOR CONLAWT FROM	M DEESSULIZER TO PRESSULIZER RELIEF TANK
DRAWING NO .: 151M-940-2	
COMPONENT IDENTIFICATION: 22-W26DM	PROCEDURE: NEP No. 15.14 REVISION: On 5
ULTRASONIC: X LIQUID PENETRANT:	MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jell Dues	DATE: 11 03 98
EXAMINER: Jug Wilson	<u>I</u> DATE: <u>11-03-98</u> LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCAT PERCENTAGE OF REDUCE	TION, ORIENTATION, TYPE OF LIMITATION AND DEXAMINATION COVERAGE.
NOTE: NO SCAN 5, SCAN 7 ! 8 LIMITED TO WELD AND US BASE METAL OULY DUE TO 9/5 MATERIAL. SCAN 2 LIMITED DUE TO 0.0. TARER US: SUPSTREAM PERCENTAGE OF CODE/PROCEDURE LIMITATION: 50	C/s (1) PR.WIDM & PR.WZLOM ARE TYPICAL OF ONE ANOTHER.
ACTUAL PART THICKNESS: 1.2" CAL. BLOCK (WPS . 17) THICKNESS: 0.719° ADEDUATE SCREEN RANGE ACHIEVED UTILIZION 45° I.D. ROLL AT 6.2 DIVISIONS.	د سPS-17
60° 1.D. ROLL AT 8.1 DIVISIONS.	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bu	Res DATE: November 5, 1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogn	125mm DATE: 11-6-98

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-34

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Pressurizer 10" Nozzle To Safe End Butt Weld PR-W67DM
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-F; Item No. B5.40
- 4. Impracticality of Compliance: 30.0% of the Pressurizer 10" Nozzle To Safe End Butt Weld PR-W67DM was inaccessible due to the Nozzle Configuration and the Weld Crown Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 30.0% of the Pressurizer 10" Nozzle To Safe End Butt Weld PR-W67DM would require modification of the original design of Pressurizer Nozzle to Safe Ends.

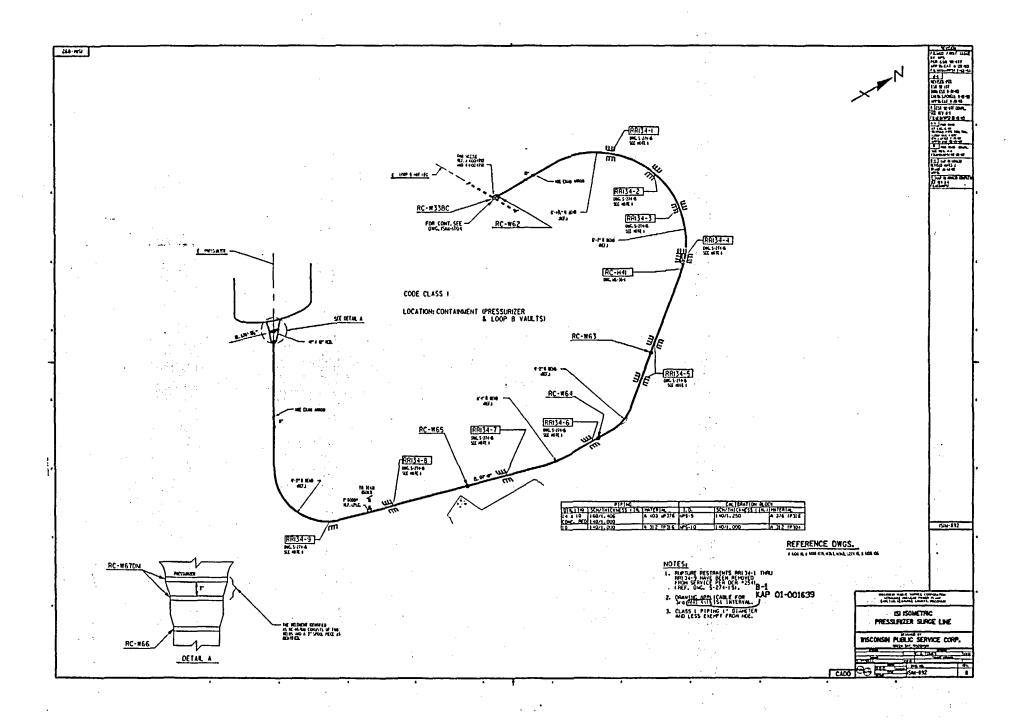
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-34

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.20 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Pressurizer 10" Nozzle To Safe End Butt Weld PR-W67DM during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-F and Item No. B5.40.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



VIGOAL EXAMINATION EMINIATION TO EXAMINATION TIEGOTE
SYSTEM OR COMPONENT: PRESSURIZER SURGE LINE
DRAWING NO.:
COMPONENT IDENTIFICATION: RC-WG7 DM PROCEDURE: NER NO. 15 14 REVISION: OR 16.
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jun Homes II DATE: 11-17-98 LEVEL
EXAMINER: 4 Confer TIT DATE: 11-17-98 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
PRESSURIZER ;
NOTZLE CONFIGURATION LIMITS WELD RC WG7 DM 45° L
RC-WG7DM 3" A SCAN AND CO°L A SCAN
KC-WG I LIM
WELD CROWN CONFIGURATION
PROCEDULAL REDUCED COUERAGE = 31 %
CODE REDUCED COVERAGE = 30%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: November 18,1998
AUTHORIZED NUCLEAR

INSERVICE INSPECTOR REVIEW:

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-35

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 12" Safety Injection Circumferential Weld SI-W120
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 12" Safety Injection Circumferential Weld SI-W120 was inaccessible due to the Valve To Pipe Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 12" Safety Injection Circumferential Weld SI-W120 would require modification of the original design of Safety Injection Piping.

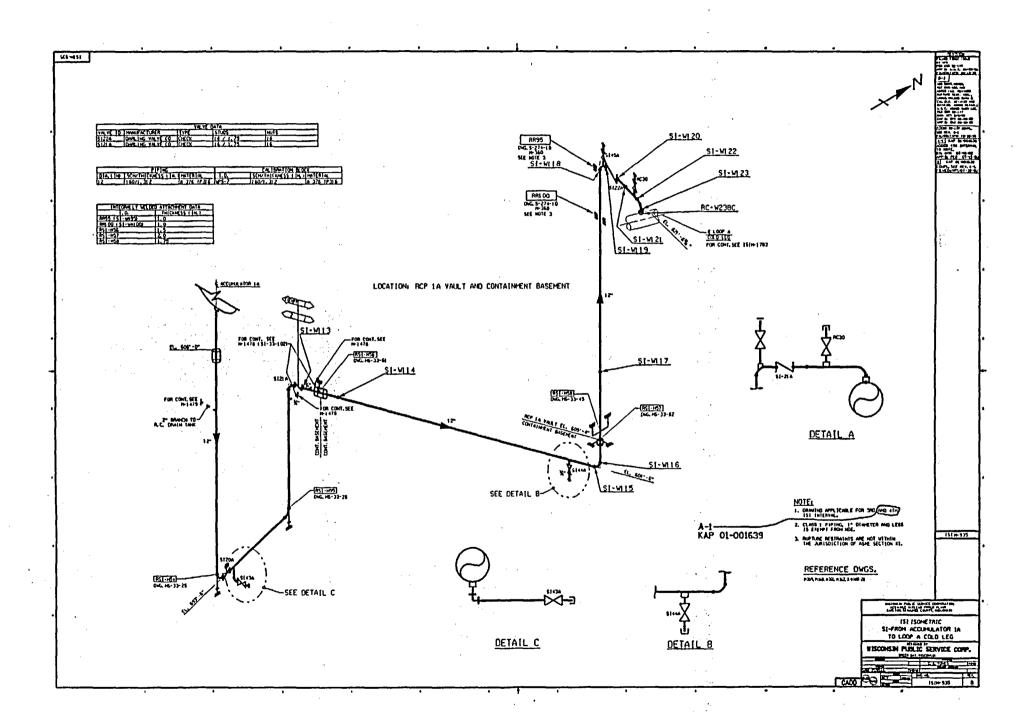
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-35

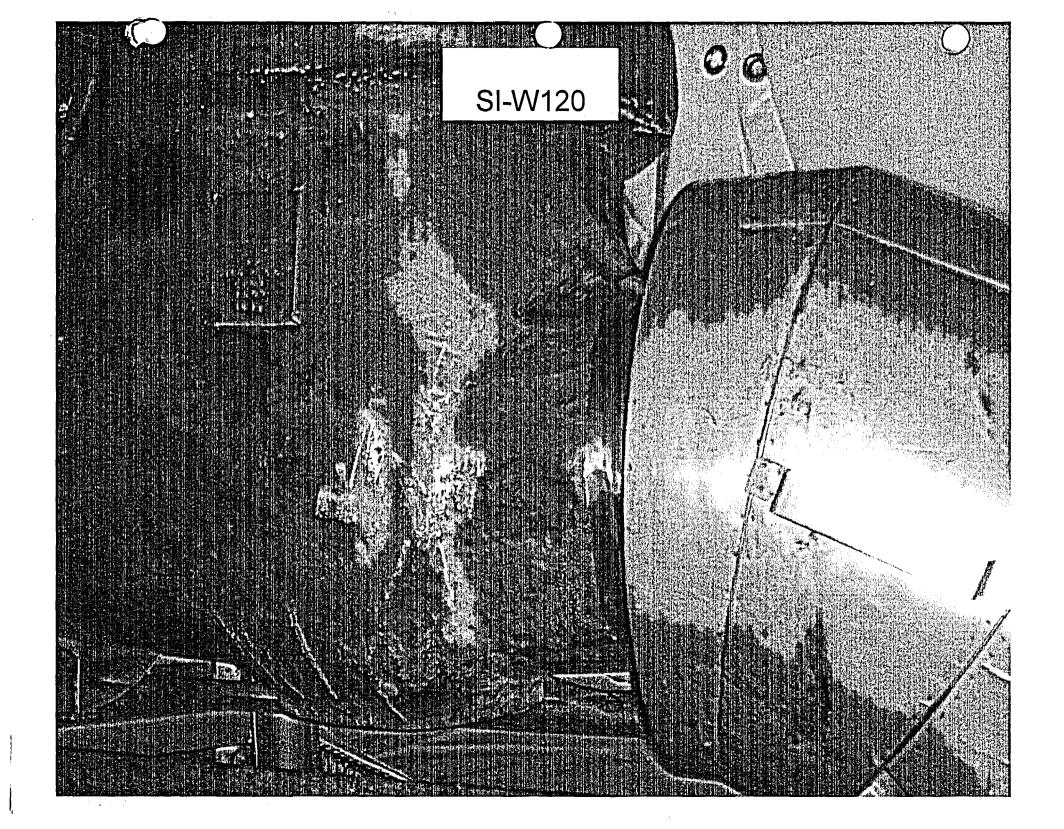
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 12" Safety Injection Circumferential Weld SI-W120 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: SI FROM ACCURALISTOR IA TO LOOPA COLLEGE DRAWING NO .: 151 M-935
COMPONENT IDENTIFICATION: SI-WIZO PROCEDURE: Q(P-911 REVISION: 026
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jell leves II DATE: 4-22-55
EXAMINER: Jan Zhon I DATE: 4-22-95 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
SI-22A SI-22A
0° scru Also Limited to Lield Allo Nownstream Base Metal Only Due to VAlue to DiDe confiduration.
4 of 5
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: April 25, 1995 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Reg. The pain DATE: 4/25/25



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-36

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 12" Safety Injection Circumferential Weld SI-W123
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 12" Safety Injection Circumferential Weld SI-W123 was inaccessible due to the Elbow to Branch Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 12" Safety Injection Circumferential Weld SI-W123 would require modification of the original design of Safety Injection Piping.

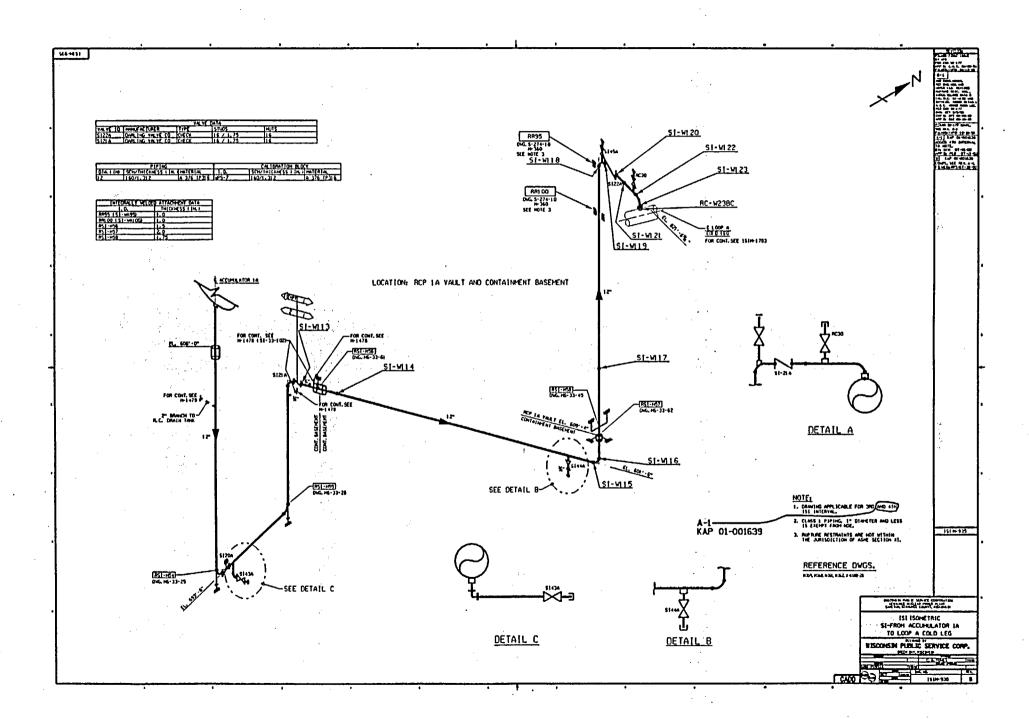
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-36

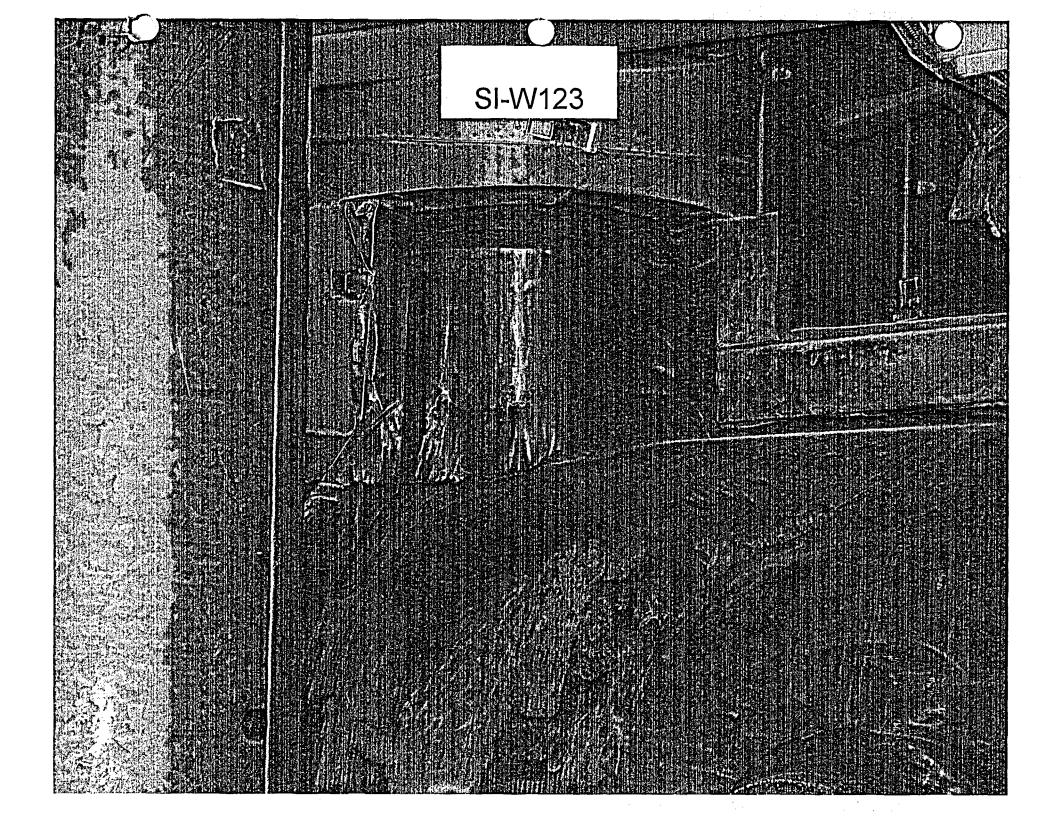
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 12" Safety Injection Circumferential Weld SI-W123 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: SI ELLON ACCOUNTAGE 14 TO LOS ACOULES DRAWING NO .: ISIM-935
COMPONENT IDENTIFICATION: SI-WIZ3 PROCEDURE: QCD -911 REVISION: 6216
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jel Jeves II DATE: 4-22-95 LEVEL EXAMINER:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
COJESAGE NOT OBTAINED - 50%
NO SCAL (2) DUE TO ELBOW TO BRANCH CONDECTION CONFIGURATION.
SCAU '7' ! '8' LIMITED TO WELD AND DOSTREAM BASE METAL ONLY OUE TO ELBOW TO BRANCH CONDECTION CONFLUENTION. DO SCAU ALSO LIMITED TO WELD AND DOSTREAM BASE METAL DUE TO ELBOW TO BRANCH LONDECTION CONFIGURATION.
<u>5055</u>
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philips C. Bukes DATE: april 34,1995
REWAUNEE NUCLEAR POWER PLANT REVIEW: Phelips C. Bukes DATE: April 34,1995 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogan Profits DATE: 4/25/95



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-37

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Safety Injection Circumferential Weld SI-W13
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 62.5% of the 6" Safety Injection Circumferential Weld SI-W13 was inaccessible due to the Valve to Pipe Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 62.5% of the 6" Safety Injection Circumferential Weld SI-W13 would require modification of the original design of Safety Injection Piping.

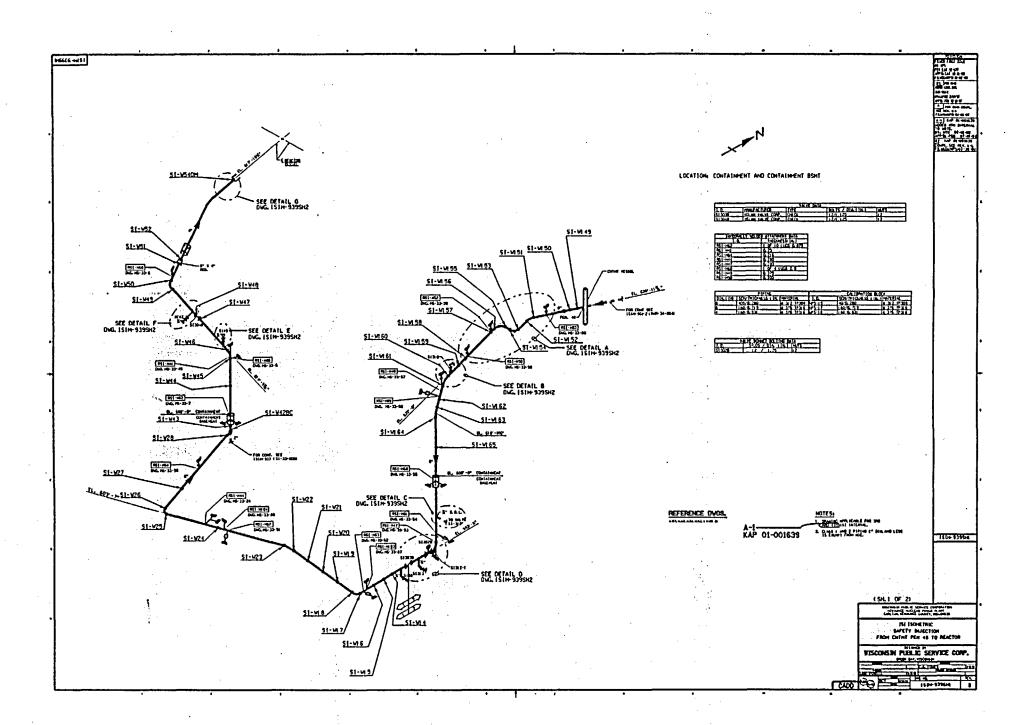
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

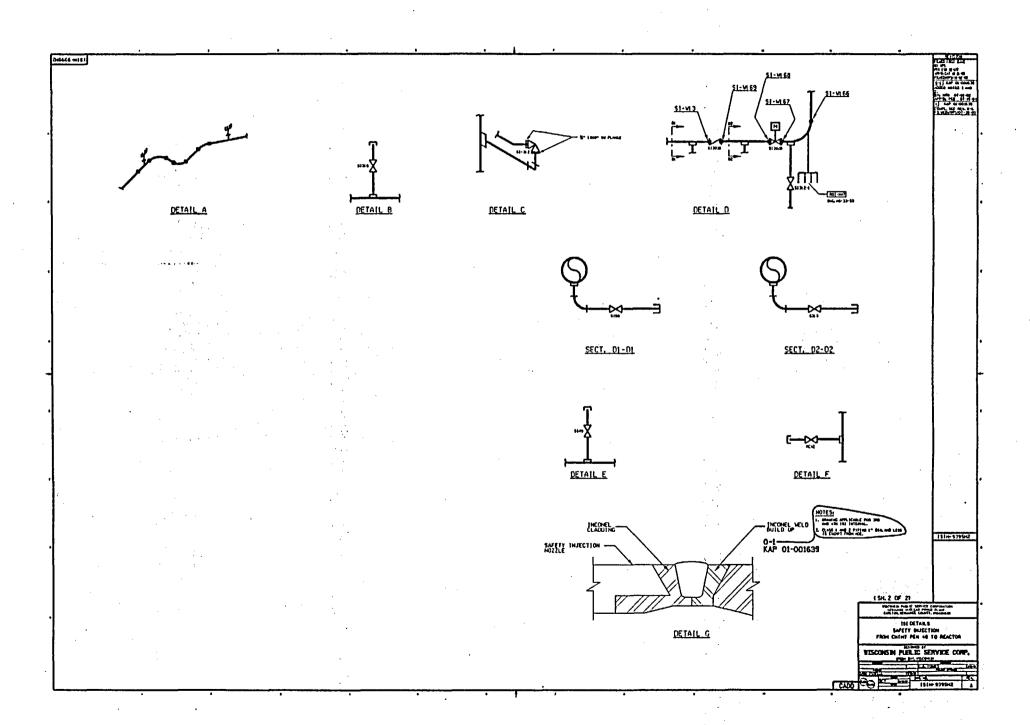
RELIEF REQUEST NO: RR-G-7-37

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Safety Injection Circumferential Weld SI-W13 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

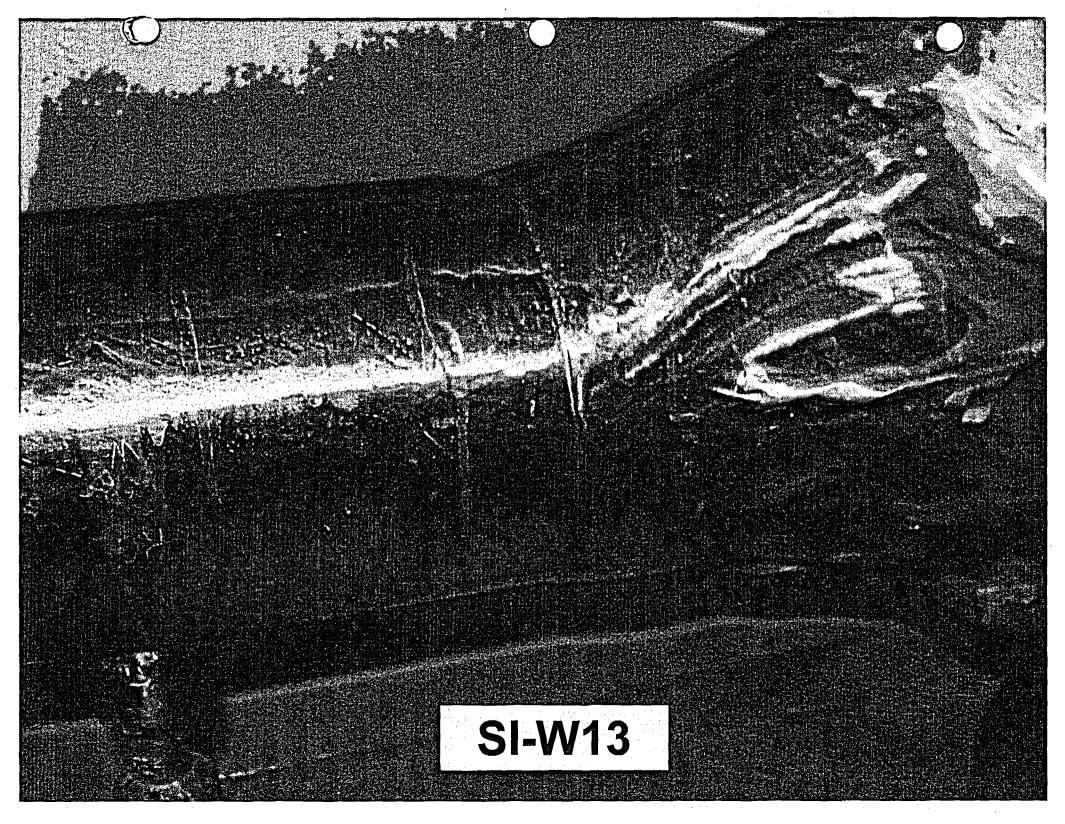
7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable





SYSTEM OR COMPONENT: CNIMT PEN 48 TO REACTOR DRAWING NO.: ISIM - 939 SHIFZ
COMPONENT IDENTIFICATION: SI-W13 PROCEDURE: QCP911 REVISION: ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: NAA. 134 II DATE: 4-8-95 LEVEL
EXAMINER: DATE: 4-8-95 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
2 SCAN SSCAN
VALUE BODY
VALUE BODY LIMITED 5,7 18 SCANS FOR 45%, 45°RL
60°S ! D° PERCENTAGE OF REDULED EXAMINATION
COVERAGE = 62.5%
Pg 3 of 4
KEWAUNEE NUCLEAR POWER PLANT REVIEW: S. A. Balaba DATE: 4/12/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Home Date: 4/12/95



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-38

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Safety Injection Circumferential Weld SI-W14
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 6.5% of the 6" Safety Injection Circumferential Weld SI-W14 was inaccessible due to a 2" Socket Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 6.5% of the 6" Safety Injection Circumferential Weld SI-W14 would require modification of the original design of Safety Injection Piping.

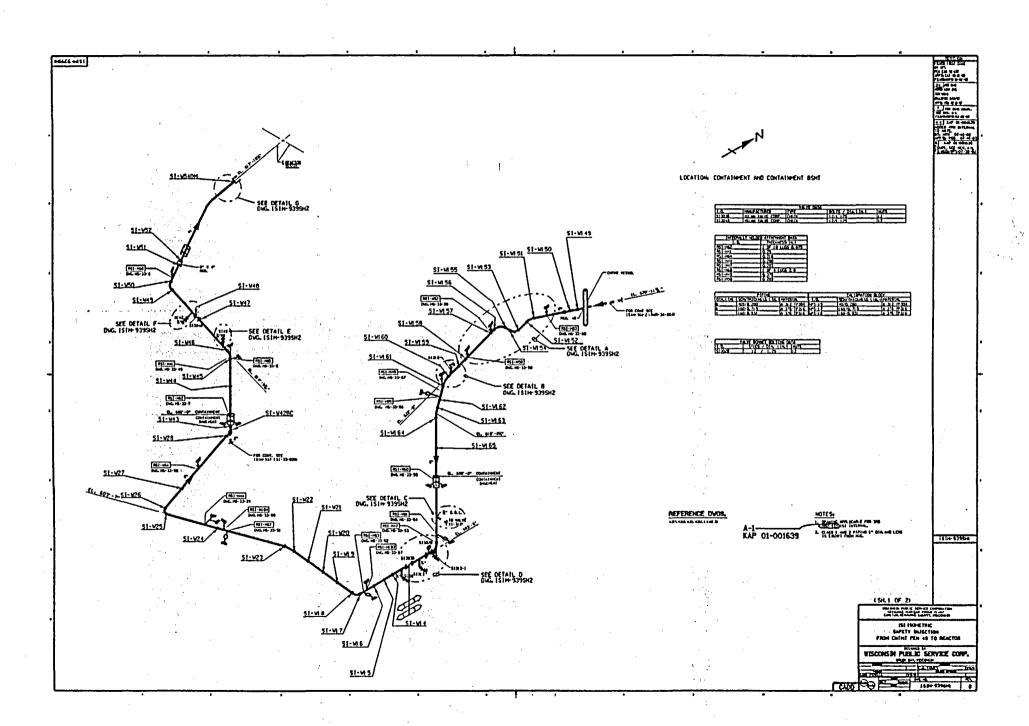
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

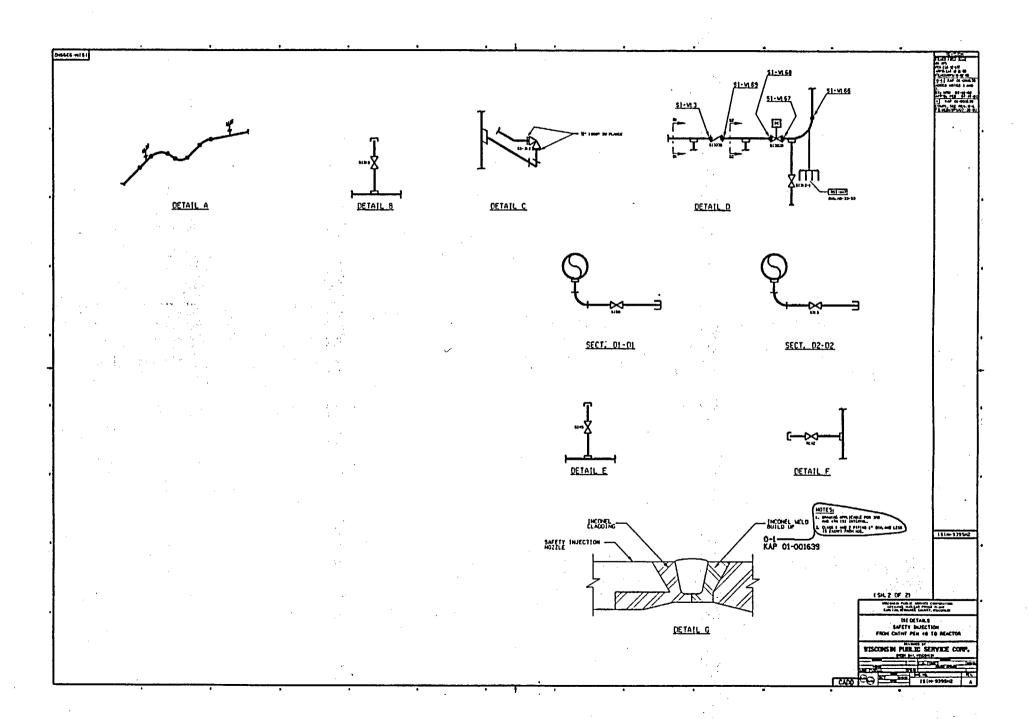
RELIEF REQUEST NO: RR-G-7-38

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Safety Injection Circumferential Weld SI-W14 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

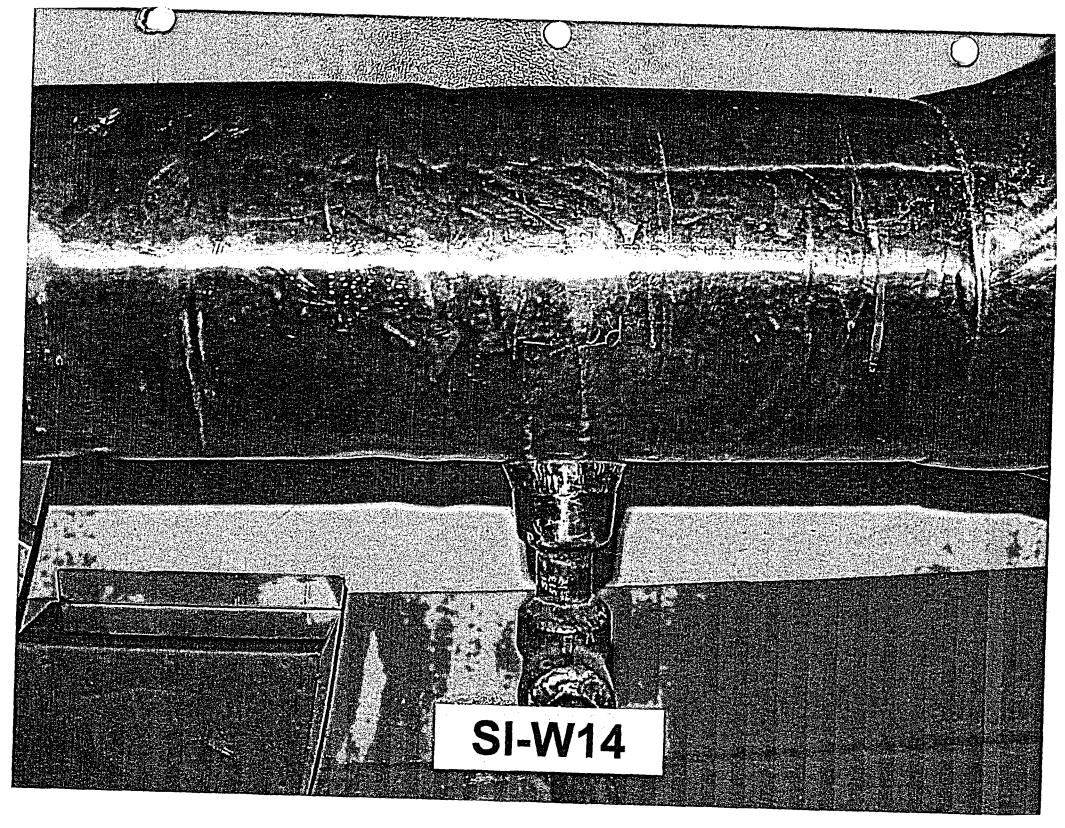
7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable





SAFETY INJECTION FROM SYSTEM OR COMPONENT: <u>CUTMT PEN 48 TO REACTOR</u> DRAWING NO.: ISIM -939 SH 1 2
COMPONENT IDENTIFICATION: SI-WIY PROCEDURE: OCP-911 REVISION: ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Nola. Beg II DATE: 4-8-75 LEVEL EXAMINER: DATE: 4-8-75 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
2 SIDE SSIDE
2.2"
WC 1.3"
SOCKET @ 8.3" TO ID.S" . 8" FROM TOE OF WELD
LIMITED SLANS 5 FOR THE 45°s, 45°RL \ 60°S
PERCENTAGE OF REDUCED EXAMINATION COVERAGE = 93.5% Pg4 of4
KEWAUNEE NUCLEAR POWER PLANT REVIEW: A Balala DATE: 4/12/95 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: 25



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-39

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Pressurizer Relief Circumferential Weld PR-W2
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 61.3% of the 6" Pressurizer Relief Circumferential Weld PR-W2 was inaccessible due to the 6" Pressurizer Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 61.3% of the 6" Pressurizer Relief Circumferential Weld PR-W2 would require modification of the original design of Pressurizer and Pressurizer Relief Nozzle.

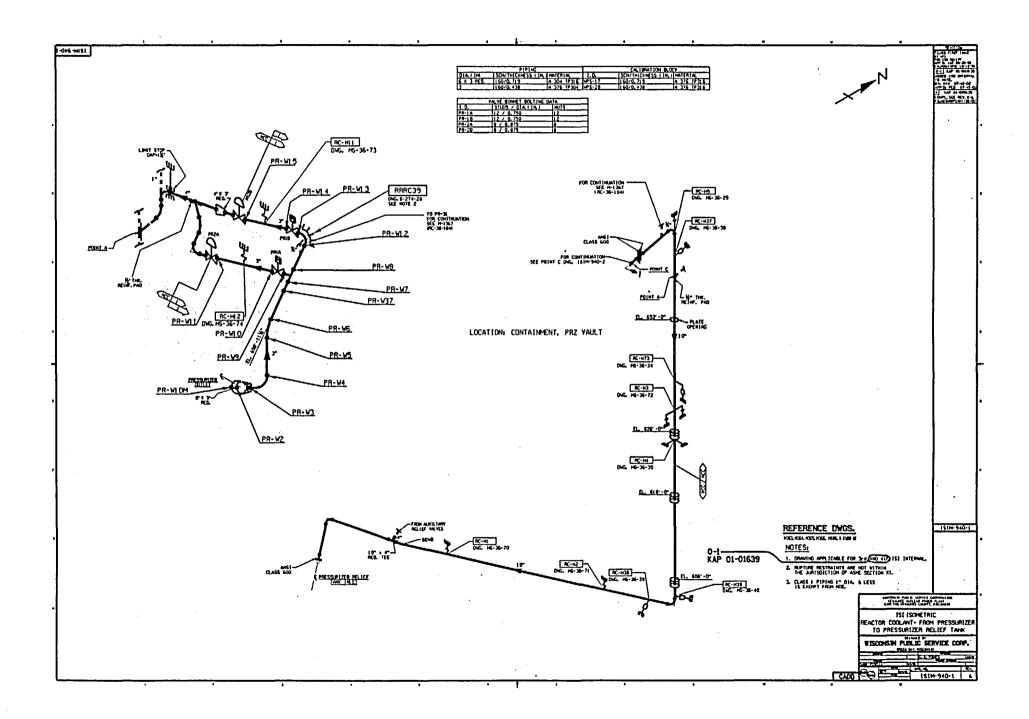
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-39

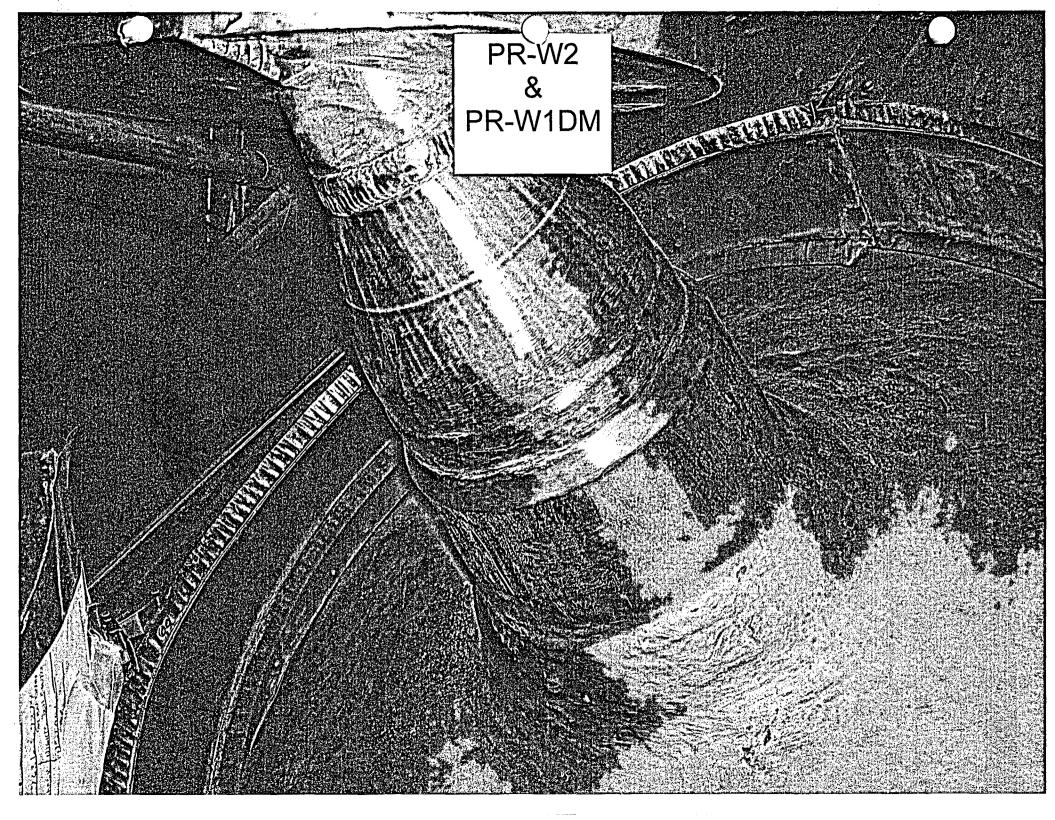
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Pressurizer Relief Circumferential Weld PR-W2 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



VIOLET AND MANUAL CONTRACTOR TO EXAMINATION THE EXAMIN			
Reactor Coolant-From Pressurizer SYSTEM OR COMPONENT: to Pressurizer Relief TANK DRAWING NO.: 151M - 940-1			
COMPONENT IDENTIFICATION: PROCEDURE: QCP-911 REVISION: OREG.			
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:			
EXAMINER: N-LA BY II DATE: 4-15-9-5			
EXAMINER: NA DATE: NA LEVEL			
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.			
Nozzle Side From Er Bow Side			
4/2			
NOZZLE CONFIGURATION RESTricted O°SCAN AND SJEAN 1748 SCAN FOR US° AND CO° Shear AND US° RL. Percentage of Reduced Examination Coverage = 61.3%			
Page 3 of 3			
KEWAUNEE NUCLEAR POWER PLANT REVIEW: E. A. Balala DATE: 4/18/95			
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Prom. Profum DATE: 4/18/95			



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-40

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 8" Residual Heat Removal
 Circumferential Weld RHR-W33
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 62.5% of the 8" Residual Heat Removal Circumferential Weld RHR-W33 was inaccessible due to the Valve Body to Pipe Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 62.5% of the 8" Residual Heat Removal Circumferential Weld RHR-W33 would require modification of the original design of Residual Heat Removal Piping.

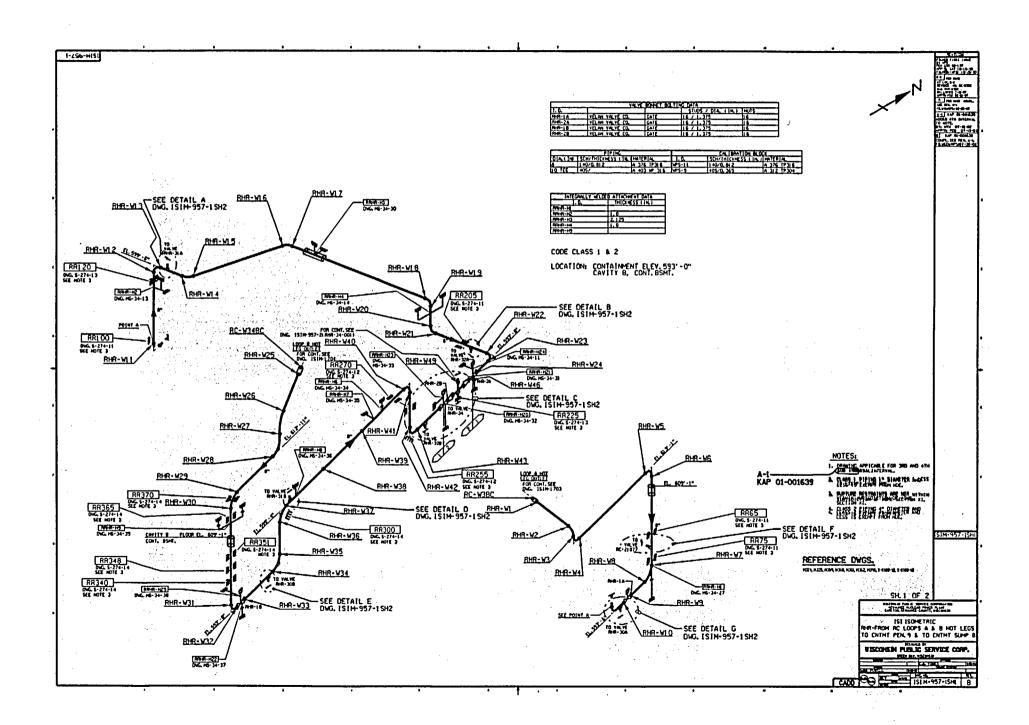
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-40

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 8" Residual Heat Removal Circumferential Weld RHR-W33 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



RHE FROM RC. LOOPS A&B HOT LEG. TO SYSTEM OR COMPONENT: CNIMT Pen. 9410 TO CNIMT SUMPB DRAWING NO .: 151M - 957-154.1		
COMPONENT IDENTIFICATION: RHR-W33 PROCEDURE: QCP 911 REVISION: ORIG.		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: NA BATE: 4-8-95 LEVEL DATE: 4-8-95		
EXAMINER:		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
VALue 5 Side		
Pipe 2 Side		
Value body Limited 45°S 60°S 45° RL AND 0°		
Percentage of Reduced Examination Coverage = 62.5 %		
Page 3 oF3		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: 2 A Balalad DATE: 4/11/95		
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Ang. Instrum DATE: 4/12/15		



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-41

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Safety Injection Circumferential Weld SI-W108
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 6" Safety Injection Circumferential Weld SI-W108 was inaccessible due to the Valve Body to Pipe Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 6" Safety Injection Circumferential Weld SI-W108 would require modification of the original design of Safety Injection Piping.

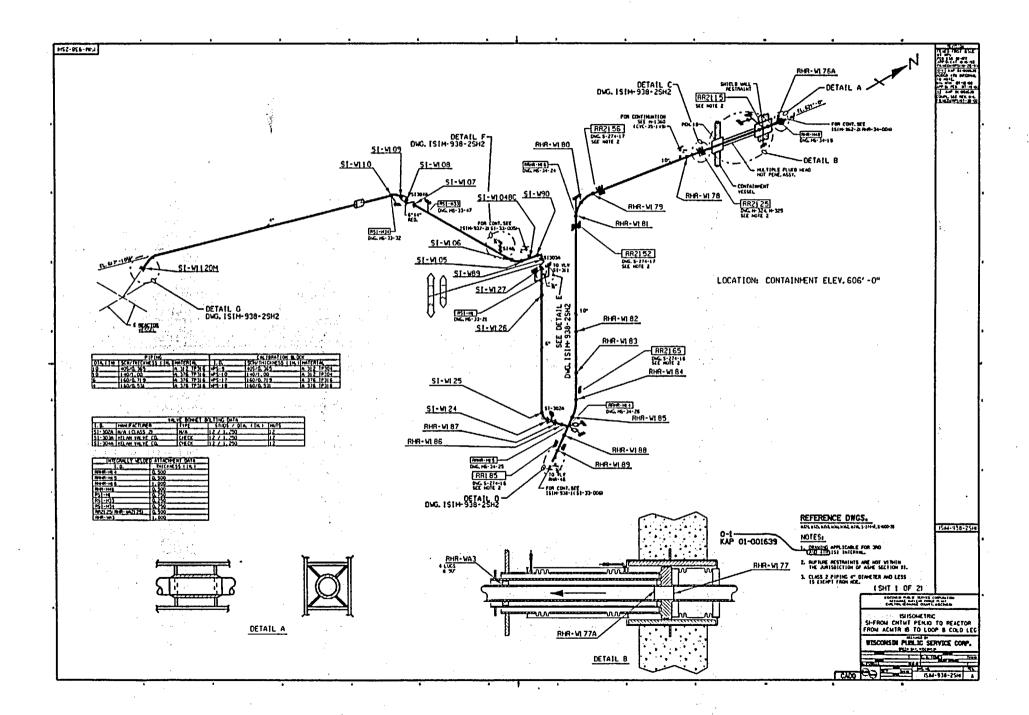
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-41

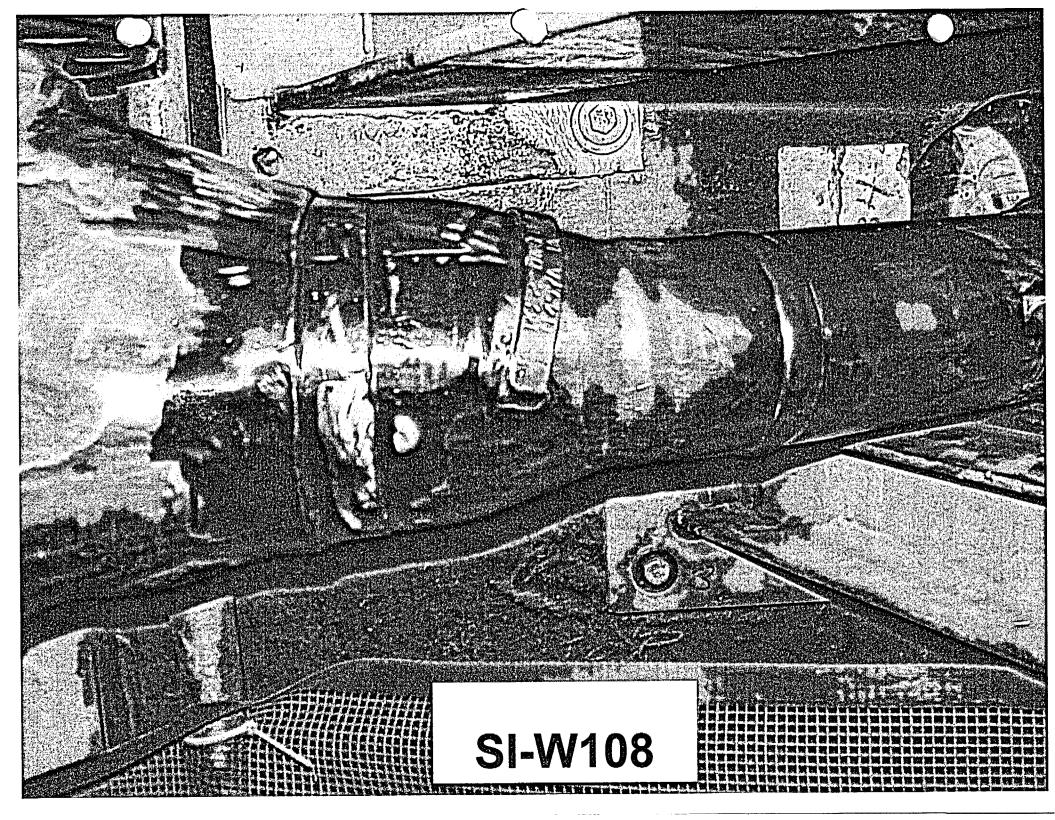
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Safety Injection Circumferential Weld SI-W108 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: SI- FROM CHIMT PEN. 10 TO REACTOR FROM ACMTR 18 TO LOOP B COLD LEG
DRAWING NO.: 151M-938-25H1
COMPONENT IDENTIFICATION: SI-WIOS PROCEDURE: NEP-15.41 REVISION: A
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Mafala II DATE: 10-30-01 LEVEL
EXAMINER:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
SI-WIOS SI-WIOS 2 SIDE 2 SIDE
FLOW
NO SCANS 5,7, \$8 ON 5 SIDE DUE TO VALUE 51-304A CONFIGURATION CODE COVERAGE REDUCED BY 50%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: November 2, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: From Within DATE: 11-2-01



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-42

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 12" Safety Injection Circumferential Weld SI-W67
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 30.0% of the 12" Safety Injection Circumferential Weld SI-W67 was inaccessible due to the O.D. Taper of 12" x 12" x 10" Tee Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 30.0% of the 12" Safety Injection Circumferential Weld SI-W67 would require modification of the original design of Safety Injection Piping.

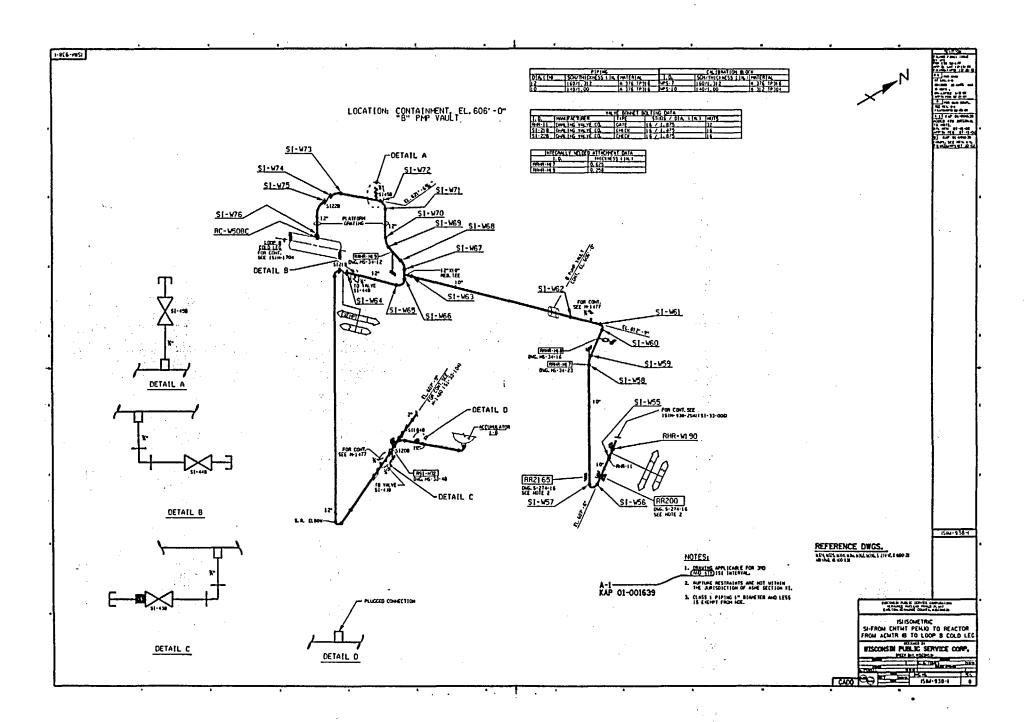
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-42

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 12" Safety Injection Circumferential Weld SI-W67 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

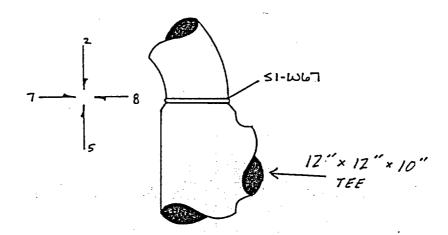
7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



SI-FROM CATHT DEN.ID TO REACTOR
SYSTEM OR COMPONENT: FROM ACMTR 18 TO LOOP B COLO LEG
DRAWING NO.: 151M-938-1
COMPONENT IDENTIFICATION: SI-WGT PROCEDURE: NED NO. 15/16 REVISION: Oris
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: DATE: 11/06/58 LEVEL DATE: 11/06/58
EXAMINER: Juy Vilone II DATE: 11-06-98 LEVEL

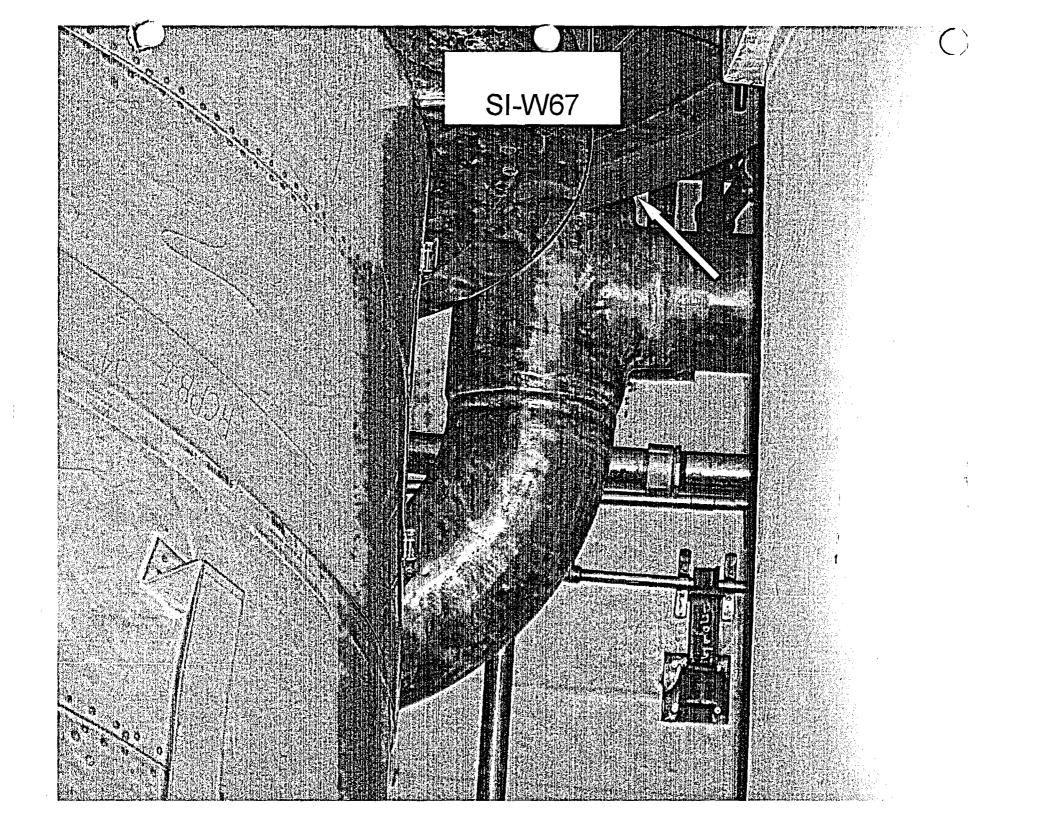
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.



NOTE: NO SCAN J, SCAN 7 ! 8 LIMITED TO WELD ! DOWN STLEAM BASE METAL ONLY DUE TO O.D. TAPER OF TEE.

PERCENT OF CODE LIMITATION: 30%
PERCENT OF PRICEOUSE LIMITATION: 45%

KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips	2 C. Bukes	DATE: November 7, 1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	Roya Materia	DATE:



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-43

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

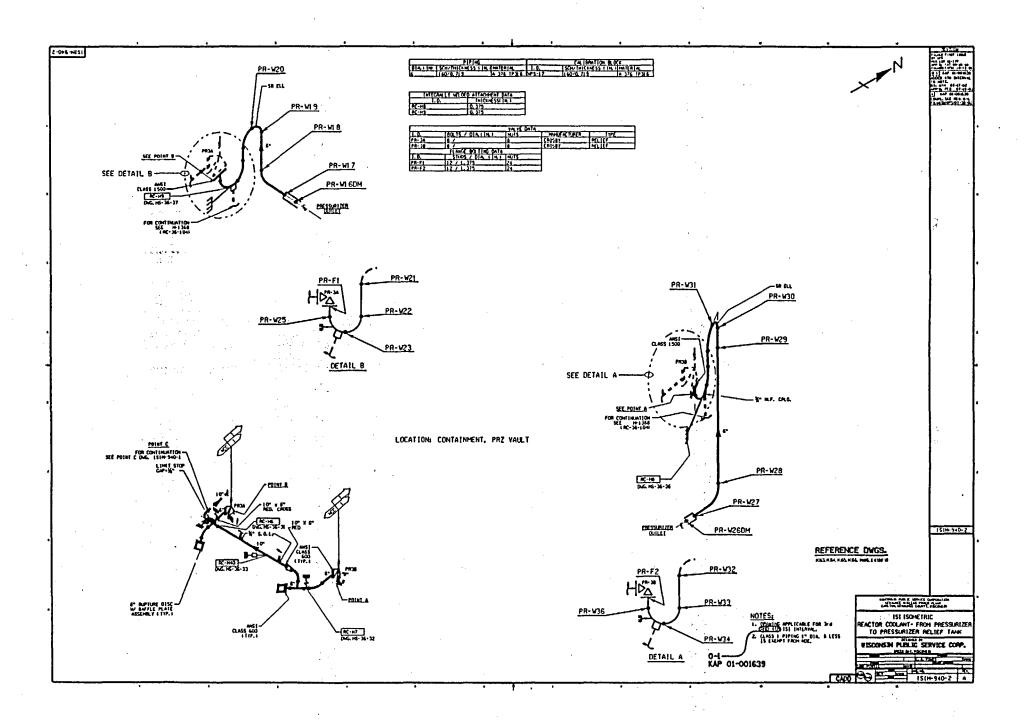
- 1. ASME Code Component Affected: 6" Pressurizer Relief Circumferential Weld PR-W17
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 51.8% of the 6" Pressurizer Relief Circumferential Weld PR-W17 was inaccessible due to the Safe End to Elbow Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 51.8% of the 6" Pressurizer Relief Circumferential Weld PR-W17 would require modification of the original design of the Pressurizer and Pressurizer Relief Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-43

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Pressurizer Relief Circumferential Weld PR-W17 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: REMITOR COMMIT-FROM PRESSURIZER TO PRESSURIZER DELIEF TANK.
DRAWING NO.: 1514 -940 - 2
COMPONENT IDENTIFICATION: PROCEDURE: NEP No. 15.16 REVISION: Ong
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jell Deves II DATE: 10/28/98 EXAMINER: DATE: 10/28/98 LEVEL DATE: 10/28/98 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
NOTE: NO SCAN 5, SCAN 7 8 LIMITED DUE TO SAFE END TO ELEOW CONFIGURATION. PELLENTAGE OF LIMITATION PER PROCEDURE 49.82 PERLENTAGE OF LIMITATION DER LODE 51.82
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes DATE: October 29,1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Fry Maynin DATE: 10-30-98

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-44

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

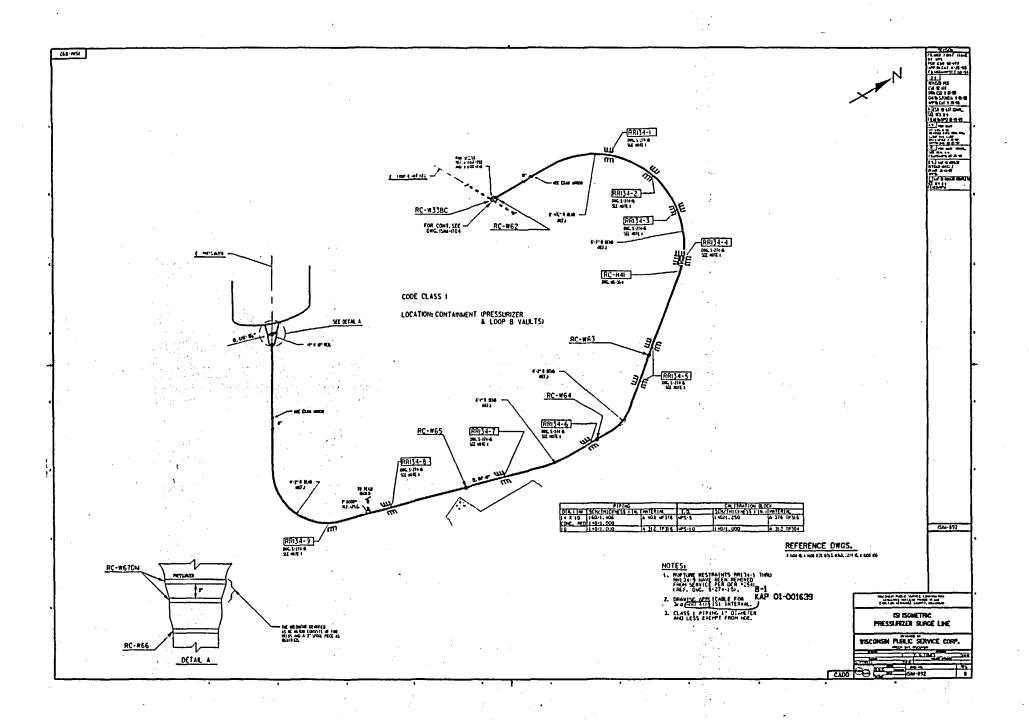
- 1. ASME Code Component Affected: 10" Pressurizer Surge Circumferential Weld RC-W64
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 21.6% of the 10" Pressurizer Surge Circumferential Weld RC-W64 was inaccessible due to Rigid Restraint RR134-6 Configuration thus restricting Ultrasonic Examination. 12.7% of the 10" Pressurizer Surge Circumferential Weld RC-W64 was inaccessible due to Rigid Restraint RR134-6 Configuration thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 21.6% and 12.7% of the 10" Pressurizer Surge Circumferential Weld RC-W64 would require modification or removal of Rigid Restraint RR134-6.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-44

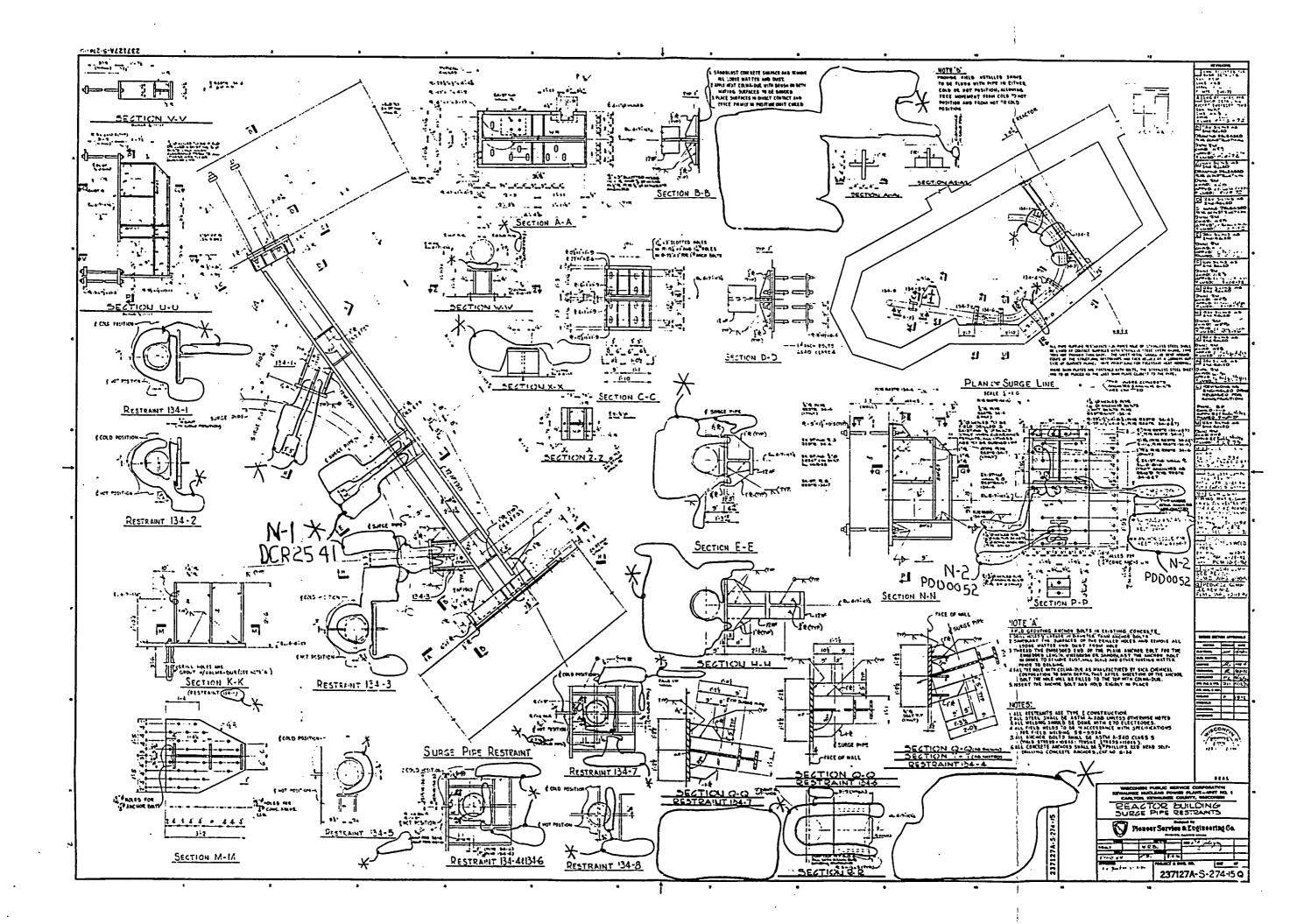
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic or Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



_		
	SYSTEM OR COMPONENT: PRESSURIZER SURGE LINE	
	DRAWING NO.: ISIM- 892	
	COMPONENT IDENTIFICATION: RC-WGU PROCEDURE: NEP ND.1516 REVISION: DRIG.	——
	ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:	
	EXAMINER: Jan 260ma II DATE: 11-18-98	·
	EXAMINER:	
	LEVEL	
	SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.)
	RESTRAINT RKI34-6	
		. •
	(RESTRAINT LIMITS ALL ANGLES D°, 45°, 60°) DUE TO LIMITED ACCESS. NON REMOVABLE LIMITATION.	
	PROCEDURAL REDUCED COVERAGE 20 %. CODE REDUCED COVERAGE 21.6 %.	
_	KEWALINEE NUCLEAR Of 14	
	POWER PLANT REVIEW: Phellips C. Bukes DATE: November 19, 19	<u> 19</u> 8
	AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogar Marin DATE: 11-20-18	

SYSTEM OR COMPONENT: PRESSURIZER SURGE LINE
DRAWING NO .: ISIM - 892
COMPONENT IDENTIFICATION: RC-W64 PROCEDURE: NEPNO.15.6 REVISION: 6 R16.
ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE: VISUAL:
EXAMINER: DATE: 11-18-98 LEVEL
EXAMINER: DATE: DATE:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
RESTRAINT RR134-6
LIMITATION AREA 2" CW AND 2" CCW
4" LIRC AREA LIMITED DUE TO NON REMOJABLE RESTRAINT.
REDUCED EXAMINATION COUERAGE
/a·7 %
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 19, 1998
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogn Mayun DATE: 11-19-98



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-45

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

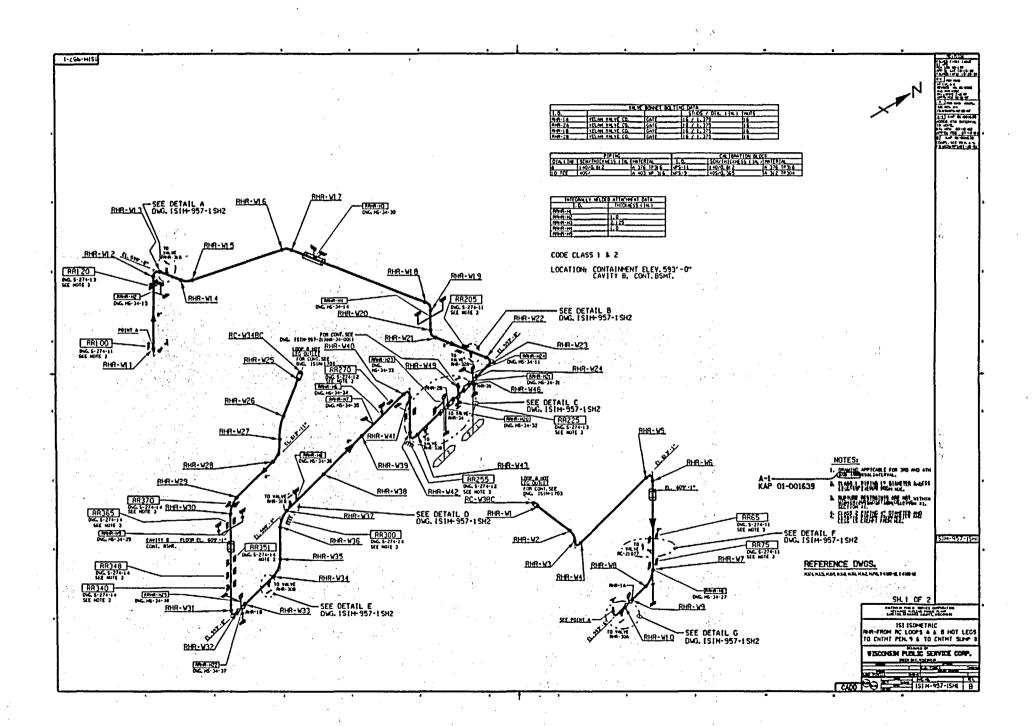
- 1. ASME Code Component Affected: 8" Residual Heat Removal
 Circumferential Weld RHR-W10
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 63.0% of the 8" Residual Heat Removal Circumferential Weld RHR-W10 was inaccessible due to the Valve Configuration and 3/4" Drain Line thus restricting Ultrasonic Examination. 1.0% of the 8" Residual Heat Removal Circumferential Weld RHR-W10 was inaccessible due to 3/4" Drain Line thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 63.0% and 1% of the 8" Residual Heat Removal Circumferential Weld RHR-W10 would require modification of the original design of the Residual Heat Removal Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-45

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic or Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

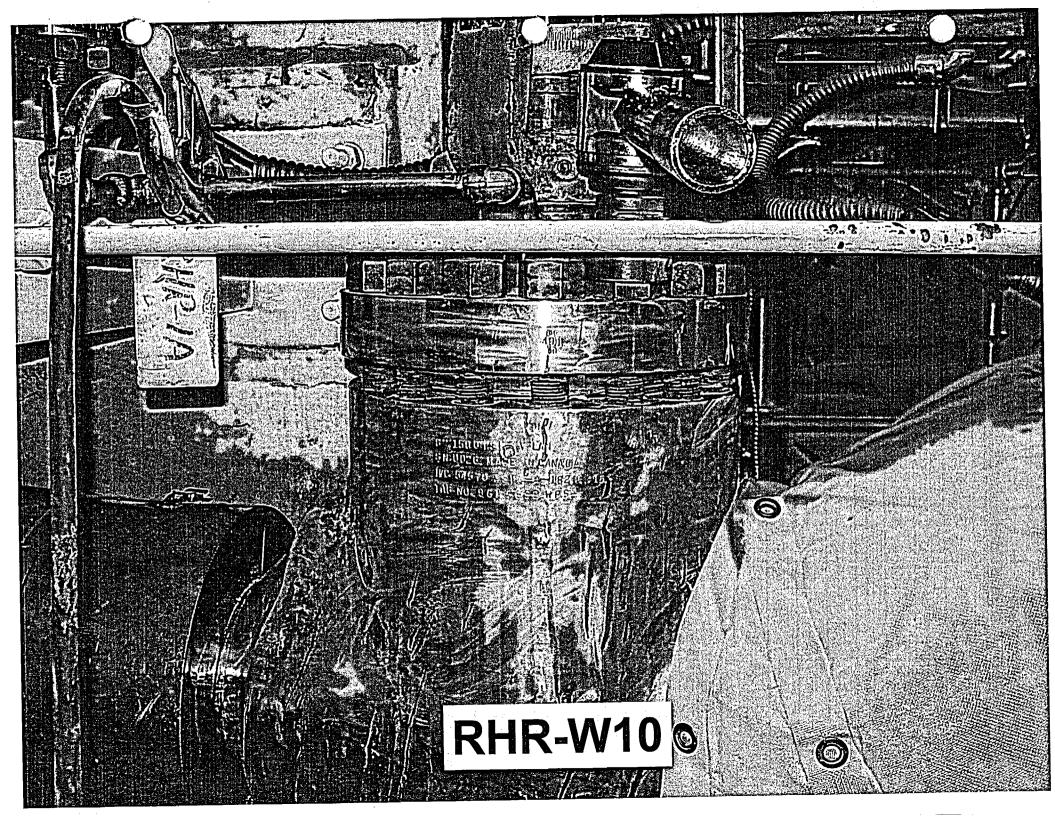
8. Precedents: Not Applicable



SYSTEM OR COMPONENT: RHR- From RC Loops A+BHo+ Legs to CHTMT Pen.9 - to CNTMT Sump B
DRAWING NO.: ISIM - 957-1 SH1
COMPONENT IDENTIFICATION: RHR-WIO PROCEDURE: NEP-15.41 REVISION: A
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER:
EXAMINER: DATE: DATE:
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
LOOKING EAST
VALVE RHR-1A VALVE RHR-1A RHR-WIO RHR-WIO
RHR-NID 34 drain line located @ BDC 44 downstream from the toe of weld RHR-WIO
45 - No scans 5,7+8 on value side due to configuration - Scan 2 limited due to drain line 60 - No scan 5 on value side due to configuration
-Scan 2 limited due to drain line Reduced Code Coverage 63%
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: October 18,2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Long Mine DATE: 10-18-01

REV.: ORIG.

SYSTEM OR COMPONENT: RHR - FROM RC LOOPS A & B HOTLEGSTO CATAT PEN. 9 & TO CATATSUMP B				
DRAWING NO.: ISIM - 95	7-1581	•		
COMPONENT IDENTIFICA	TION: RHR-WIO	_ PROCEDURE: NEPNO	1.15.6 REVISION: ORIG	
ULTRASONIC: LIC	QUID PENETRANT:	_ MAGNETIC PARTICLE	:: VISUAL:	
EXAMINER: MFRiful	1	T DATI	<u>=: 10-10-01</u>	
EXAMINER: NA		LEVEL DATE	=:	
SKETCH TO PROVIDE: A	PPROXIMATE SIZE, LOC ERCENTAGE OF REDUC	ATION, ORIENTATION, T ED EXAMINATION COVE	YPE OF LIMITATION AND ERAGE.	
LOOKING EAST VALVA RHR-IA RHA-WID FLOW		FROM TOE OF WE REDUCED CODE C	LELD RHR WID	
KEWAUNEE NUCLEAR POWER PLANT REVIEW:	Phillip C. B.	kes	DATE: October 13, 2001	
AUTHORIZED NUCLEAR INSERVICE INSPECTOR	12	Ans.	DATE: /6-/3-0/	



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-46

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

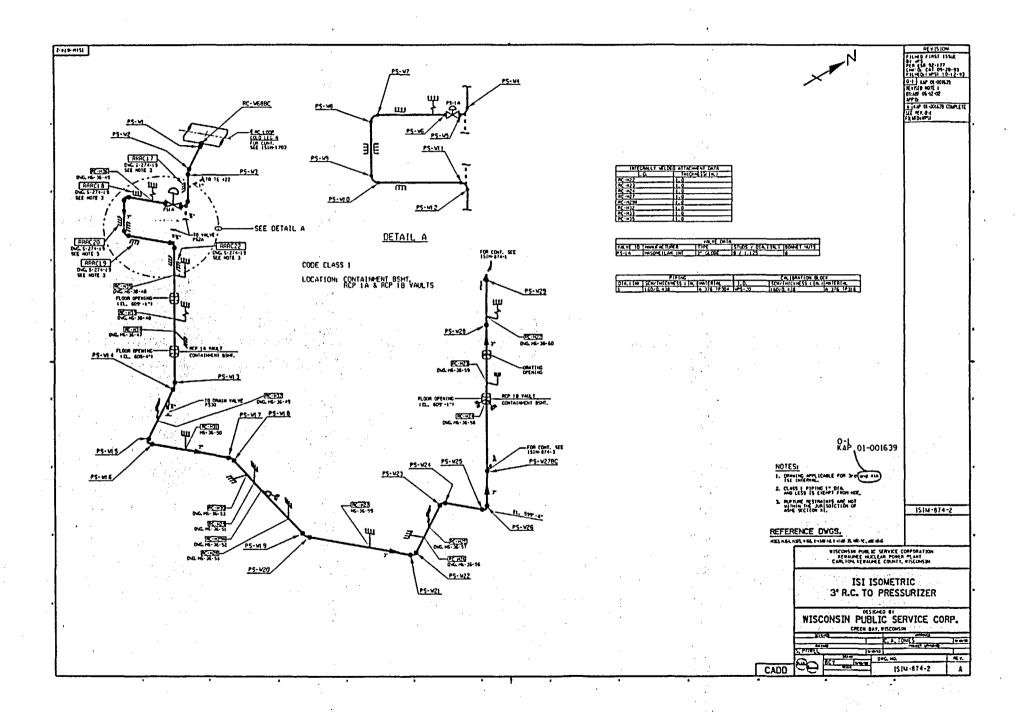
- 1. ASME Code Component Affected: 3" Pressurizer Spray Circumferential Weld PS-W4
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.21
- 4. Impracticality of Compliance: 8.8% of the 3" Pressurizer Spray Circumferential Weld PS-W4 was inaccessible due to a 3/4" Line thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 8.8% of the 3" Pressurizer Spray Circumferential Weld PS-W4 would require modification of the original design of the Pressurizer Spray Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

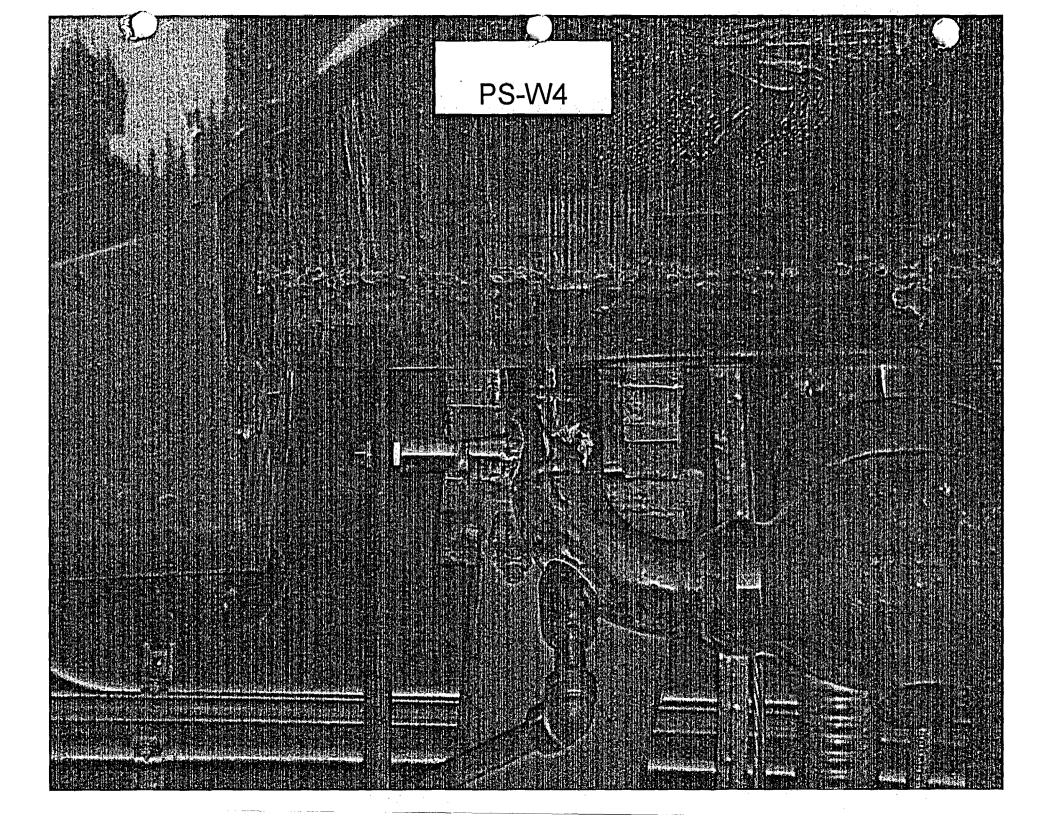
RELIEF REQUEST NO: RR-G-7-46

- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINAT	ION RECORD
SYSTEM OR COMPONENT: 3" R.C. TO PRESSURIZER	
DRAWING NO .: ISIM - 874 - 2	
COMPONENT IDENTIFICATION: PS-W4 PROCEDURE: NEP NO.	15.6 REVISION: DRIG.
ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE:	VISUAL:
EXAMINER: <u>Davis Thomas</u> II DATE:	8P-P5-01
EXAMINER: N A DATE:	ΝΑ
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TY PERCENTAGE OF REDUCED EXAMINATION COVER	
FLOW PS-WH	(1)
NOTE: NO EXAMINATION PERFORMED FOR 2.5 DOWNSTREAM BASE METAL DUE TO	5" DF 3/4" LINE.
PERCENTAGE OF EXAM LIMITATION	8.8 %
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes	DATE: <u>October 30, 199</u> 8
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW	DATE: 10-21-98



KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-47

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

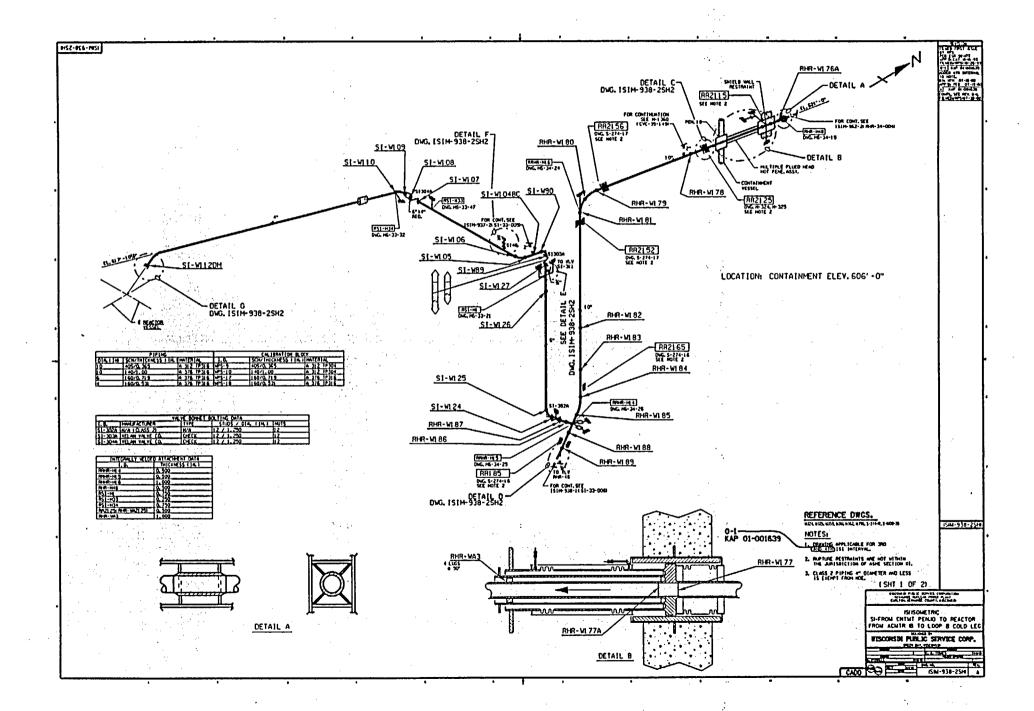
- 1. ASME Code Component Affected: 10" Residual Heat Removal Circumferential Weld RHR-W188
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.11
- 4. Impracticality of Compliance: 49.8% of the 10" Residual Heat Removal Circumferential Weld RHR-W188 was inaccessible due 10" x 10" x 6" Tee Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 49.8% of the 10" Residual Heat Removal Relief Circumferential Weld RHR-W188 would require modification of the original design of the Residual Heat Removal Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

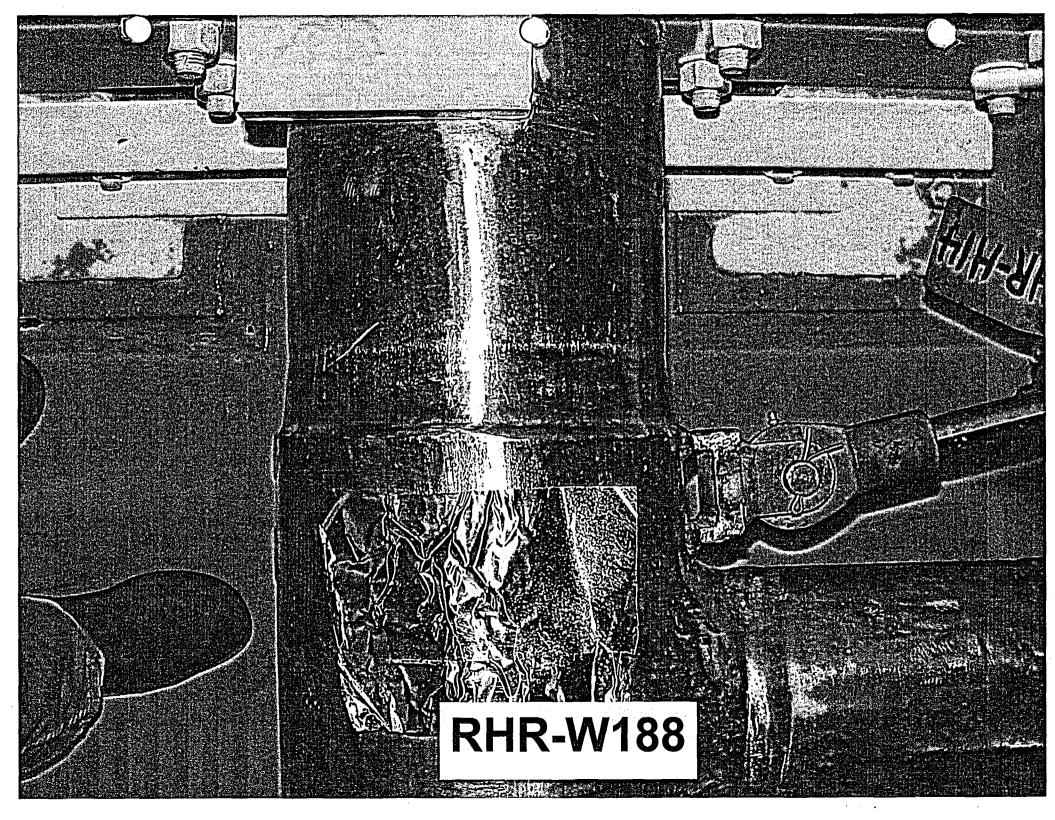
RELIEF REQUEST NO: RR-G-7-47

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 10" Residual Heat Removal Circumferential Weld RHR-W188 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SI - FROM CHT MT Pen 10 TO REALOR SYSTEM OR COMPONENT: From ACMTR 18 TO Loop B COLL Leg. DRAWING NO.: 151M -938-25H1		
COMPONENT IDENTIFICATION: RHR-W 188 PROCEDURE: QCP 911 REVISION: ORIG		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
1 l 1 2 es.		
EXAMINER: NA A. Bog II DATE: 4-10-95 LEVEL		
EXAMINER: NA DATE: NA		
LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND		
PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		
Pipe		
Pipe 2 Size Weld 55:08		
Tee Configuration Limited 0° SCAN AND 5 SCAN FOR 45°S, 60°S AND 45°RL.		
Percentage of Reduced Examination Coverage = 49.8%		
PAje 2 of 3		
KEWAUNEE NUCLEAR POWER PLANT REVIEW: E. A. Balata DATE: 4/12/95		
AUTHORIZED NUCLEAR		
INSERVICE INSPECTOR REVIEW: Program DATE: 4/12/55		



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-48

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 10" Residual Heat Removal
 Circumferential Weld RHR-W414
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.11
- 4. Impracticality of Compliance: 59.0% of the 10" Residual Heat Removal Circumferential Weld RHR-W414 was inaccessible due to Valve Configuration and 2- 3/4" Line Weldolets thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 59.0% of the 10" Residual Heat Removal Relief Circumferential Weld RHR-W414 would require modification of the original design of the Residual Heat Removal Piping.

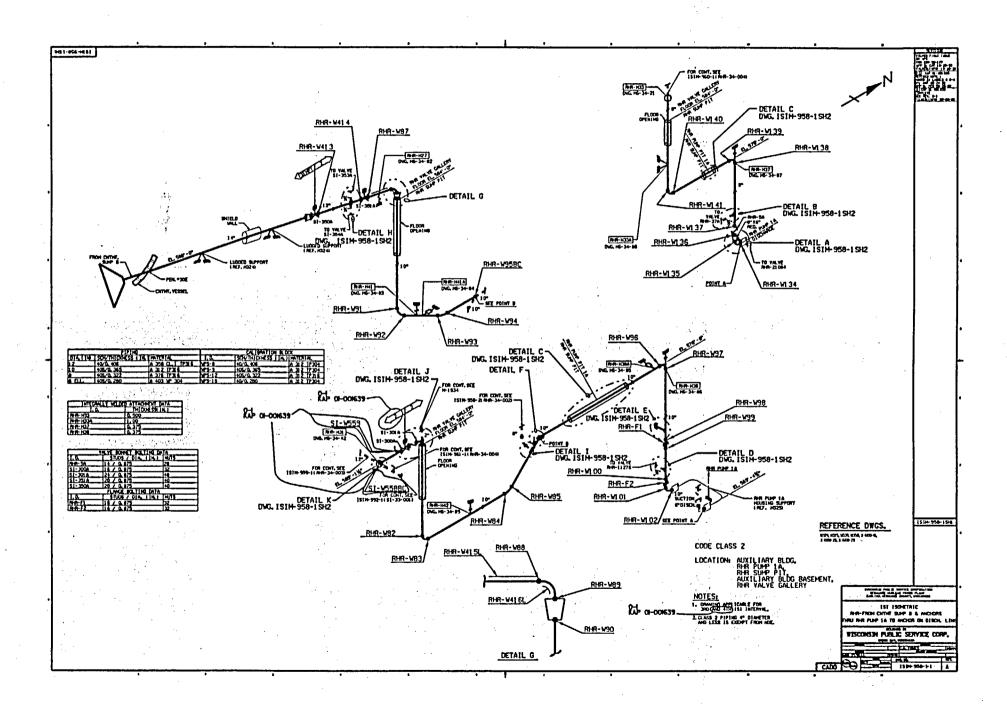
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-48

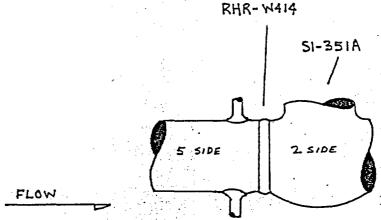
6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 10" Residual Heat Removal Circumferential Weld RHR-W414 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD		
RHR- FROM CNTMT SUMP B AND ANCHORS THRU RHR PUMP IA		
SYSTEM OR COMPONENT: TO ANCHOR ON DISCH. LINE.		
DRAWING NO.: 151M-958-15H1		
COMPONENT IDENTIFICATION: RHR-W414 PROCEDURE: NEF-15,41 REVISION: A		
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:		
EXAMINER: MATCHE DATE: 10-24-01 LEVEL		
EXAMINER: NA DATE: NA		
LEVEL		
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.		



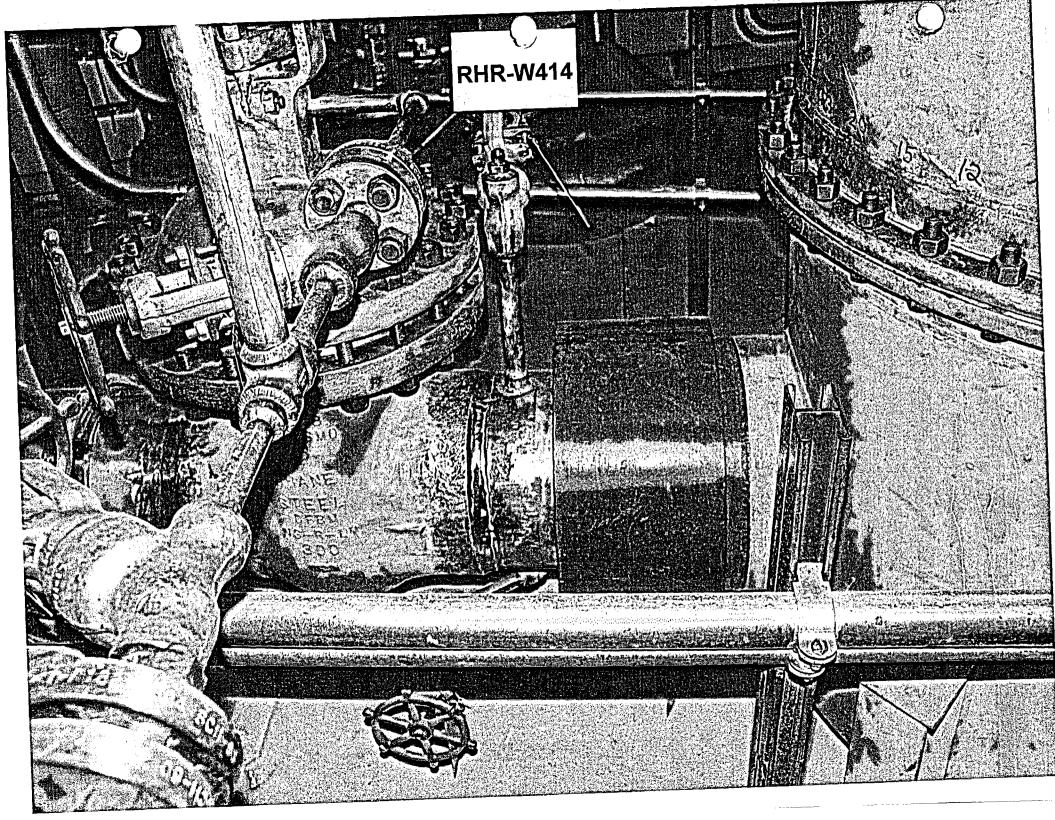
NO SCANS 2.7. AND 8 ON 2 SIDE DUE TO VALVE CONFIGURATION.

45° AND 70° SCAN 5 LIMITED FOR 2" AT TDC AND FOR 2" AT BDC DUE TO WELD O'LETS.

WELD O'LETS LOCATED 3/4" FROM TOE OF WELD AT T.D.C. AND 3/4" FROM TOE

OF WELD AT B.D.C.

WELD O'LETS LOCATED 3/4 FROM TOE OF WELD AT T.D.C. AM	10 74 FROM TOE
OF WELD AT B.D.C.	
그 가는 생생 나는 그를 맛있죠? 중요 선생님들이 얼마나 그는 가는 것이다.	
CODE COVERAGE REDUCED BY 59%	
KEWAUNEE NUCLEAR	. 41
POWER PLANT REVIEW: Phillip C. Bukes	_ DATE: October 25,2001
AUTHORIZED NUCLEAR / 7.1	
INSERVICE INSPECTOR REVIEW: Ky Myuni	DATE: 10-26 +1
<u> </u>	



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-49

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

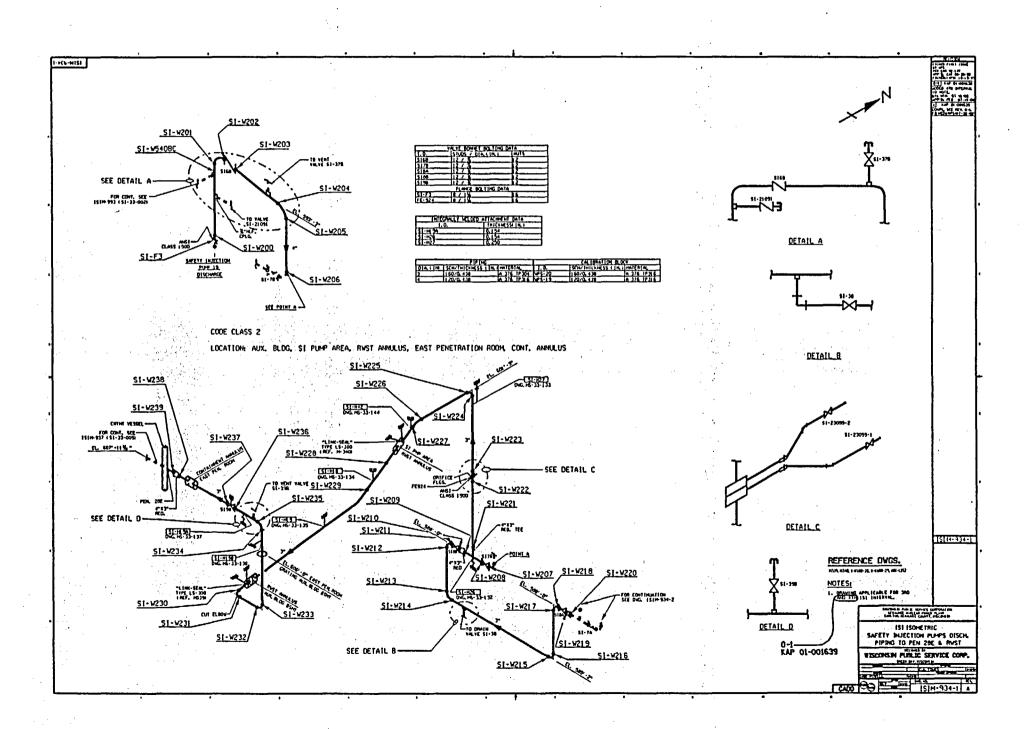
- 1. ASME Code Component Affected: 3" Safety Injection Circumferential Weld SI-W234
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.21
- 4. Impracticality of Compliance: 20.0% of the 3" Safety Injection Circumferential Weld SI-W234 was inaccessible due to the Elbow Intradose Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 20.0% of the 3" Safety Injection Circumferential Weld SI-W234 would require modification of the original design of the Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-49

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 3" Safety Injection Circumferential Weld SI-W234 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.21.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



THE STATE OF THE S
SYSTEM OR COMPONENT: DISCURSES PARE FROM PED. 365 ! RUSTDRAWING NO.: 151M - 934-1
COMPONENT IDENTIFICATION: SI-6234 PROCEDURE: QCP-911 REVISION: ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: July Bly DATE: 4-22-95
EXAMINER: NA DATE: NA
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
45° SHEAR AND 60° SHEAR, SLAN Z, LIMITED CONTACT FROM 6" TO 10" ON INTRADESE OF ELBON DUE TO WRIATURE
of Elbow.
45° SHEAR - 2090 REDUCED EXAM. COUERAGE
600 SHEAR - 2070 REDUCED EXAM. COVERAGE.
SI-W234 LINTRADOSE OF ELACU
FLOW FAGE 1 OF 2
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philips C. Bukes DATE: April 24, 1995
POWER PLANT REVIEW: Thelips C. Bukes DATE: Opul 24, 1995 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Frage Profus DATE: 4/25/85

KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-50

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

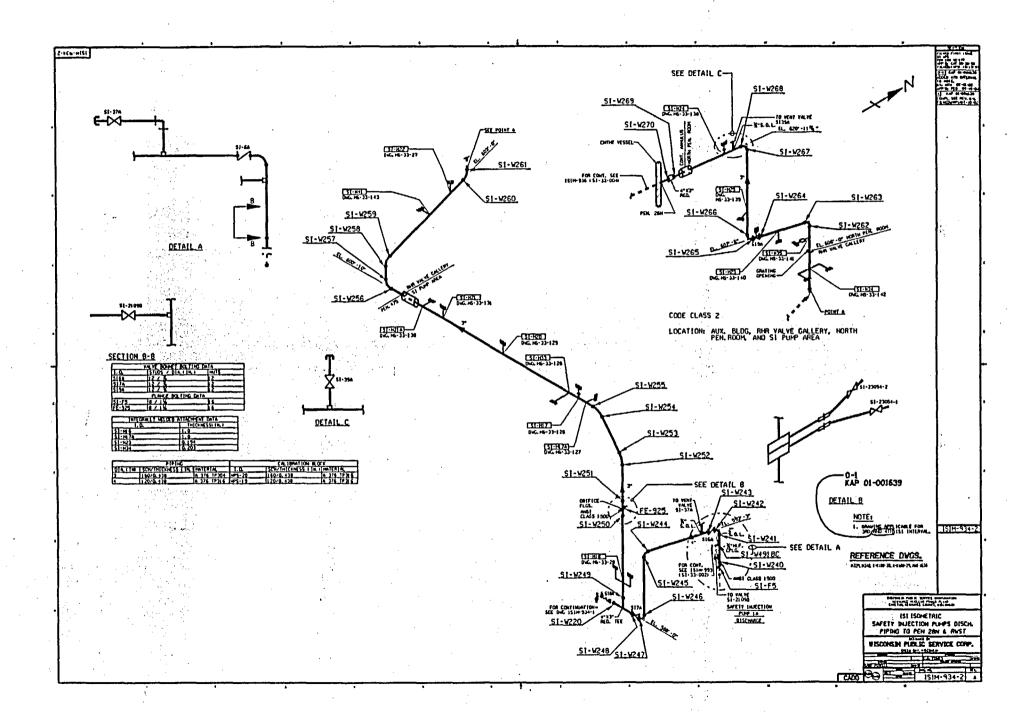
- 1. ASME Code Component Affected: 3" Safety Injection Circumferential Weld SI-W262
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.21
- 4. Impracticality of Compliance: 20.0% of the 3" Safety Injection Circumferential Weld SI-W262 was inaccessible due to the Elbow Intradose Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 20.0% of the 3" Safety Injection Circumferential Weld SI-W262 would require modification of the original design of the Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

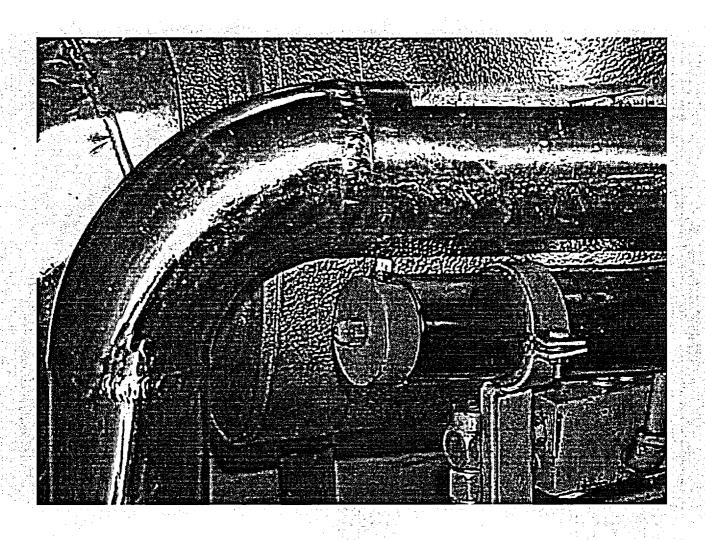
RELIEF REQUEST NO: RR-G-7-50

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 3" Safety Injection Circumferential Weld SI-W262 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.21.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SAFETY INJECTION DUMPS SYSTEM OR COMPONENT: DECKARGE PANO TO PEN 284 ; RUST DRAWING NO.: 151 N - 934-2
COMPONENT IDENTIFICATION: St - W262 PROCEDURE: QCP-911 REVISION: 0816
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Johl Blif II DATE: 4-22-55
EXAMINER: DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND
PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
45° SHEAR AND 60° SHEAR, SCAN 2. LIMITED
CONTACT Flow 6" TO 10" ON INTERBOSE OF ELACU JUE TO CUENTICE OF ELACUD.
45° SHEAR - 2090 REDUCED EXAM. COUERAGE.
BO SHEKE! ZOTO TOLLES CALL SOCIATION
SI-WZWZ ZSIAW INTRADOSE OF ELAOW
PAGE 1 c = 1
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes DATE: April 2 4 1995
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Any Mate Date: 4/25/95



3" Safety Injection Weld SI-W262

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-51

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

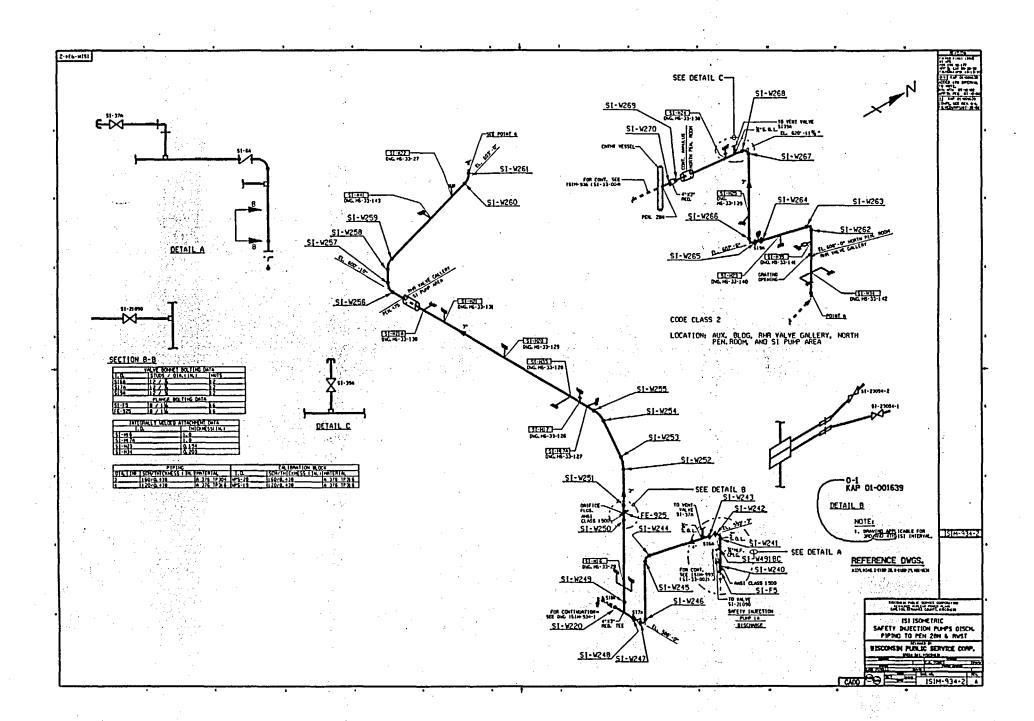
- 1. ASME Code Component Affected: 3" Safety Injection Circumferential Weld SI-W249
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.21
- 4. Impracticality of Compliance: 15.5% of the 3" Safety Injection Circumferential Weld SI-W249 was inaccessible due to 4" x 4" x 3" Reducing Tee Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 15.5% of the 3" Safety Injection Circumferential Weld SI-W249 would require modification of the original design of the Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-51

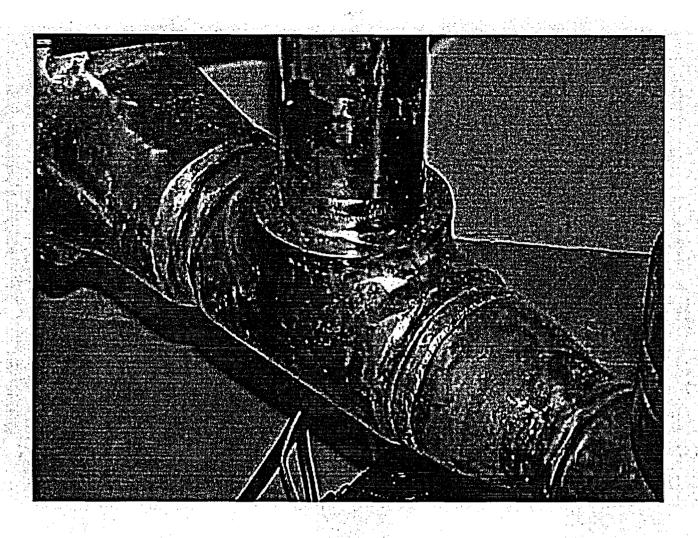
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 3" Safety Injection Circumferential Weld SI-W249 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.21.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD
SAFETY INJECTION PUMPS SYSTEM OR COMPONENT: DISCH, PIPING TO PEN. 28N AND RWST
DRAWING NO.: 151M - 934 - 2
COMPONENT IDENTIFICATION: <u>S1-W249</u> PROCEDURE: <u>NFPNo. 15.16</u> REVISION: <u>Orig</u>
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jug Wilsonse II DATE: 11-9-98 LEVEL
EXAMINER: <u>Jeant Homan</u> II DATE: 11-9-98 LEVEL
OVETOU TO DECUME. ADDROVIMATE SIZE LOCATION OF DESIGNATION TYPE OF UNITATION AND
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
WCW = .625"
THICKNESS = . 438"
51-W249

REDUCED CODE COVERAGE: 15.5%
4×3" RED TEE REDUCED PROCEDURE COVERAGE: 40.5 %
SCANS 5, 7, AND 8 LIMITED DUE TO TEE CONFIGURATION
- SEE ABOVE SKETCH FOR DIMENSIONS
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: November 11, 1998
ALITHORIZED NUICLEAR
INSERVICE INSPECTOR REVIEW: Koya Morries DATE: 11-12-78



3" Safety Injection Weld SI-W249

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-52

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 3" Safety Injection Circumferential Weld SI-W307
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.21
- 4. Impracticality of Compliance: 47.5% of the 3" Safety Injection Circumferential Weld SI-W307 was inaccessible due to 3" x 3" x 3" Tee Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 47.5% of the 3" Safety Injection Circumferential Weld SI-W307 would require modification of the original design of the Safety Injection Piping.

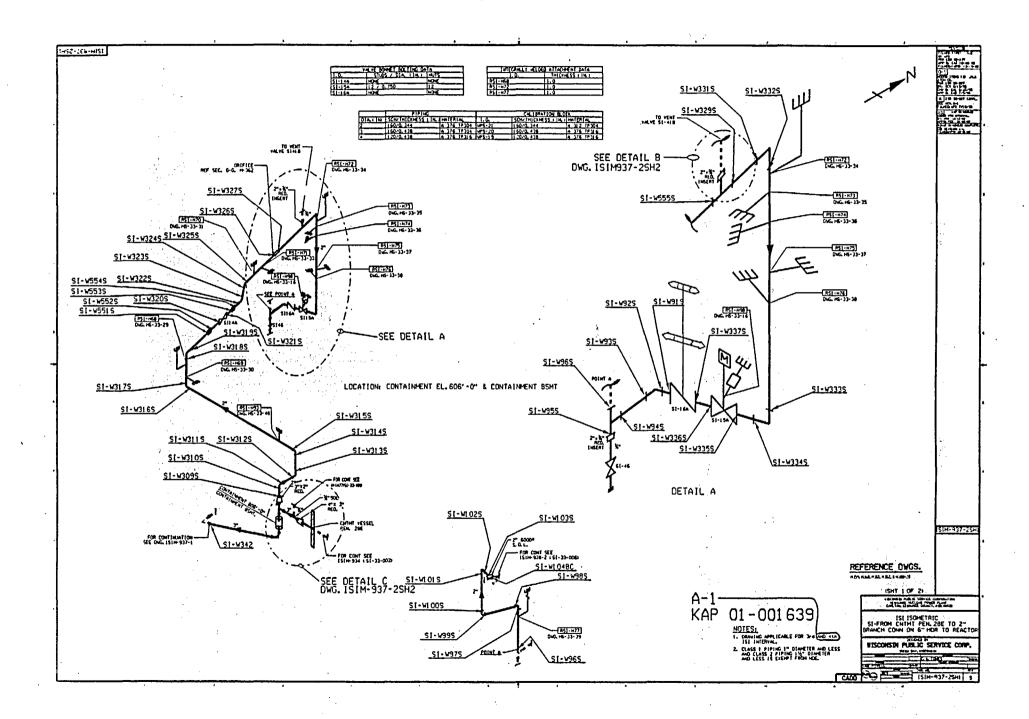
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

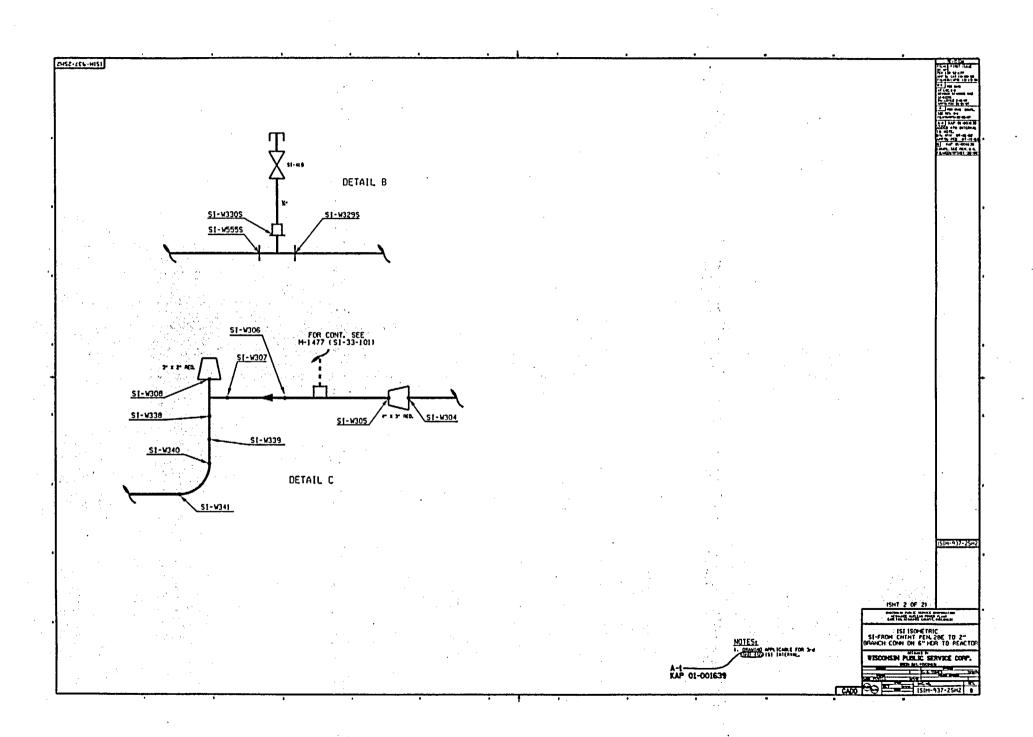
RELIEF REQUEST NO: RR-G-7-52

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Liquid Penetrant Examination was performed on 3" Safety Injection Circumferential Weld SI-W307 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-1 and Item No. C5.21.

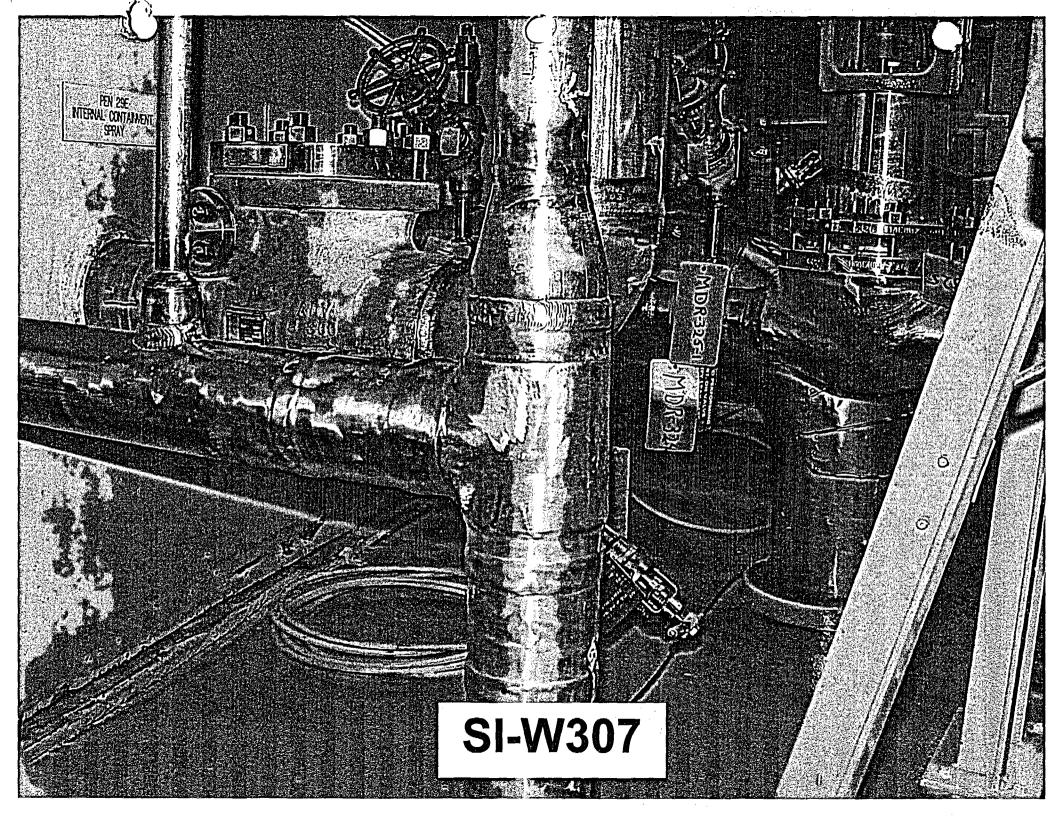
7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable





SYSTEM OR COMPONENT: SI - FROM CATAT PEN.	28E TO 2" BRANCH CONN ON 6"	HOR TO REACTOR
DRAWING NO .: ISIM - 437-25H2		
COMPONENT IDENTIFICATION: 51-W307	PROCEDURE: <u>NEP+15,41</u>	REVISION:A
ULTRASONIC: X LIQUID PENETRANT:	MAGNETIC PARTICLE:	VISUAL:
EXAMINER: TIM COBURN AM	DATE:	10/16/01
EXAMINER: MSkilpek	DATE:	10-16-01
	LEVEL	
SKETCH TO PROVIDE: APPROXIMATE SIZE, LO PERCENTAGE OF REDU	CATION, ORIENTATION, TYPE CED EXAMINATION COVERAGE	OF LIMITATION AND
2SIDE SSIDE SIDE	45,°70°- NOSCAN 2 DUETOT	EE CONFIGURATION
	REDUCED CODE COVERAGE 47.	<i>57</i> .
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C- Be AUTHORIZED NUCLEAR	7.1.	ATE: 0-19-01



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-53

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

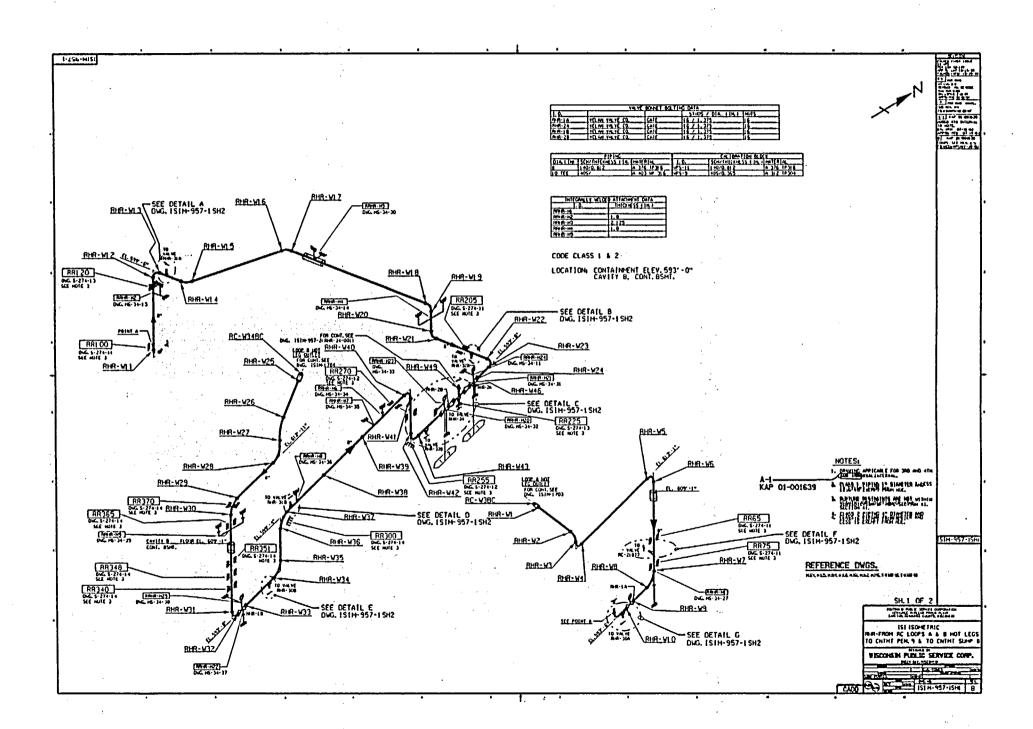
- 1. ASME Code Component Affected: 10" Residual Heat Removal Circumferential Weld RHR-W48
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-1; Item No. C5.13
- 4. Impracticality of Compliance: 7.0% of the 10" Residual Heat Removal Circumferential Weld RHR-W48 was inaccessible due to the Box Rigid Restraint RR225 Configuration thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 7.0% of the 8" Residual Heat Removal Circumferential Weld RHR-W48 would require removal of the Box Rigid Restraint RR225.

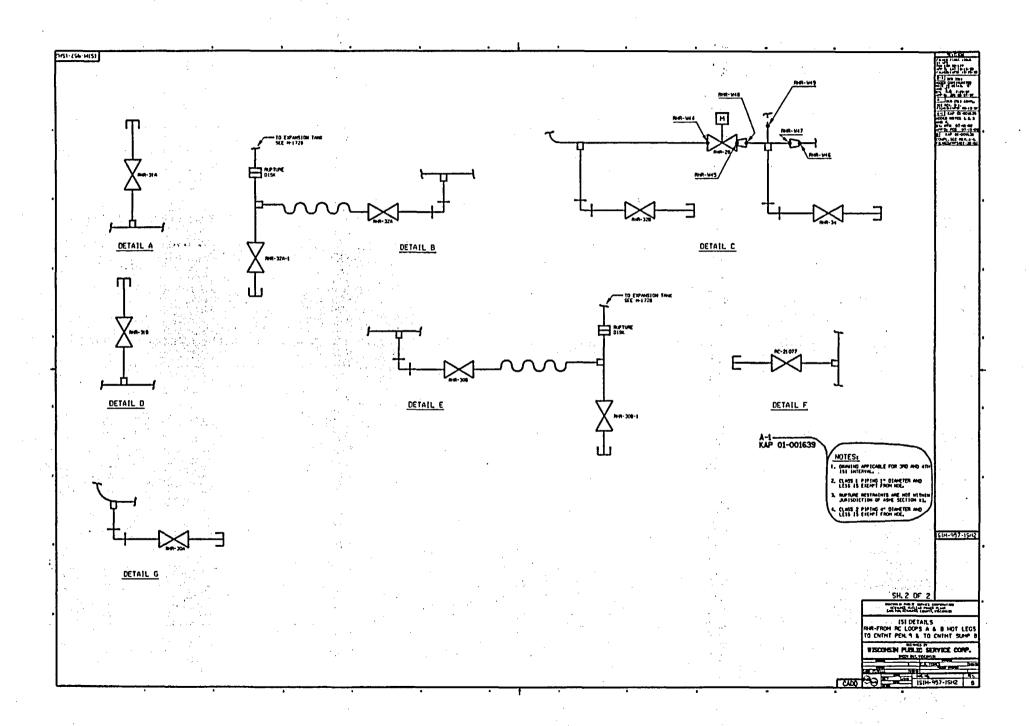
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-53

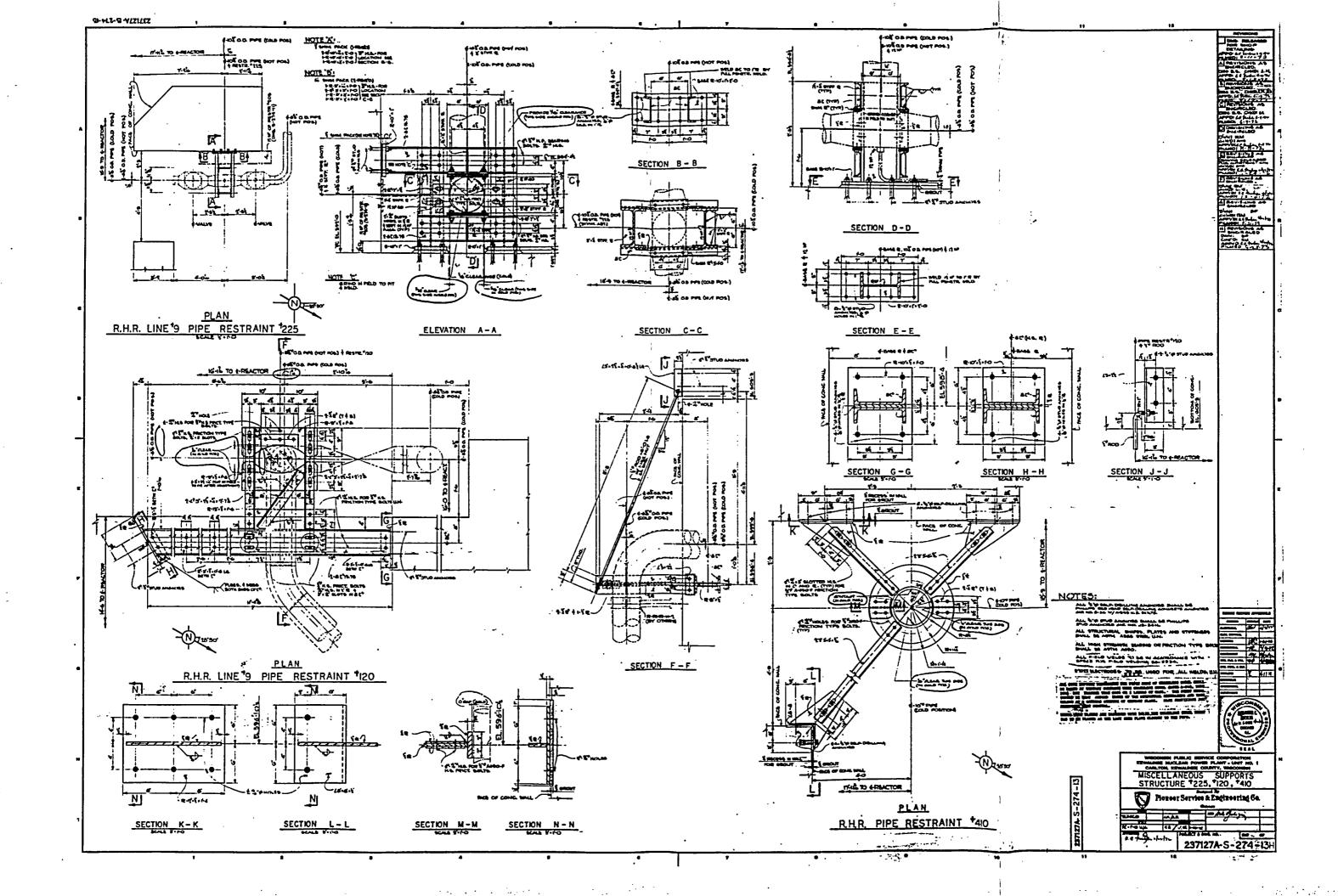
- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

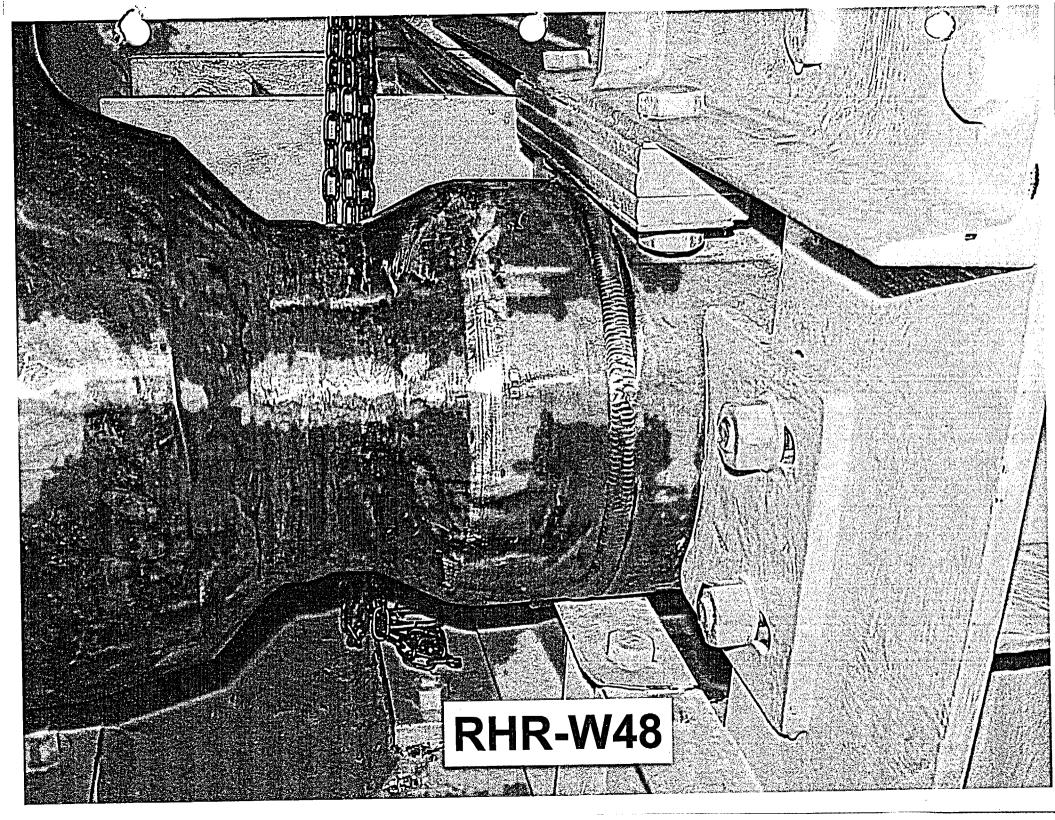
8. Precedents: Not Applicable





VISUAL EXAMINATION FIMILATION TO EXAMINA	ATTOM TILEOUTED
RHR - FROM RC LOOPS A & B HOT SYSTEM OR COMPONENT: CNTMT PEN 9 & TO CNTMT	
DRAWING NO.: <u>ISIM - 957 - ISH2</u>	
COMPONENT IDENTIFICATION: RHR - W48 PROCEDURE: NEP /	<u> 16,156</u> REVISION: <u>0RIG</u>
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICL	E: VISUAL:
EXAMINER: Onle Jense ARLEN JENSEN II DAT	E: <u>9-29-01</u>
EXAMINER: NA NA DAT	E: <u>NA</u>
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, PERCENTAGE OF REDUCED EXAMINATION COV	ERAGE.
NO EXAM FROM 1.5 CCW TO 1.5 CW AND 14.5 TO 18.5 FR	om TDC. DUE TO
BOY RESTRAINT INTERFERENCE	
REDUCED EXAM COVERAGE IS 20%.	
34" CIRCUMFERENCE 7" REDUCED COVERAGE	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes	DATE: <u>Otoler 1, 2001</u>
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Korn Millian	DATE:





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-54

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

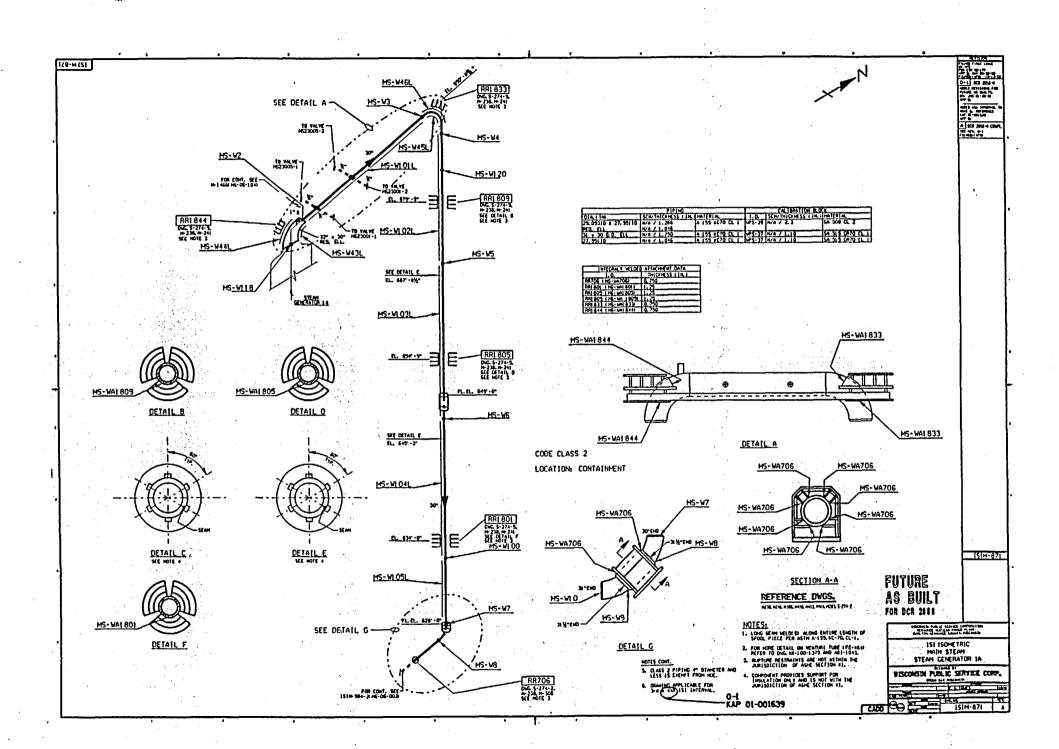
- 1. ASME Code Component Affected: 30" Main Steam Circumferential Weld MS-W3
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.51
- 4. Impracticality of Compliance: 37.0% (Calculated by Kewaunee Nuclear Power Plant) of the 30" Main Steam Circumferential Weld MS-W3 was inaccessible due to the Reducing Elbow O.D. Taper and thickness change configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 37.0% of the 30" Main Steam Circumferential Weld MS-W3 would require modification of the original design of the Main Steam Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-54

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Magnetic Particle Examination was performed on the 30" Main Steam Circumferential Weld MS-W3 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-2 and Item No. C5.51.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: MAIN STM STM GEN IA DRAWING NO.: 151M - 871
COMPONENT IDENTIFICATION: MS - W3 PROCEDURE: (XP. 913 REVISION: ORIL
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: Jodd P. Blef II DATE: 4-18-95
LEVEL
EXAMINER: <u>Irani Ikma</u> <u>I</u> DATE: <u>4-18-25</u> LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
2 5
71
FLOW
PIPE RED. ELBOW
NO SCANS 5. 7.8 DN RED. ELBOW DUE TO OD TAPER AND THICKNESS CHANGE. NO O' SCAN DUE TO TAPER.
2.62
KEWAUNEE NUCLEAR POWER PLANT REVIEW: E: A Balalad DATE: 4/19/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: 12 / 25 / 25

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-55

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

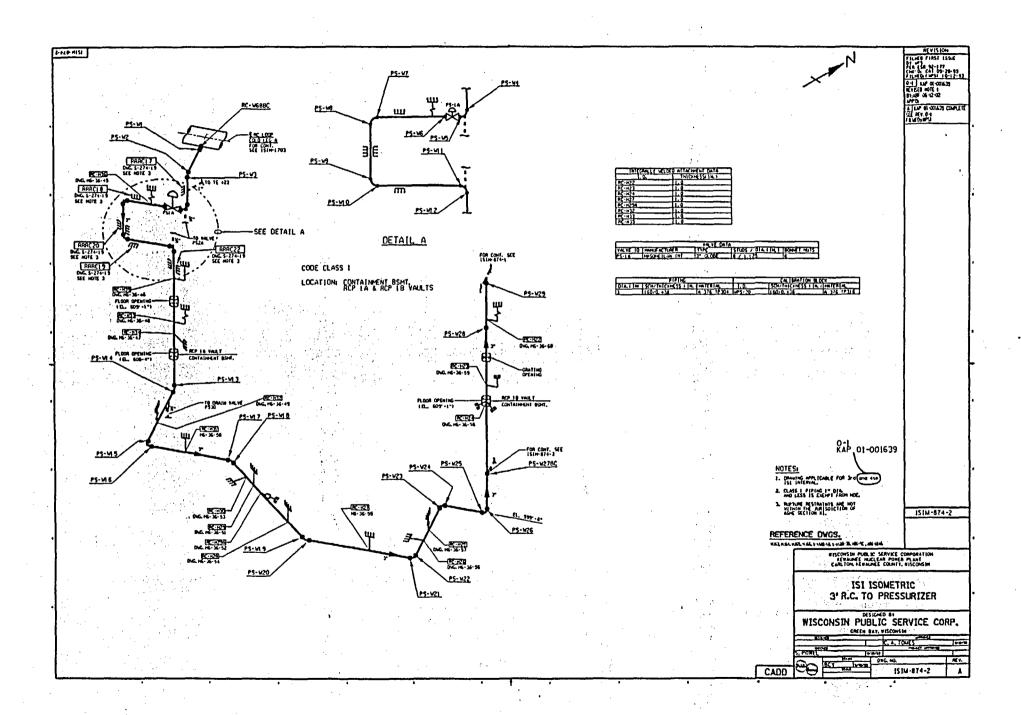
- 1. ASME Code Component Affected: 3" Pressurizer Spray Circumferential Weld PS-W3
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.21
- 4. Impracticality of Compliance: 27.0% of the 3" Pressurizer Spray Circumferential Weld PS-W3 was inaccessible due to a Branch Connection thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 27.0% of the 3" Pressurizer Spray Circumferential Weld PS-W3 would require modification of the original design of the Pressurizer Spray Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-55

- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD
SYSTEM OR COMPONENT: 3" R.C. To Pressurizer DRAWING NO.: ISIM -874-2 COMPONENT IDENTIFICATION: PS-W3 PROCEDURE: QCP-101 REVISION: ORIG
ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE: VISUAL:
EXAMINER: NUA. BY II DATE: 4-13-15
EXAMINER: NA NA DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
ps_w2
A E LA W
PS-W3
BRANCH Connection Limits P.T. Exam. of Base material For 3 inches.
View Looking Toward RCP-1A Page 1 of 2
KEWAUNEE NUCLEAR POWER PLANT REVIEW: 2 A Balila DATE: 4/15/95
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royan Profiner DATE: 4/16/95

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-56

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

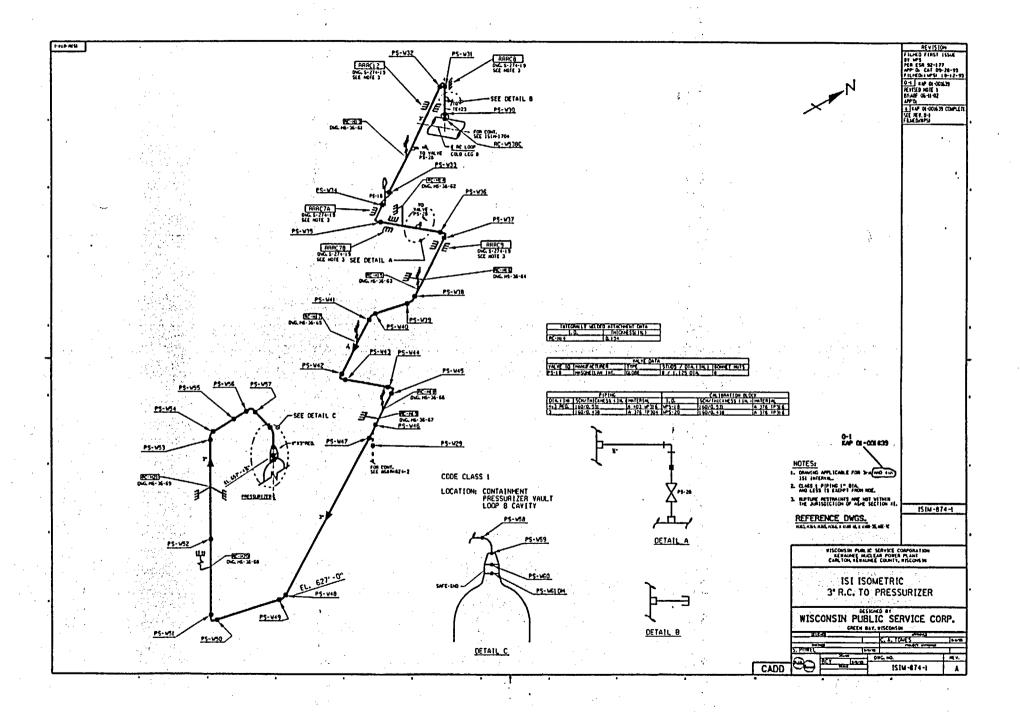
- 1. ASME Code Component Affected: 3" Pressurizer Spray Circumferential Weld PS-W34
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.21
- 4. Impracticality of Compliance: 37.0% of the 3" Pressurizer Spray Circumferential Weld PS-W34 was inaccessible due to a Whip Restraint RRRC7A thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 37.0% of the 3" Pressurizer Spray Circumferential Weld PS-W34 would require removal of the Whip Restraint RRRC7A.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

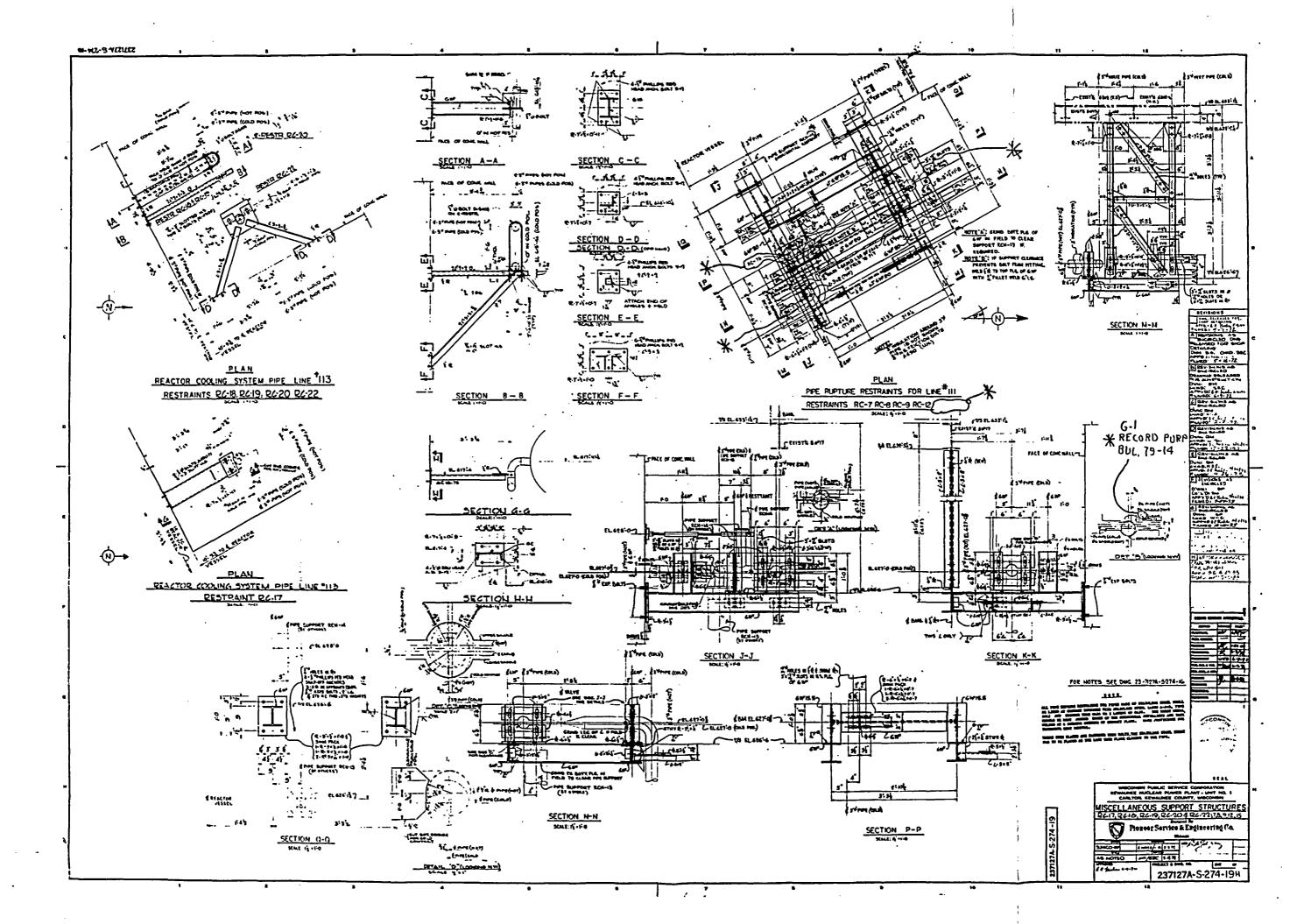
RELIEF REQUEST NO: RR-G-7-56

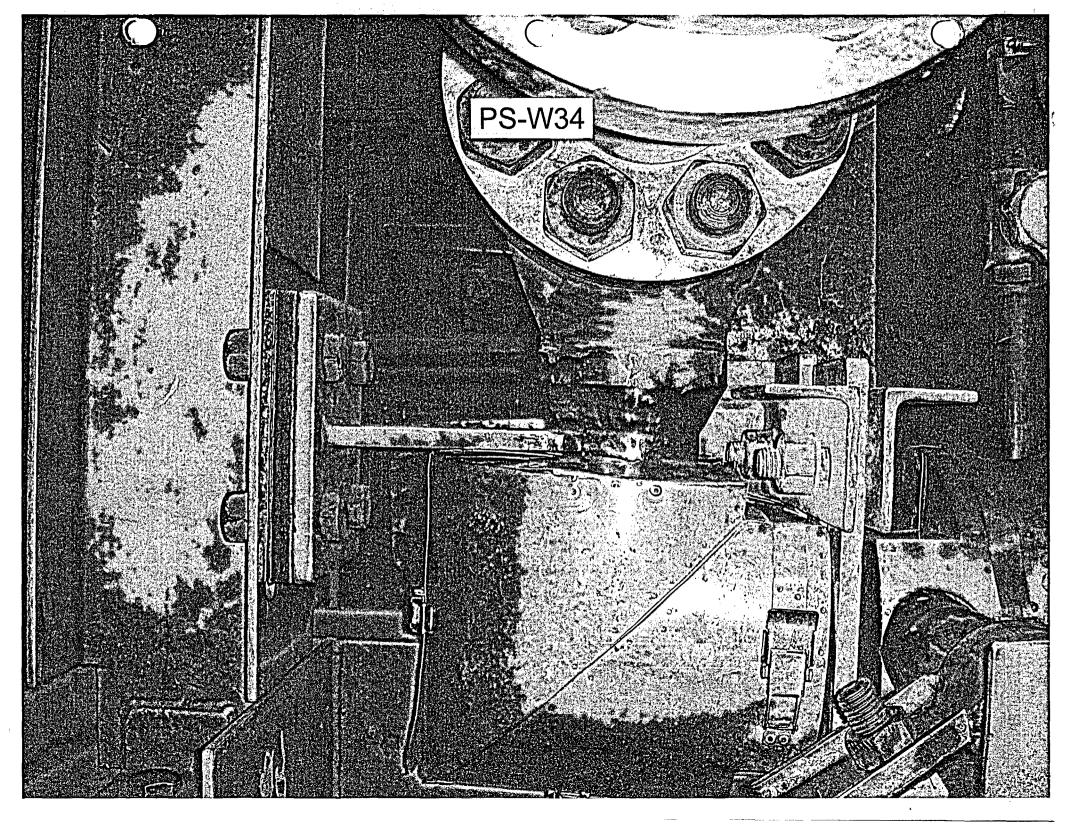
- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



TIOUTE EXAMINATION TO EXAMINATION TECOTIO
SYSTEM OR COMPONENT: 3" RC. TO Pressurizer DRAWING NO.: 151M - 874-1
COMPONENT IDENTIFICATION: PS_W34 PROCEDURE: QCP-901 REVISION: ORIG.
ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE: VISUAL:
EXAMINER: NIA. BOT II DATE: 4-14-95
EXAMINER: NUM. 1801. DATE: 4 - 14-95 LEVEL
EXAMINER: DATE: 4-14-95-
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
Top View
Weld PS- W3H
FRONT View
Whip Restraint Limits 3.5" of CIRCumference of Weld For P.T. Exam.
Percentage of Reduced Examination Coverage = 63%
KEWAUNEE NUCLEAR. S. A DIA DIA 1/1-/9-
POWER PLANT REVIEW:
INSERVICE INSPECTOR REVIEW: Kozu Ontu DATE: 4/18/95





THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-57

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

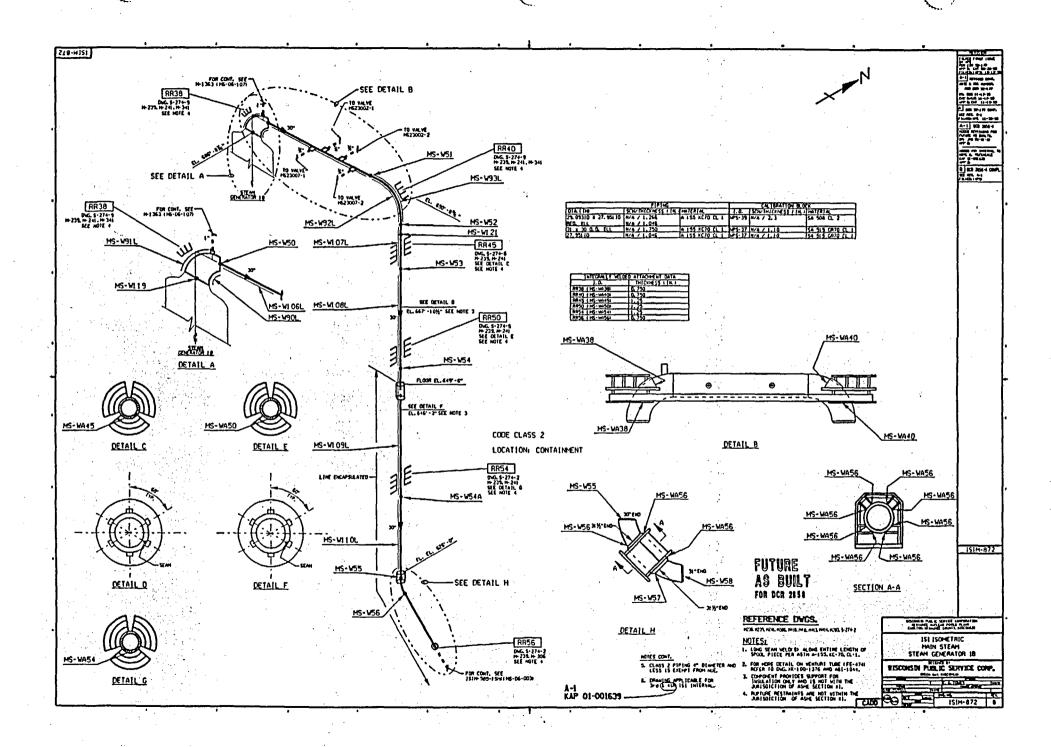
- 1. ASME Code Component Affected: 3" Pressurizer Spray Circumferential Weld PS-W10
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.21
- 4. Impracticality of Compliance: 36.0% of the 3" Pressurizer Spray Circumferential Weld PS-W10 was inaccessible due to a Hanger Support RRRC19 thus restricting Surface Liquid Penetrant Examination.
- 5. Burden Caused by Compliance: To provide for access to the 36.0% of the 3" Pressurizer Spray Circumferential Weld PS-W10 would require removal of the Hanger Support RRRC19.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

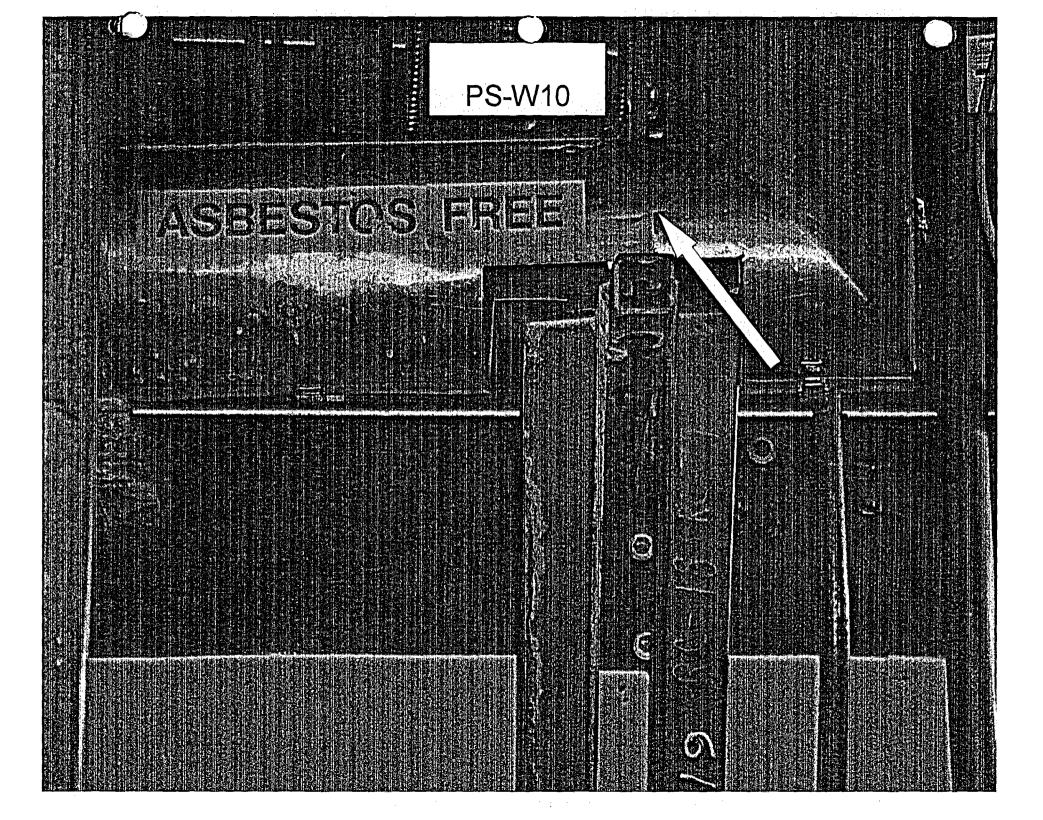
RELIEF REQUEST NO: RR-G-7-57

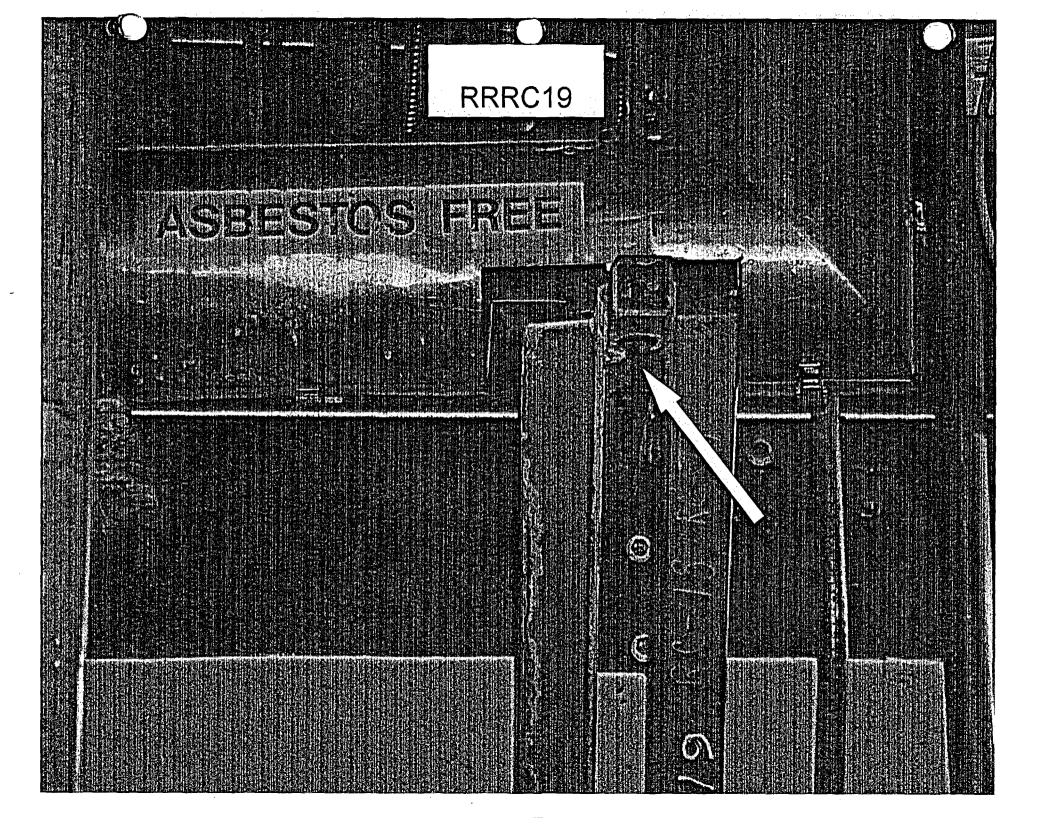
- 6. Proposed Alternative and Basis for Use: No alternative Code required Liquid Penetrant examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD
SYSTEM OR COMPONENT: 3"RC TO Pressurizer DRAWING NO.: 151M -874-2 COMPONENT IDENTIFICATION: PS_WID PROCEDURE: QCP_961 REVISION. OPTG ULTRASONIC: LIQUID PENETRANT: X MAGNETIC PARTICLE: VISUAL:
EXAMINER: NA DATE: 4-13-95 LEVEL EXAMINER: NA DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE. PS-W10 SIDE VIEW FRONT VIEW FACING REALIOR
HANGER Support Limits P.T. EXAM At BOHOM OF WELL For 4 inches. Page 2 of 2
KEWAUNEE NUCLEAR POWER PLANT REVIEW: En A. Balata DATE: 4/15/95 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Frage Water DATE: 4/16/95





KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-58

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

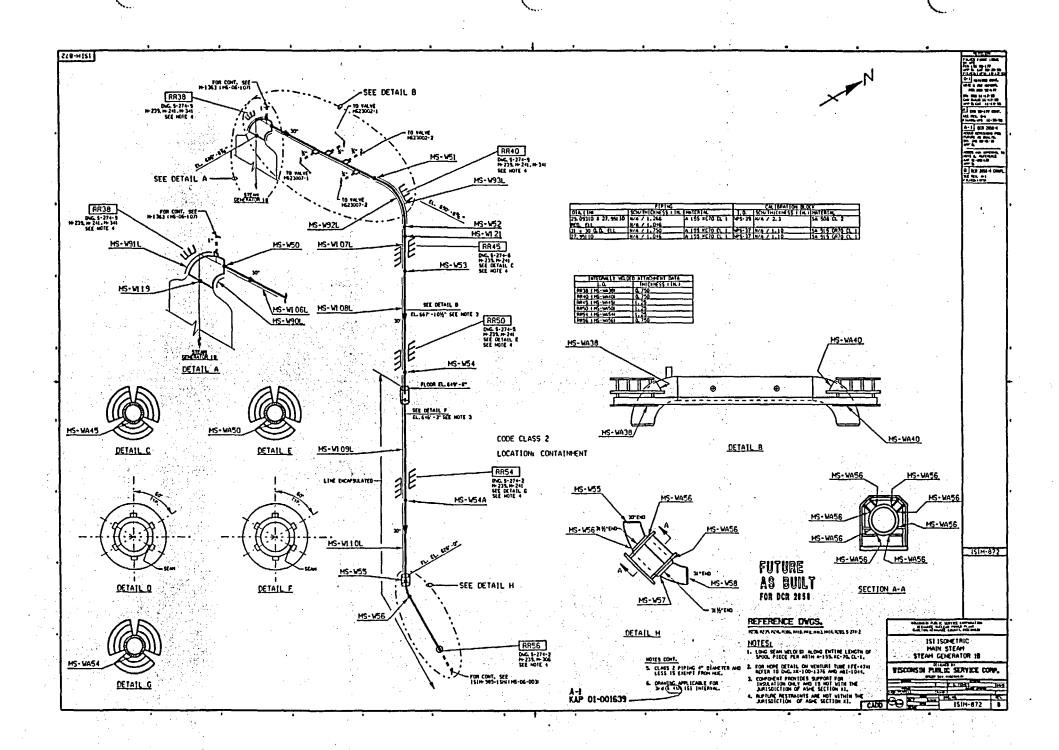
- 1. ASME Code Component Affected: 30" Main Steam Circumferential Weld MS-W50
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.51
- 4. Impracticality of Compliance: 8.0% the 30" Main Steam Circumferential Weld MS-W50 was inaccessible due to the O.D. Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 8.0% of the 30" Main Steam Circumferential Weld MS-W50 would require modification of the original design of the Main Steam Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-58

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Magnetic Particle Examination was performed on the 30" Main Steam Circumferential Weld MS-W50 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-2 and Item No. C5.51.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: MALK STEAM STEAM GENE	CRATOR IB
DRAWING NO.: ISIM- 872	
COMPONENT IDENTIFICATION: 45- 650 PROCEDURE: NE	P-15.40REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTIC	CLE: VISUAL:
	ATE: 10/12/01
EXAMINER: Mothlach II D.	ATE: <u>10/12/01</u>
LEVEL	
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION PERCENTAGE OF REDUCED EXAMINATION CO	
MS-W91L	
5 SIDE 2 SIDE	
MS-W50	
450 - NO SCAN 5,7,8 ON 5 SIDE DUE TO O.D. COI	NEIBURATION.
REDUCED CODE COVERAGE BY 8%.	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukea	DATE: October 19, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: And Mynine	DATE: 10-19-01

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-59

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

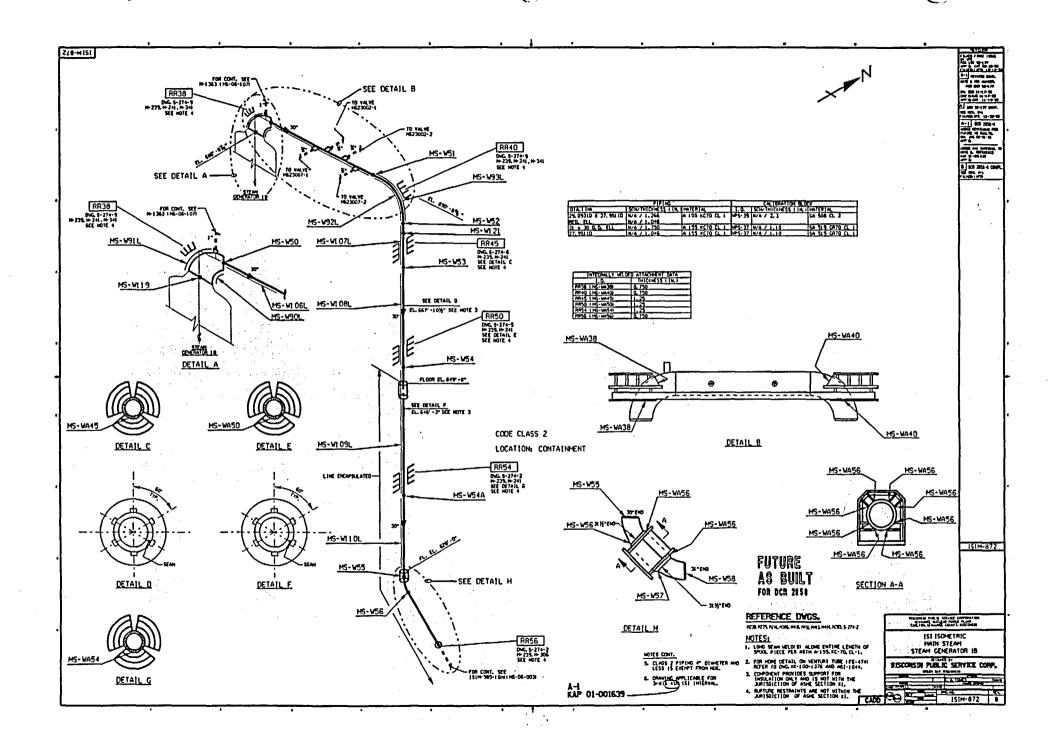
- 1. ASME Code Component Affected: 32" Main Steam Longitudinal Weld MS-W90L
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.52
- 4. Impracticality of Compliance: 22.0% the 32" Main Steam Longitudinal Weld MS-W90L was inaccessible due to the 1" Elbow Taper Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 22.0% of the 32" Main Steam Longitudinal Weld MS-W90L would require modification of the original design of the Main Steam Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

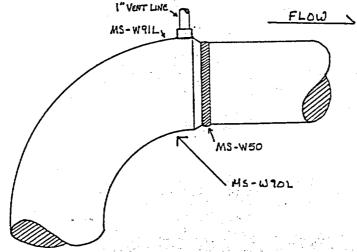
RELIEF REQUEST NO: RR-G-7-59

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Magnetic Particle Examination was performed on the 32" Main Steam Longitudinal Weld MS-W90L during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination C-F-2 and Item No. C5.52.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: MAIN STEMM	STEAM LENERATOR IB
DRAWING NO .: ISIM-872	
COMPONENT IDENTIFICATION: M5- W91L	PROCEDURE: NEA-15.40 REVISION: A
ULTRASONIC: X LIQUID PENETRANT:	
EXAMINER: TIM COBURN MM-	DATE: 10/12/01 LEVEL
	the contract of the contract o
EXAMINER: Montage	DATE: 10/12/01 LEVEL
	CATION, ORIENTATION, TYPE OF LIMITATION AND



450 - LIMITED 2,5,7,8 SCAN ON WELD M5-W9OL DUE TO 1" ELBOW TAPER. SCANS 2,5,7,8 LIMITED BY 3" ON WELD MS-WAIL DUE TO I" VENT LINE.

WELD MS-W9DL REDUCED CODE COVERAGE BY 22%. WELD MS - WAIL REDUCED CODE COVERAGE BY 63%.

KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukeo	DATE: October 18, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royan Morani	DATE: /0 -/9-0/

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-60

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

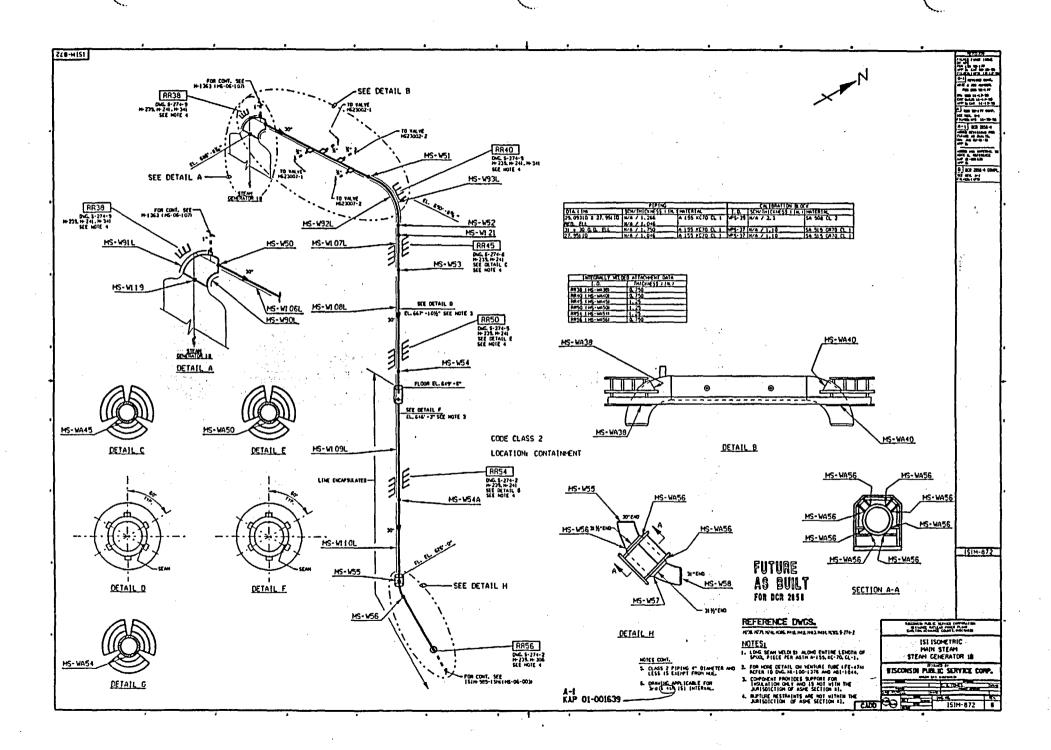
- 1. ASME Code Component Affected: 32" Main Steam Longitudinal Weld MS-W91L
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.52
- 4. Impracticality of Compliance: 63.0% the 32" Main Steam Longitudinal Weld MS-W91L was inaccessible due to a 1" Vent Line thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 63.0% of the 32" Main Steam Longitudinal Weld MS-W91L would require modification of the original design of the Main Steam Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-60

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: MAIN STEAM GENERATOR IS
DRAWING NO.: ISIM-872
พร – พจดน COMPONENT IDENTIFICATION: พร – พจนน PROCEDURE: <u>NEP - เรง40</u> REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TIM COBURN MM- DATE: 10/12/01
EXAMINER: Malyula TE DATE: 10/12/01 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
AS-WAIL
MS-W50
MS-W7DL
450 - LIMITED 2,5,7,8 SCAN ON WELD MS-WOOL DUE TO 1" ELBOW TAPER.
SCRNS 2,5,7,8 LIMITED BY 3" ON WELD MS-W91L DUE TO 1" VENT LINE.
WELD MS-WADL REDUCED CODE COVERAGE BY 22%. WELD MS-WAIL REDUCED CODE COVERAGE BY 63%.
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C. Bukes DATE: October 18, 2001
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Form DATE: 10-19-01

KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-61

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

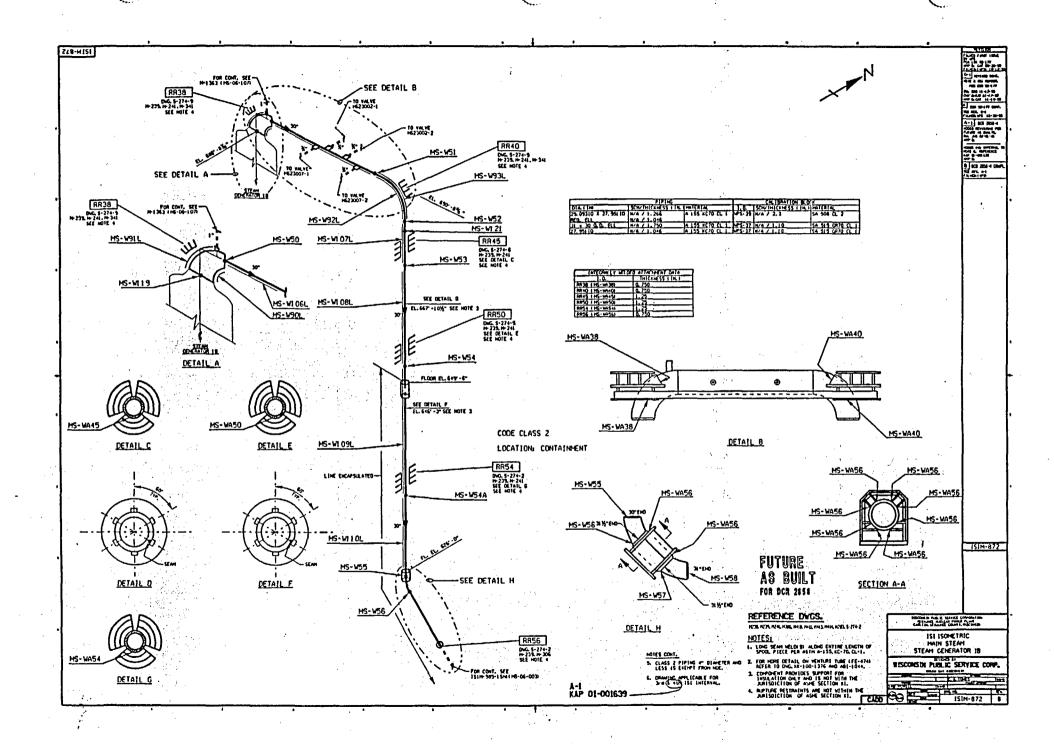
- 1. ASME Code Component Affected: 32" Main Steam Longitudinal Weld MS-W91L
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.52
- 4. Impracticality of Compliance: 50.0% the 32" Main Steam Longitudinal Weld MS-W91L was inaccessible due to a 1" Vent Line thus restricting Surface Magnetic Particle Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 32" Main Steam Longitudinal Weld MS-W91L would require modification of the original design of the Main Steam Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-61

- 6. Proposed Alternative and Basis for Use: No alternative Code required Surface Magnetic Particle Examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: MAIN STEAM STEAM GENERATOR 1B	
DRAWING NO.: <u>ISIM - 872</u>	
COMPONENT IDENTIFICATION: MS-W91L PROCEDURE: NEPNO.15.7 REVISION: ORIG	
ULTRASONIC: LIQUID PENETRANT: MAGNETIC PARTICLE:X VISUAL:	
EXAMINER: Print II DATE: 10/11/01 LEVEL TO DATE: 10/11/01	٠.
EXAMINER: DATE. TOTAL	
LEVEL	
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.	
MS-W91L LINE LIMITED FROM 1" +0 4" 3	
MS-W50	
	-
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukes DATE: October 13, 2001	
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Any Trypus: DATE: 10-15-01	

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-62

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

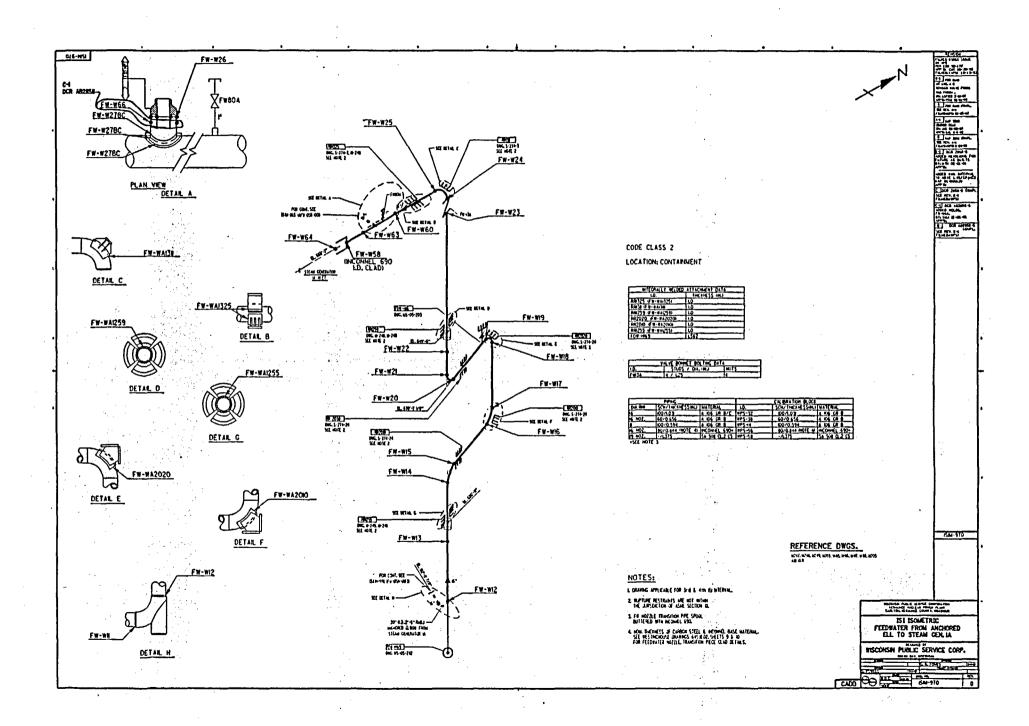
- 1. ASME Code Component Affected: 16" Feedwater Circumferential Weld FW-W24
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.51
- 4. Impracticality of Compliance: 8.0% the 16" Feedwater Circumferential Weld FW-W24 was inaccessible due to the Valve Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 8.0% of the 16" Feedwater Circumferential Weld FW-W24 would require modification of the original design of the Feedwater Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-62

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Magnetic Particle Examination was performed on 16" Feedwater Circumferential Weld FW-W24 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-2 and Item No. C5.51.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: FEEDWATER FROM ANCHORED ELL TO STEAM GEN. IA
DRAWING NO.: 1514 - 970
COMPONENT IDENTIFICATION: FW - W 24 PROCEDURE: NEP-15. 40 REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TIM CORIJEN MINISTER DATE: 10/20/01
EXAMINER: <u>NA</u> DATE: <u>NA</u> LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
YALVE FW-13A UNLUE FW-13A US° - SCAN 5,7,8 RESTRICTED DUE TO VALUE CONFIGURATION. 60° - SCAN 5 RESTRICTED DUE TO VALUE CONFIGURATION.
REDUCED CODE COVERAGE BY 8%.
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Long Wigner DATE: 10-29-01

KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-63

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

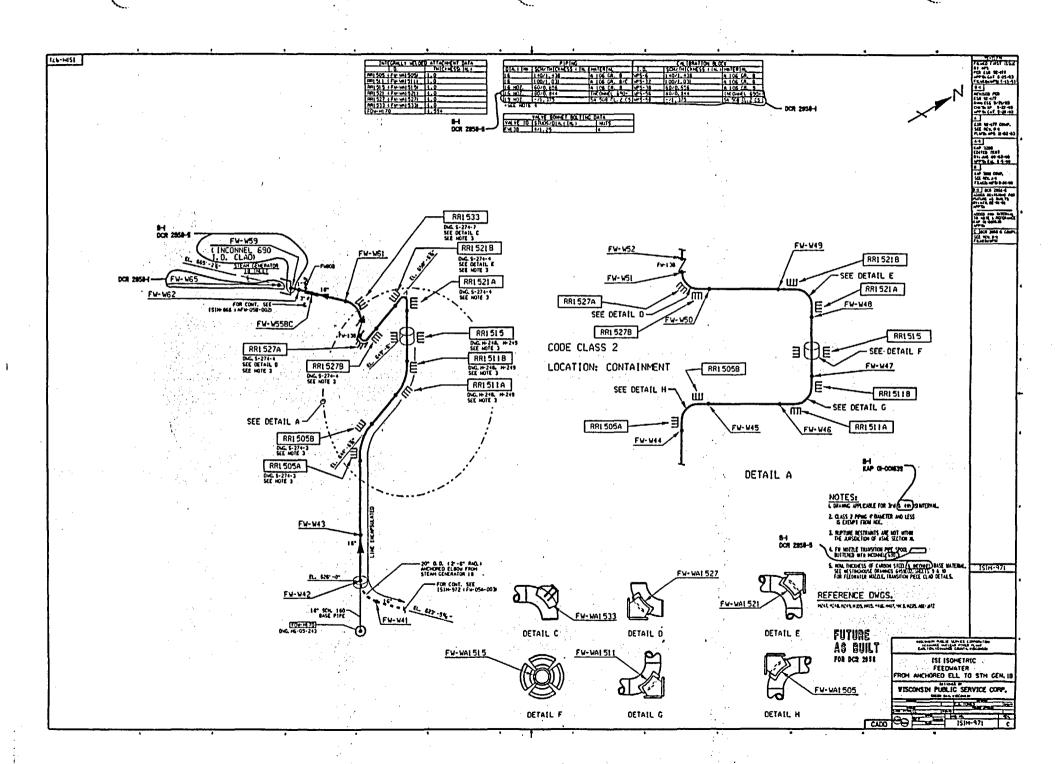
- 1. ASME Code Component Affected: 16" Feedwater Circumferential Weld FW-W52
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-F-2; Item No. C5.51
- 4. Impracticality of Compliance: 8.0% the 16" Feedwater Circumferential Weld FW-W52 was inaccessible due to the Valve Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 8.0% of the 16" Feedwater Circumferential Weld FW-W52 would require modification of the original design of the Feedwater Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-63

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.30 each 3 1/3 Year Period for evidence of leakage. Surface Magnetic Particle Examination was performed on 16" Feedwater Circumferential Weld FW-W52 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-F-2 and Item No. C5.51.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



SYSTEM OR COMPONENT: FEEDWATER FROM ANCHORED ELL TO STM GEN. 18
DRAWING NO.: ISIM - 971
COMPONENT IDENTIFICATION: FW-WSZ PROCEDURE: NEP-15.42REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: TIM COBURN WINT I DATE: 10/20/01
EXAMINER: NA DATE: NA LEVEL
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
FW-WS2.
VALVE FW-13B
45° - SCAN 5,7,8 RESTRICTED DUE TO VALUE CONFIBURATION. 60° - SCAN 5 RESTRICTED DUE TO VALUE CONFIBURATION.
REDUCED CODE COVERAGE BY 8%.
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip C. Bukas DATE: October 7, 2001
NEWAUNEE NUCLEAR POWER PLANT REVIEW: Philip C. Bukas DATE: Octobriz, 2001 AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: From Date: 10/29/81

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-64

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Regenerative Heat Exchanger

 Tubesheet to Shell Circumferential Weld

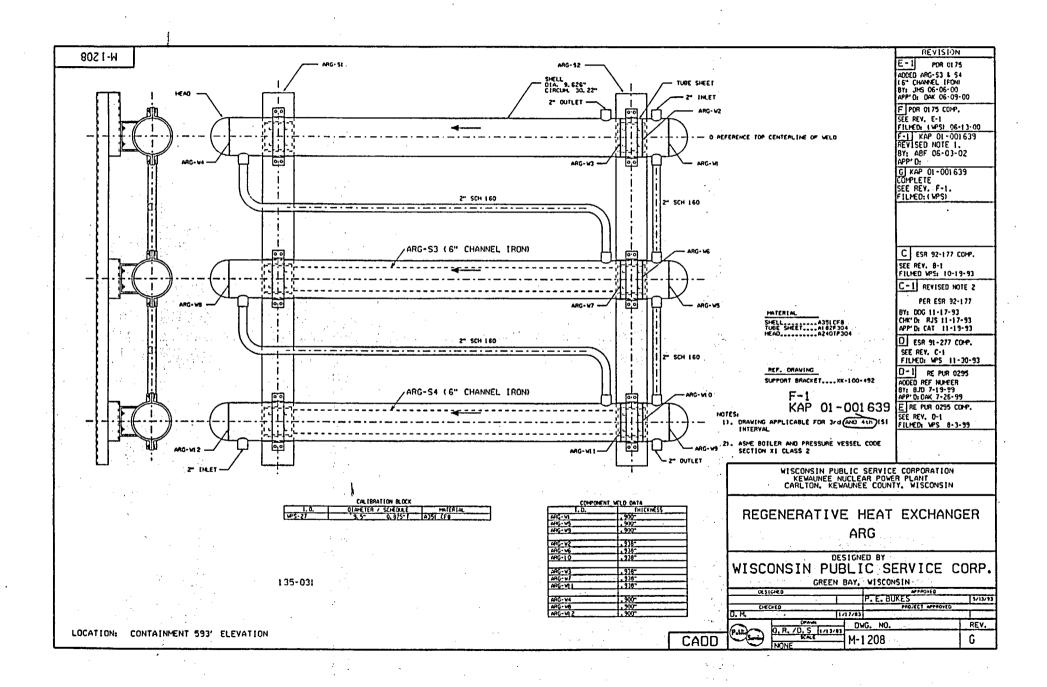
 ARG-W11
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWC-2500-1; Examination Category C-A; Item No. C1.30
- 4. Impracticality of Compliance: 41.4% of the Regenerative Heat Exchanger Tubesheet to Shell Circumferential Weld ARG-W11 was inaccessible due to Pipe Clamp Connected to Rigid Support Bracket Assembly thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 41.4% of the Tubesheet to Shell Circumferential Weld ARG-W11 would require removal and modification of the Rigid Support Bracket Assembly in a 1R 2R per hour Radiation Field of the Regenerative Heat Exchanger.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

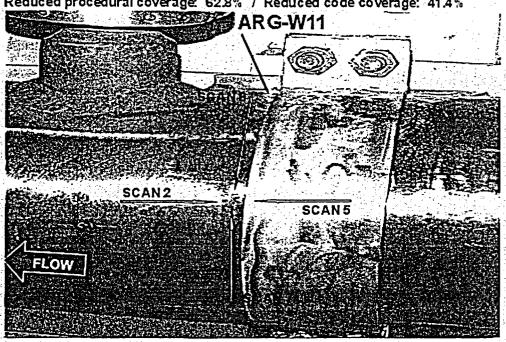
RELIEF REQUEST NO: RR-G-7-64

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access caused by the Rigid Support Bracket and the High Radiation Fields. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWC-2500-1; Examination Category C-H; Item C7.10 each 3 1/3 Year Period for evidence of leakage.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



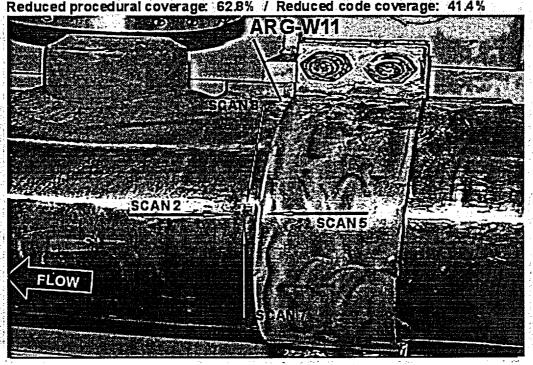
VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD
SYSTEM OR COMPONENT: REGENERATIVE HEAT EXCHANGER ARG
DRAWING NO.:
COMPONENT IDENTIFICATION: ARG-WII PROCEDURE: NEPNO. 15.17 REVISION: ORIG
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: J.T.mm III DATE: 4-25-03 LEVEL
EXAMINER: DATE: DATE:
LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to pipe clamp connected to rigid support bracket assembly. Scan 2, 7 & 8, also limited to 90% of length due to welded lugs around circumference of vessel. Reduced procedural coverage: 62.8% / Reduced code coverage: 41.4% ARG-W11



KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillip & Bukes	DATE: <u>April 25,2003</u>
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Logue Magnen	DATE: 4-25-03

No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to pipe clamp connected to rigid support bracket assembly. Scan 2, 7 & 8, also limited to 90% of length due to welded lugs around circumference of vessel.

Reduced procedural coverage: 62.8% / Reduced code coverage: 41.4%



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-65

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

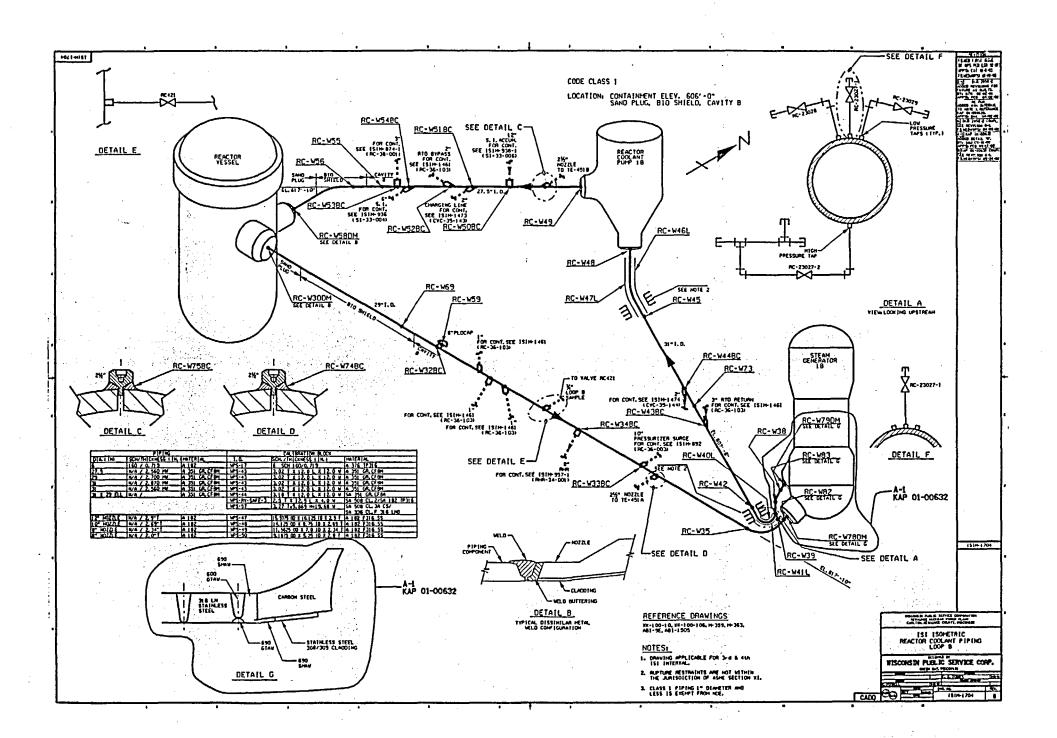
- 1. ASME Code Component Affected: 12" Reactor Coolant Pipe Branch Connection Weld RC-W50BC
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.31
- 4. Impracticality of Compliance: 53.0% of Reactor Coolant Pipe 12" Branch Connection Weld RC-W50BC was inaccessible for Ultrasonic Examination due to the Branch Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Ultrasonic Examination for Reactor Coolant Pipe 12"Branch Connection Weld RC-W50BC would require modification of the original design of the Reactor Coolant Pipe.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-65

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic Examination is available for Reactor Coolant Pipe 12" Branch Connection RC-W50BC due to the Branch Nozzle Configuration. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item No. B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Reactor Coolant Pipe 12" Branch Connection Weld RC-W50BC during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.31.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable



REV.: ORIG.

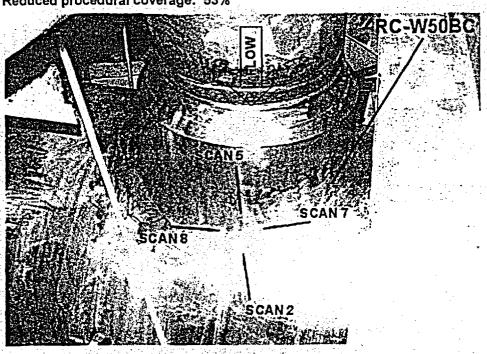
WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: REACTOR COOCHIT PIPING LOOP B
DRAWING NO.:ISIM -1704
COMPONENT IDENTIFICATION: RC-WSOBC PROCEDURE: NEP No. 15.39 REVISION: Orig
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: MM-M II DATE: 4/17/03 LEVEL
EXAMINER: A.W. JENSEN all I DATE: 4-17-03 LEVEL

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.

> No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to branch connection configuration.

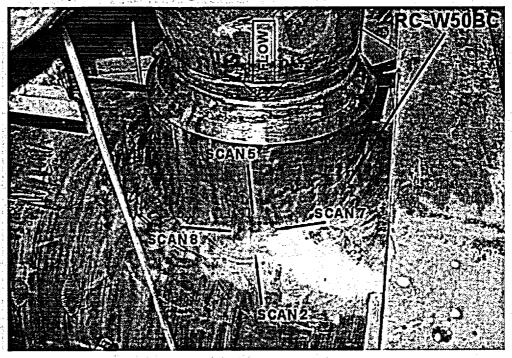
Reduced procedural coverage: 53%



KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phille	p.C. Bukes	DATE: april 22,2003
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW:	Loga Moguin	DATE: <u>Y-22-03</u>

No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to branch connection configuration.

Reduced procedural coverage: 53%



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-66

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 12" Safety Injection Circumferential Weld SI-W75
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 12" Safety Injection Circumferential Weld SI-W75 was inaccessible due to the Valve To Elbow Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 12" Safety Injection Circumferential Weld SI-W75 would require modification of the original design of Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

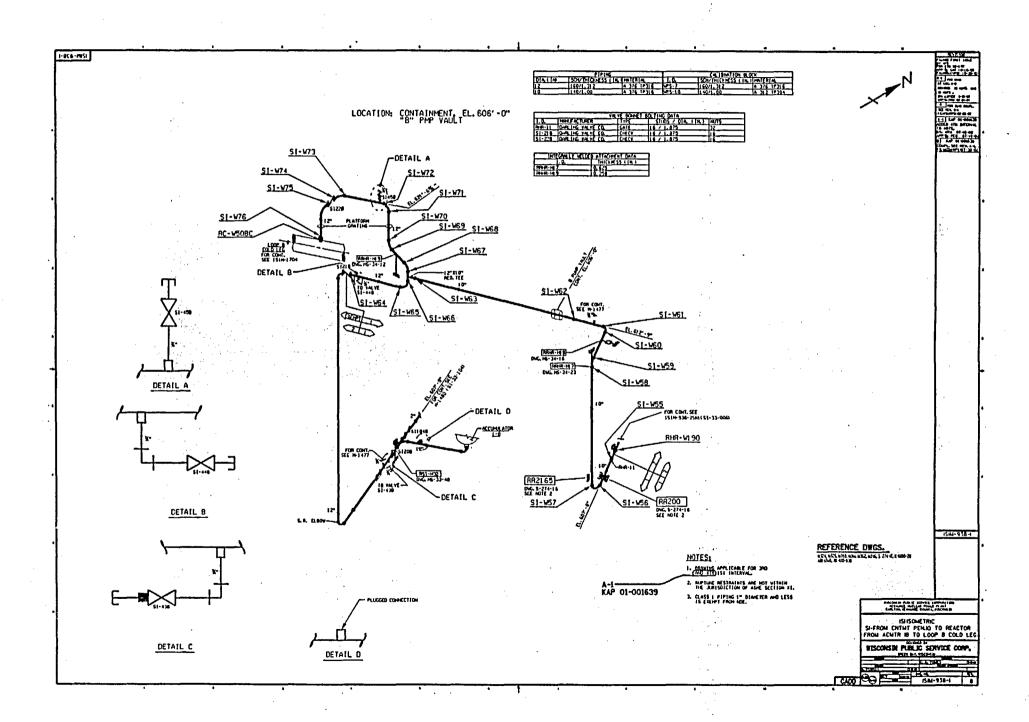
RELIEF REQUEST NO: RR-G-7-66

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 12" Safety Injection Circumferential Weld SI-W75 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable

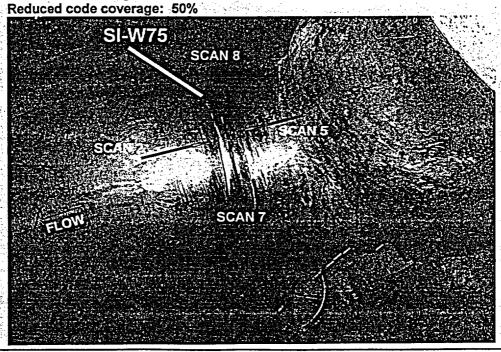


WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: SI- FROM ACMTR 1B TO	TO REACTOR	-D LEG
DRAWING NO.: 151M - 938-1		
		NEP-15.41 REVISION: A
ULTRASONIC: X LIQUID PENETRANT:	MAGNETIC PAR	RTICLE: VISUAL:
EXAMINER: A.W. JENSEN alg	LEVEL	DATE: 4/18/03 DATE: 4-18-03
0	LEVEL	

SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.

No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to valve to elbow configuration.



KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phil	Ples C. Bukes	DATE: <u>april 22,2003</u>
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: _	Roger Materia	DATE: <u>4-22-03</u>

No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to valve to elbow configuration.

Reduced code coverage: 50%

SI-W75
SCANS
SCANS
SCANS
SCANS

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-67

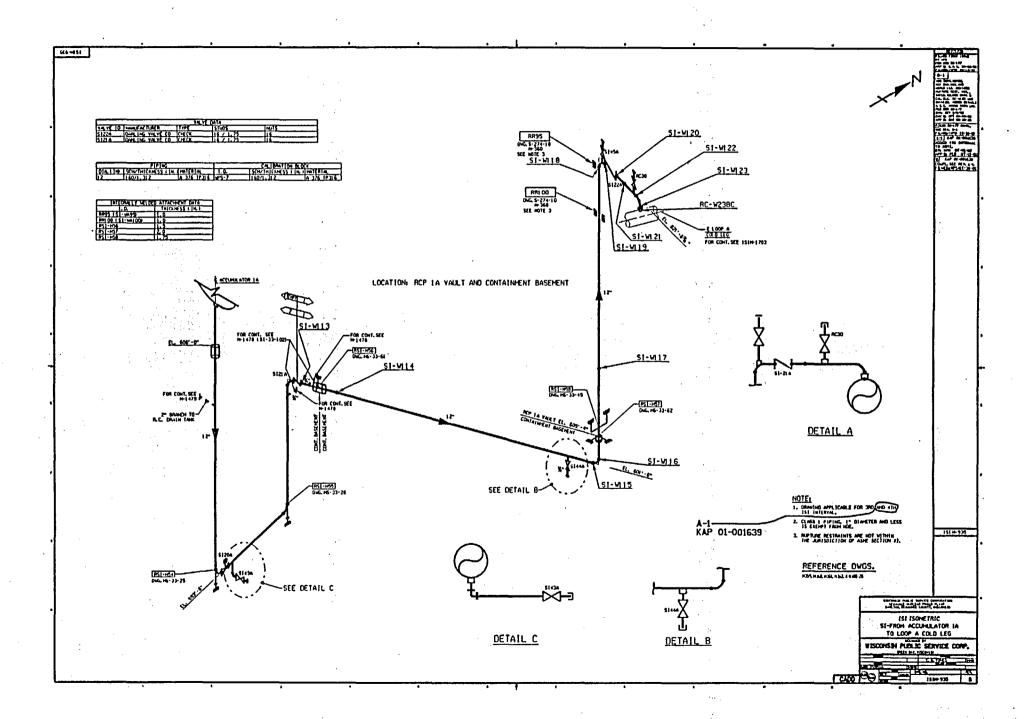
PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 12" Safety Injection Circumferential Weld SI-W119
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 12" Safety Injection Circumferential Weld SI-W119 was inaccessible due to the Elbow to Valve Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 12" Safety Injection Circumferential Weld SI-W119 would require modification of the original design of Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-67

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 12" Safety Injection Circumferential Weld SI-W119 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004
- 8. Precedents: Not Applicable
- 9. References: Not Applicable

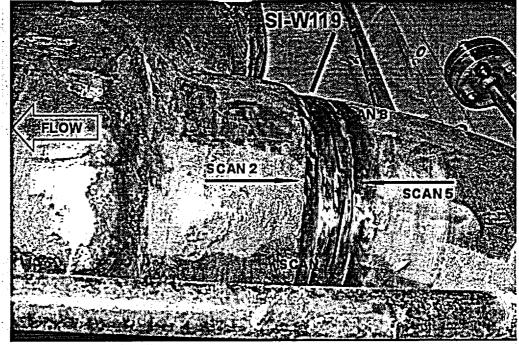


WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: SI - FROM ACCUMULATOR 14 TO LOO	P "A" colo leb
DRAWING NO.: 151M-935	
COMPONENT IDENTIFICATION: SI- WILS PROCEDURE:	NEP -15.41 REVISION: A
ULTRASONIC: Y LIQUID PENETRANT: MAGNETIC PAI	RTICLE: VISUAL:
EXAMINER: J.L. Devers JO LEVEL	DATE: 4-21-03
EXAMINER:LEVEL	DATE:
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTAT PERCENTAGE OF REDUCED EXAMINATION No scan 2, scan 7 & 8 limited to weld and upstream base me	I COVERAGE.
elbow to valve configuration. Reduced code coverage: 50% SI-W119	
SCAN 2	SCAN5
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillips C Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Royn Miguin	DATE: <u>April 22, 2003</u>
AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Rogan Maguin	DATE: 4-22-03

No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to elbow to valve configuration.

Reduced code coverage: 50%



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-68

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Safety Injection Circumferential Weld SI-W47
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 6" Safety Injection Circumferential Weld SI-W47 was inaccessible due to the Pipe to Valve Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 6" Safety Injection Circumferential Weld SI-W47 would require modification of the original design of Safety Injection Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

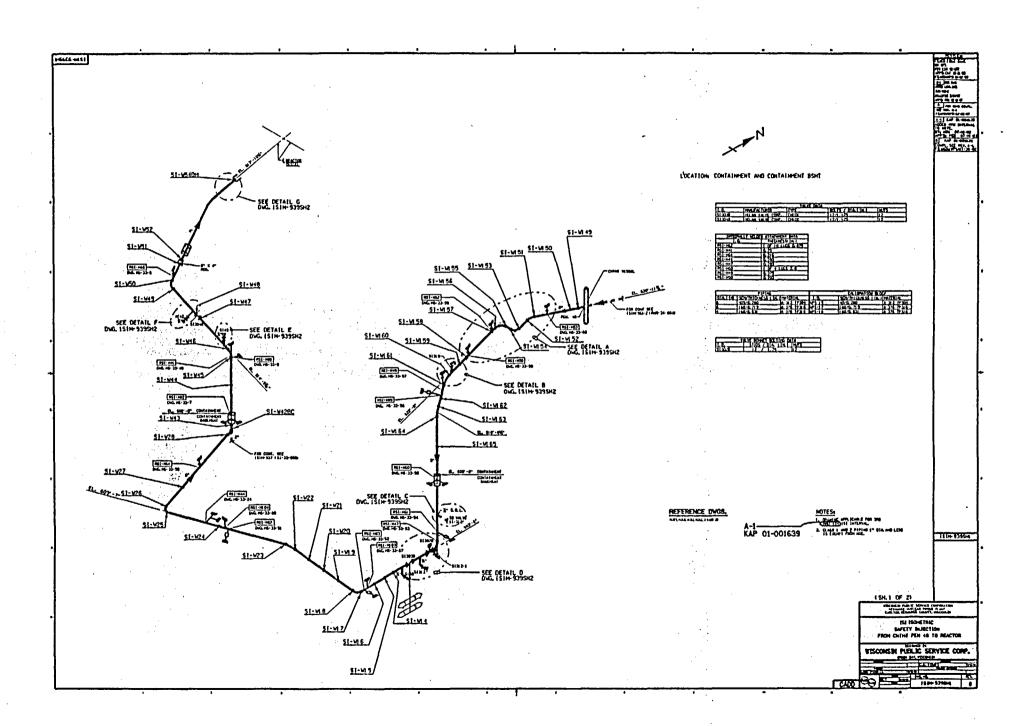
RELIEF REQUEST NO: RR-G-7-68

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Safety Injection Circumferential Weld SI-W47 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable

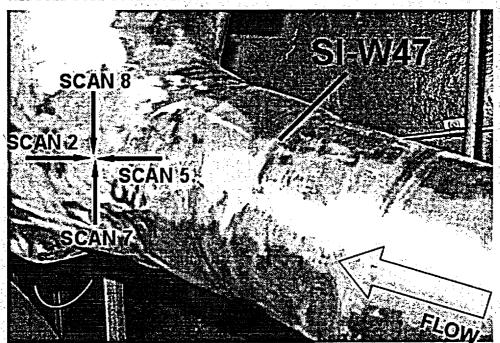
9. References: Not Applicable



WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: SAFETY INJECTION FROM CNIMI PEN. 48 TO REACTOR
DRAWING NO.: 151M-9395H1
COMPONENT IDENTIFICATION: <u>SI - ω 47</u> PROCEDURE: <u>NEP - 15-41</u> REVISION: <u>A</u>
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: JAM- JE DATE: 4/21/03 LEVEL
EXAMINER: NA NA DATE: NA LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to pipe to valve configuration REDUCED CODE COVERAGE: 50% SCAN 8 SCAN 2 SCAN 5
SCAN-7A FLOW
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Phillys C. Bukes AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW: Roger Magnetic Date: 4-22-03

No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to pipe to valve configuration REDUCED CODE COVERAGE: 50%



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-69

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 6" Reactor Coolant Circumferential Weld RC-W29
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 50.0% of the 6" Reactor Coolant Circumferential Weld RC-W29 was inaccessible due to the Pipe to Nozzle Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 50.0% of the 6" Reactor Coolant Circumferential Weld RC-W29 would require modification of the original design of Reactor Coolant Piping.

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

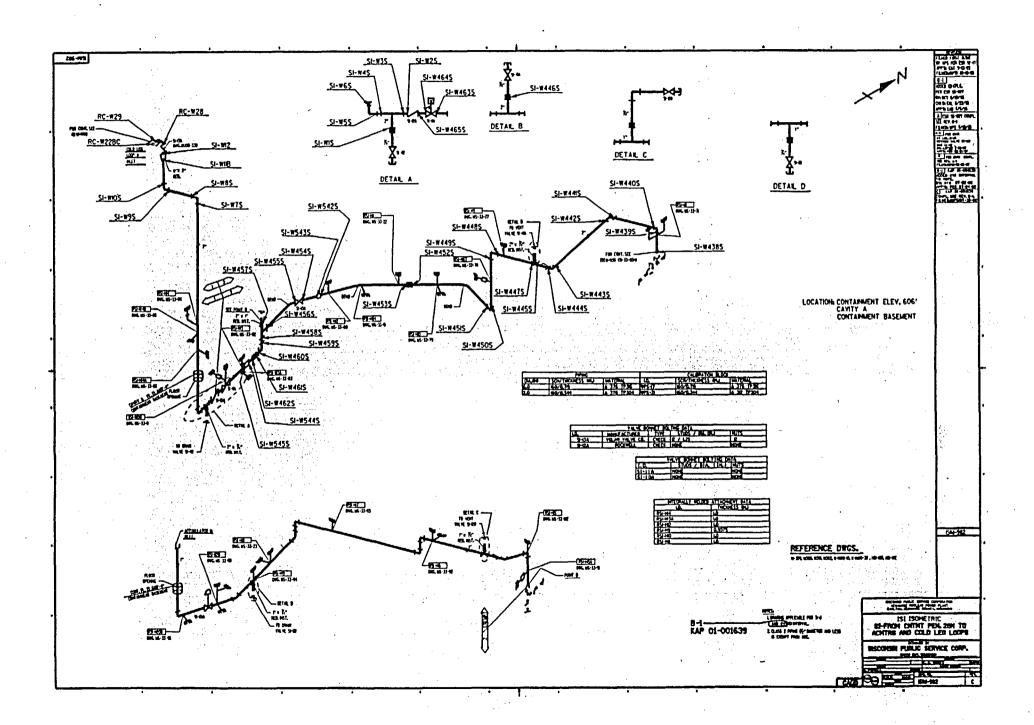
RELIEF REQUEST NO: RR-G-7-69

6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on 6" Reactor Coolant Circumferential Weld RC-W29 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.

7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 – June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable

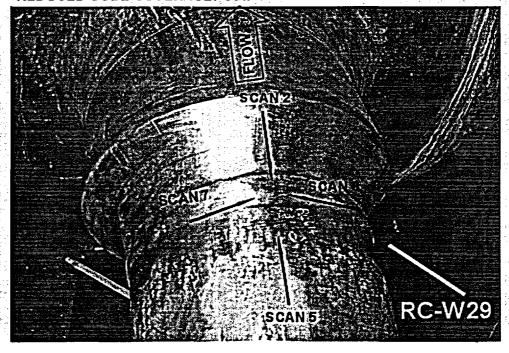


WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: 51 -	FROM CNTMT PEN. 28N TO	ACMTRS AND C	OLD LEGT LOOPS
DRAWING NO.: 151M - 982			
COMPONENT IDENTIFICATION: _	<u>RC - ω29</u> PROCEDUR	IE: <u>NEP-15.41</u>	REVISION: A
ULTRASONIC: X LIQUID PE	ENETRANT: MAGNETIC	PARTICLE:	VISUAL:
0 1/-			
EXAMINER: ///M-//	ル LEVEL	DATE:	4/21/03
EXAMINER: NA		•	
EXAMINER: NA	NA LEVEL	DATE:	
OVETOLI TO DECUME A DECOM	MATE OLŽE I SOSTISMI SOSTISMI	TATION TYPE O	C I INJUTATION AND
SKETCH TO PROVIDE: APPROXI PERCEN	IMATE SIZE, LOCATION, ORIEN TAGE OF REDUCED EXAMINAT		
No scan 2, scan 7 & 8 pipe to nozzle configur REDUCED CODE CO		metal only, due to	
KEWAUNEE NUCLEAR POWER PLANT REVIEW: Philadelia AUTHORIZED NUCLEAR INSERVICE INSPECTOR REVIEW	SCANS SCANS	RC-W2	9 E: <u>4-22-03</u>
INSERVICE INSPECTOR REVIEW	: Koga Mohum	DAT	E: 4-22-03

No scan 2, scan 7 & 8 limited to weld and upstream base metal only, due to pipe to valve configuration

REDUCED CODE COVERAGE: 50%



THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-70

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: 27.5" I.D. Reactor Coolant Pipe Circumferential Weld RC-W19
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-J; Item No. B9.11
- 4. Impracticality of Compliance: 54.0% of Reactor Coolant Pipe 27.5" I.D. Circumferential Weld RC-W19 was inaccessible for Ultrasonic Examination due to the Reactor Coolant Pump To Pipe Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to perform Ultrasonic Examination for Reactor Coolant Pipe 27.5" I.D. Circumferential Weld RC-W19 would require modification of the original design of the Reactor Coolant Pump and Reactor Coolant Pipe.

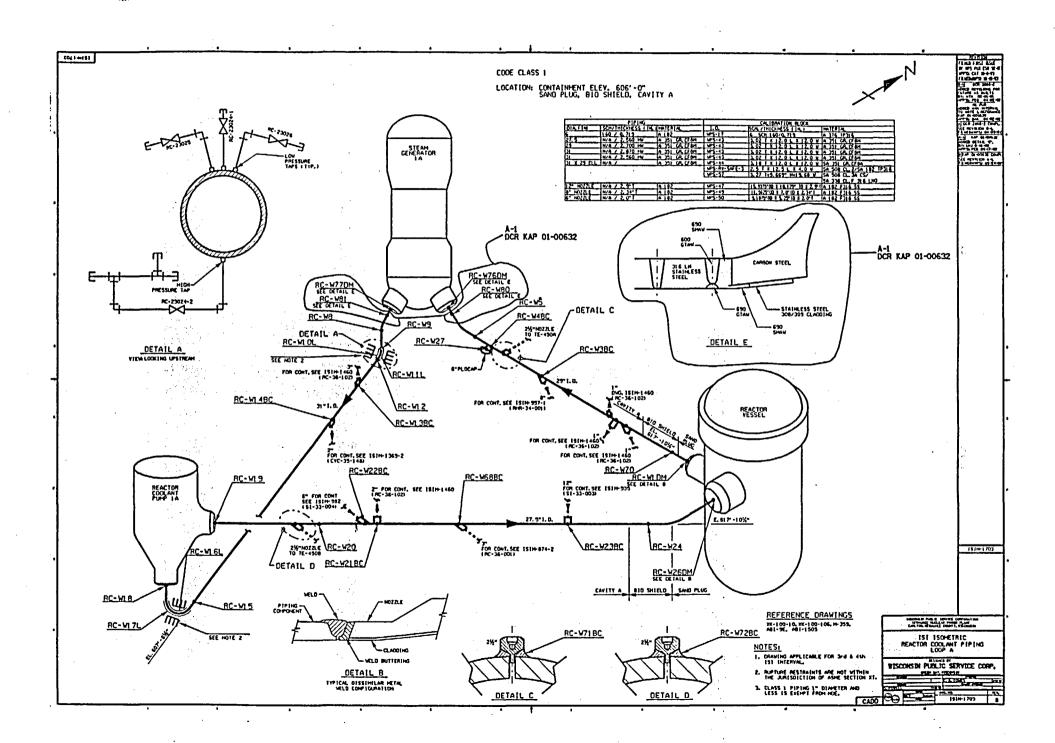
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-70

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic Examination is available for Reactor Coolant Pipe 27.5" I.D. Circumferential Weld RC-W19 due to the Reactor Coolant Pump To Reactor Coolant Pipe Configuration. VT-2 Examinations were performed during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-P; Item No. B15.50 each Refueling Outage for evidence of leakage. Surface Liquid Penetrant Examination was performed on Reactor Coolant Pipe 27.5" I.D. Circumferential Weld RC-W19 during the 3rd Interval as required by ASME Boiler and Pressure Vessel Code Section XI 1989 Edition: Table IWB-2500-1; Examination Category B-J and Item No. B9.11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable



WISCONSIN PUBLIC SERVICE CORPORATION KEWAUNEE NUCLEAR POWER PLANT ULTRASONIC, LIQUID PENETRANT, MAGNETIC PARTICLE AND VISUAL EXAMINATION LIMITATION TO EXAMINATION RECORD

SYSTEM OR COMPONENT: BEACTOR COOLANT PIPING LOOP A
DRAWING NO.: 151M - 1703
COMPONENT IDENTIFICATION: <u>PC-W19</u> PROCEDURE: <u>NEP NO. 15.13</u> REVISION: A
ULTRASONIC: X LIQUID PENETRANT: MAGNETIC PARTICLE: VISUAL:
EXAMINER: A.W. JENSEN Oul II DATE: 4-21-03 LEVEL EXAMINER: II DATE: 4/21/03 LEVEL
SKETCH TO PROVIDE: APPROXIMATE SIZE, LOCATION, ORIENTATION, TYPE OF LIMITATION AND PERCENTAGE OF REDUCED EXAMINATION COVERAGE.
No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to pump to pipe configuration.
Reduced procedural coverage: 54%
FLOW GAN 7
RC-W19

KEWAUNEE NUCLEAR
POWER PLANT REVIEW: Phillips C. Bukes

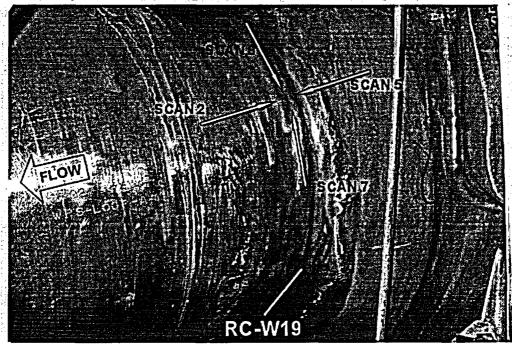
AUTHORIZED NUCLEAR

AUTHORIZED NUCLEAR

Regar Magnen

No scan 5, scan 7 & 8 limited to weld and downstream base metal only, due to pump to pipe configuration.

Reduced procedural coverage: 54%







THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-71

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Reactor Vessel Shell To Flange Weld RV-W1
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-A; Item No. B1.30
- 4. Impracticality of Compliance: 25.56% (Single Side Coverage) and 46.98% (Two Direction Coverage) of the Remote Ultrasonic Examination of the Reactor Vessel Shell To Flange Weld RV-W1 was inaccessible due to the Flange To Shell Configuration thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 25.56% (Single Side Coverage) and 46.98% (Two Direction Coverage) of the Reactor Vessel Shell To Flange Weld RV-W1 would require modification of the original design of Reactor Vessel.

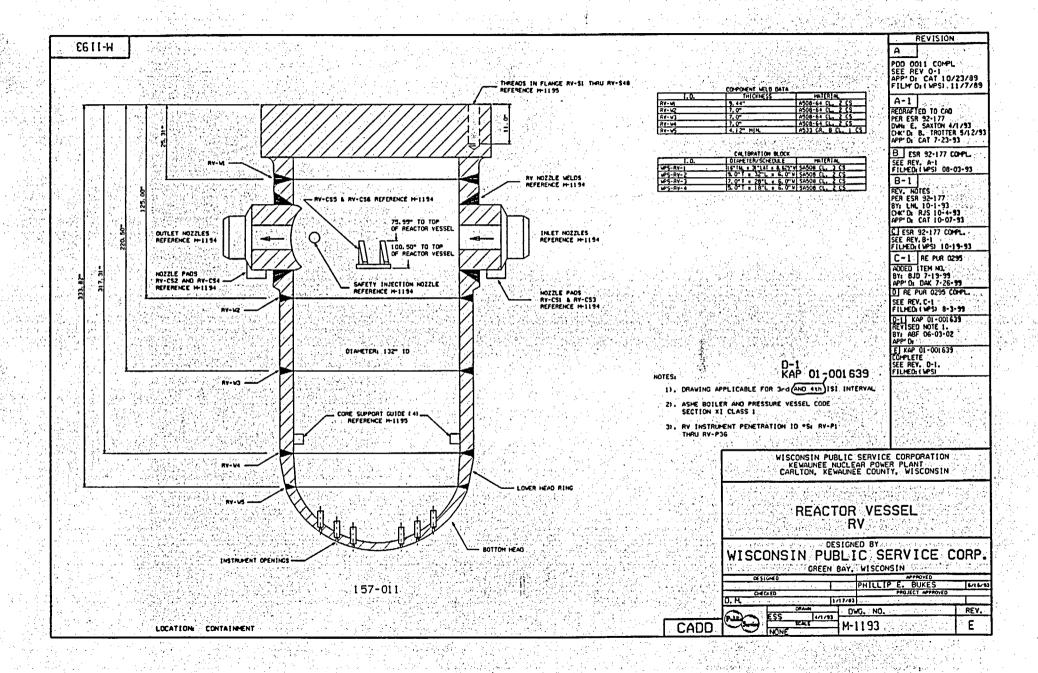
KEWAUNEE NUCLEAR POWER PLANT THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-71

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable



RESULTS NO. OF INDICATIONS N/A NI X STATUS N/A RI EXAM DOCUMENTATION INDICATION DOCUMENTATION EXPARAGON ANALYSIS LOG ASSESSMENT SHEET X PARAGON ACQUISITION LOG PARAGON HARD COPY X SCAN PRINT OUT OTHER (Specify) X COVERAGE BREAKDOWN	WELD NO. RV-W1	COMPONENT	Shell to Flange Weld
NI X STATUS N/A RI	LIMITATIONS: NO		74.44 % Complete (Single Sided Coverage Breakdown Sheet
EXAM DOCUMENTATION INDICATION DOCUMENTATION X PARAGON ANALYSIS LOG	RESULTS	NO. OF INDI	CATIONS N/A
EXAM DOCUMENTATION INDICATION DOCUMENTATION	NI <u>X</u>	STATUS	N/A
		TION INDICATION	<u>DOCUMENTATION</u>
SCAN PRINT OUT OTHER (Specify) COVERAGE BREAKDOWN	X PARAGON ANALYSIS L	.OG □ ASS	ESSMENT SHEET
SCAN PRINT OUT OTHER (Specify) COVERAGE BREAKDOWN	이 그렇게 하면 잘 되었습니다.)N LOG □ PAF	AGON HARD COPY
		□ОТЬ	IER (Specify)
mments: See figures 1 and 2 for limitations	X COVERAGE BREAKDO	WN	
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VELD	NO.	RV	'-W1						
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		WELD	Shear VOLUME	45 L S WELD	Single VOLUME	45 L WELD	26.05	WELD.	VOLUME
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WELD NORV-W1	COMPONENT Shell	to Flange Weld
LIMITATIONS: NO □		mplete (Two Directions) reakdown Sheet
RESULTS	NO. OF INDICATIONS	N/A
NI <u>X</u>	STATUS	N/A
RI EXAM DOCUMENTATION	N INDICATION DOCUME	<u>NTATION</u>
X PARAGON ANALYSIS LOG	□ASSESSMEN	TSHEET
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X SCAN PRINT OUT	OTHER (Spec	ify)
X COVERAGE BREAKDOWN		
X COVERAGE BREAKDOWN Comments: See figures 1 and 2		

	NAME	Kewa	aunee			We	sDy	ne		
VELD I	ΝΟ.	RV	-W1							
OMPO	ONENT	hell to Fl	ange Wel	1		Inte	rnati	ona		
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		100.00	57.64	100.00	57.64	100.00	57.64			

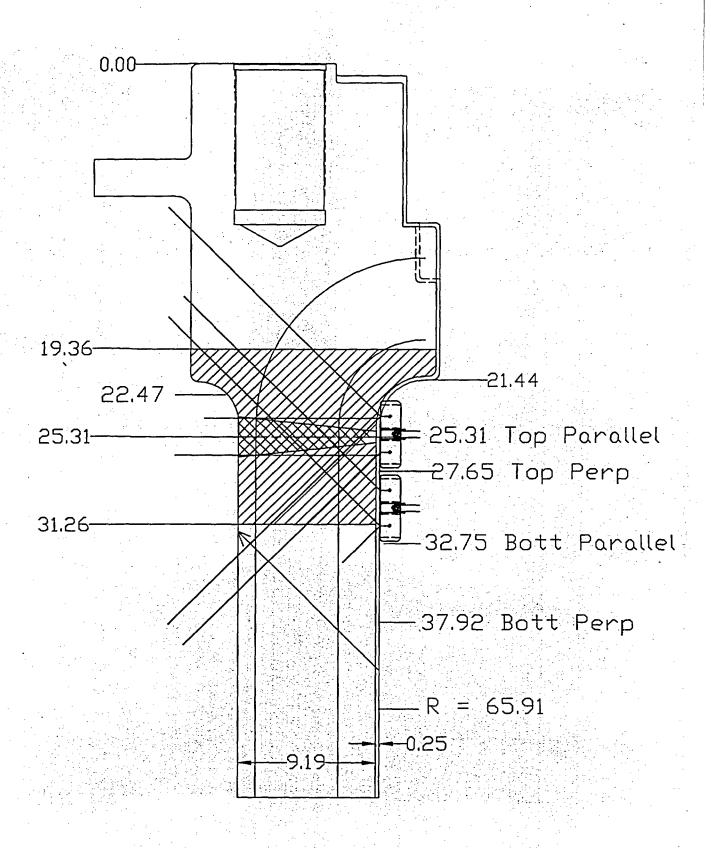


Figure 1

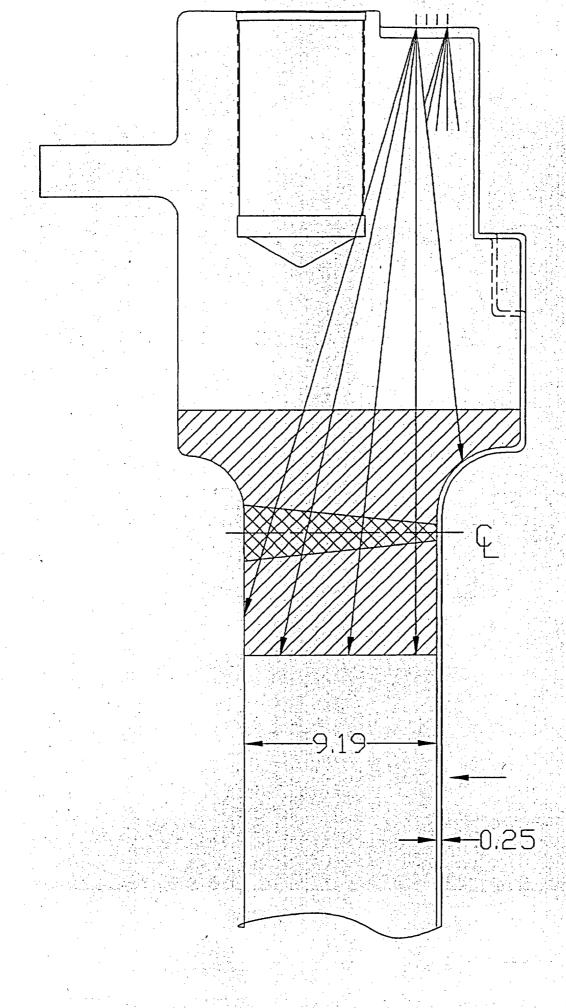


Figure 2

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-72

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

- 1. ASME Code Component Affected: Reactor Vessel Safety Injection Nozzle
 To Vessel Weld RV-W11
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-D; Item No. B3.90
- 4. Impracticality of Compliance: 11.69% (Two Sided Coverage) of the Remote Ultrasonic Examination of the Reactor Vessel Safety Injection Nozzle to Vessel Weld RV-W11 was inaccessible due to the Safety Injection Nozzle Protrusion thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 11.69% (Two Sided Coverage) of the Reactor Vessel Safety Injection Nozzle to Vessel Weld RV-W11 would require modification of the original design of Reactor Vessel Safety Injection Nozzle.

KEWAUNEE NUCLEAR POWER PLANT

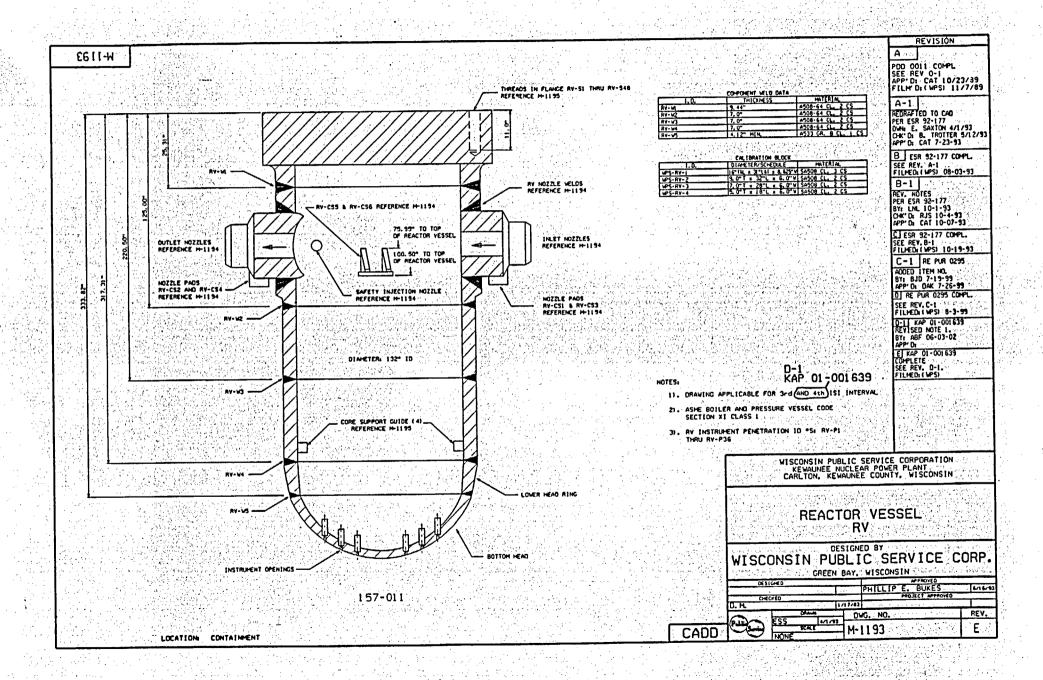
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-72

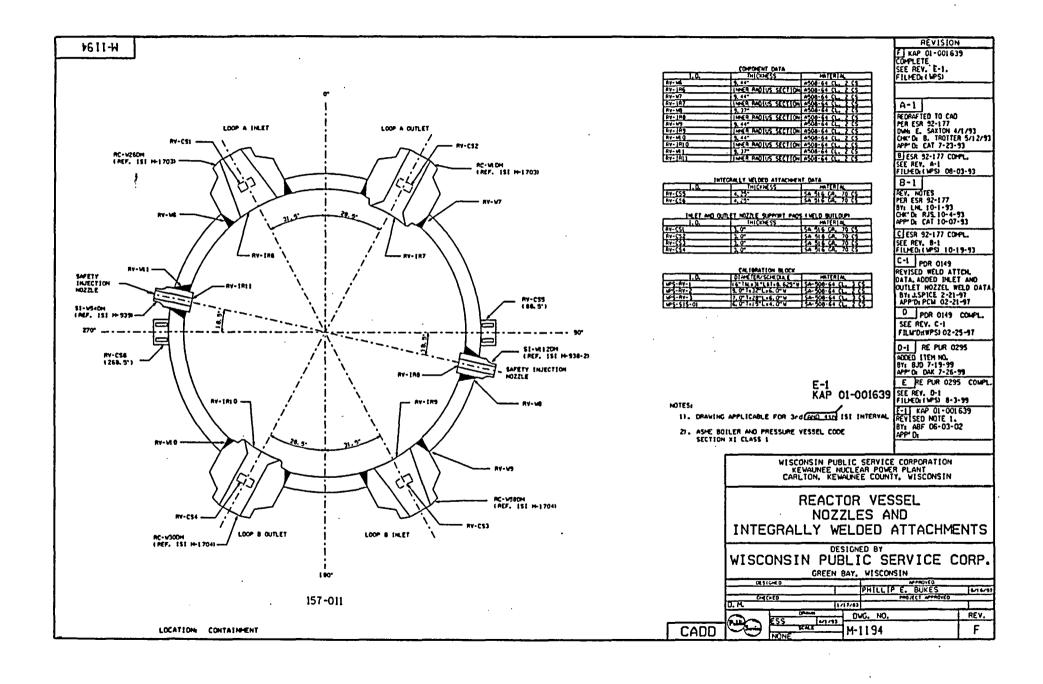
- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. However, 90.04% coverage of the Reactor Vessel Safety Injection Nozzle to Vessel Weld RV-W11 was achieved performing Remote Ultrasonic Examination with personnel qualified for performing Single Sided Examination of the Reactor Vessel Safety Injection Nozzle To Vessel Weld RV-W11.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable



and the Consequence Administration

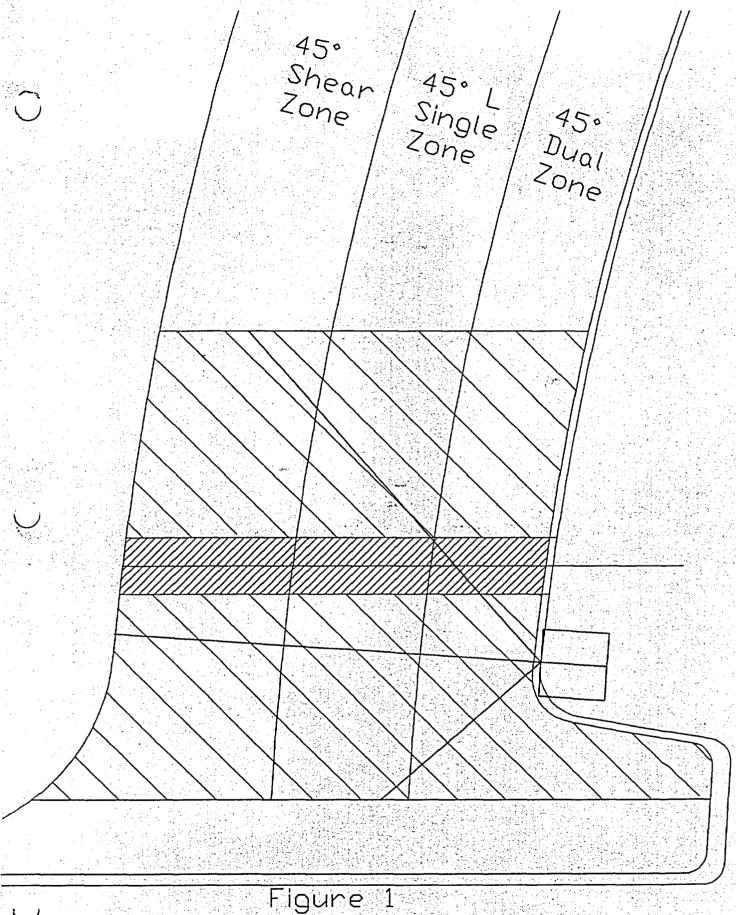


ANT NIANE	Kewaune	114	
ANT NAME	Newaune	<u>Unit</u>	
ÆLD NO.	RV-W11	COMPONENT	Safety Injection Noz-Shell @288.
MITATIONS: NO		YES X	88.31 % Complete (two sided coverage
			See Coverage Breakdown Sheet
RESULTS		NO. OF	INDICATIONS 1
NI		STATU	Code Allowable
NI <u> </u>		STATU	S <u>Code Allowable</u>
		STATU	S Code Allowable
RI)		STATU	S Code Allowable
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LD NO.	RV-V	V11						
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Combined Bore&Sta		Ber Miller Michigan					100.00	
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REA	CTOR VESSE	L WELD RES	SULTS SUMMARY
PLANT NAME	Kewaune	<u>Unit</u>	고 17일 : 1 시간 1
WELD NO.	RV-W11	_COMPONENT	Safety Injection Noz-Shell @288.5°
LIMITATIONS: 1	10	YES X	90.04 % Complete (single sided coverage) See Coverage Breakdown Sheet
RESULTS		NO. OF	INDICATIONS 1
NI S		STATU	S Code Allowable
RI	X		
EXAMI	OCUMENTATION	<u>INDICA</u>	TION DOCUMENTATION
X PARAGO	N ANALYSIS LOG		☐ASSESSMENT SHEET
X PARAGO	N ACQUISITION L	og 🗵	PARAGON HARD COPY
X SCAN PR	INT OUT		☐OTHER (Specify)
X COVERA	GE BREAKDOWN		
Comments: Coverage calculation and the Tan Scan (pa	is based on the Bore	e and Star scan (con	nbined) as perpendicular,
			undergreichen State (b. 1886) der Geber der der Ge
		Analyst	1911 Date: 11/13/11

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		And Felicians in	DEAIVI		SLE SIDE		RAGE		Fig. 18th March 2016 Comment of the
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Tan/Stan Coverage Illustration of the Safety Injection Nozzle Protrusion Limitation

KEWAUNEE NUCLEAR POWER PLANT

THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-73

PROPOSED ALTERNATIVE IN ACCORDANCE WITH 10CFR 50.55a(g)(5)(iii).

INSERVICE INSPECTION IMPRACTICALITY

- 1. ASME Code Component Affected: Reactor Vessel Lower Head
 Circumferential Weld RV-W4
- 2. Applicable Code Edition and Addenda: 1989, No Addenda
- 3. Applicable Code Requirement: Table IWB-2500-1; Examination Category B-A; Item No. B1.21
- 4. Impracticality of Compliance: 13.86% (Two Direction Coverage) of the Remote Ultrasonic Examination of the Reactor Vessel Lower Head Circumferential Weld RV-W4 was inaccessible due to the 4 Lower Core Supports thus restricting Ultrasonic Examination.
- 5. Burden Caused by Compliance: To provide for access to the 13.86% (Two Direction Coverage of the Reactor Vessel Lower Head Circumferential Weld RV-W4 would require modification of the original design of Reactor Vessel.

KEWAUNEE NUCLEAR POWER PLANT

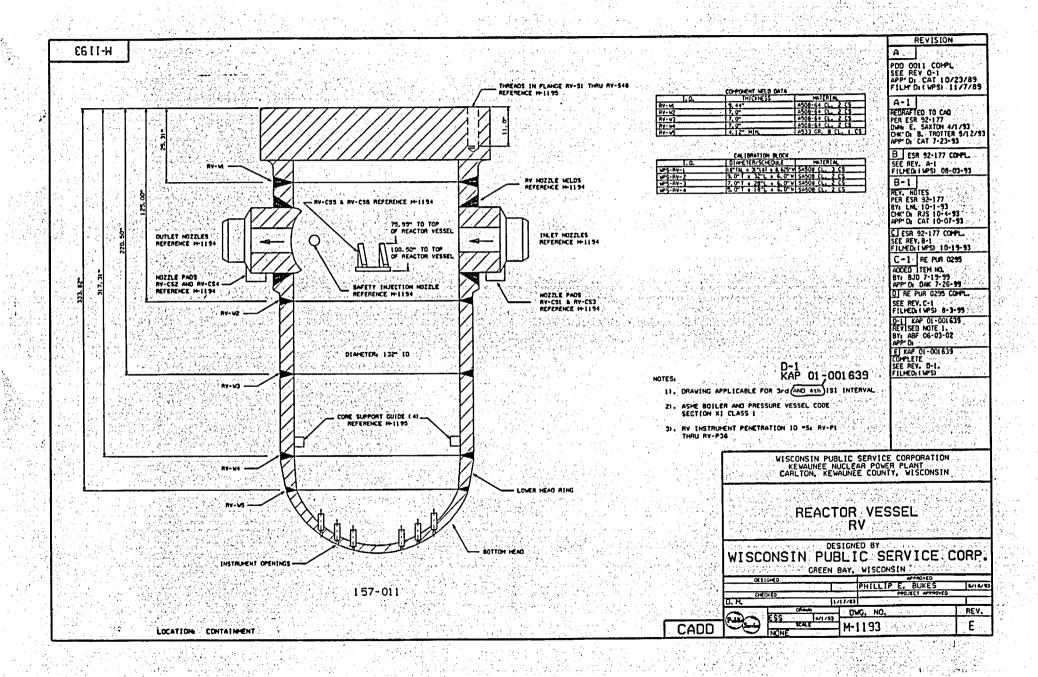
THIRD 10 - YEAR INTERVAL JUNE 16, 1994 - JUNE 16, 2004

RELIEF REQUEST NO: RR-G-7-73

- 6. Proposed Alternative and Basis for Use: No alternative Code required Ultrasonic examination is available due to the limited access. However, 90.82% coverage of the Reactor Vessel Lower Head Circumferential Weld RV-W4 was achieved performing Remote Ultrasonic Examination with personnel qualified for Single Sided Examination of the Reactor Vessel Lower Head Circumferential Weld RV-W4.
- 7. Duration of Proposed Alternative: 3rd Ten Year Interval June 16, 1994 June 16, 2004

8. Precedents: Not Applicable

9. References: Not Applicable



REACTOR VESS	EL WELD RE	SULTS SUMMARY
PLANT NAME Kewaung	<u>lee</u> Unit	
WELD NO. RV-W4	COMPONENT	Lower Shell to Lower Head
LIMITATIONS: NO X	YES	90.82 % Complete (One Direction) See Coverage Breakdown Sheet
RESULTS	NO. OI	F INDICATIONS N/A
NI X	STATU	JS <u>N/A</u>
RI		
EXAM DOCUMENTATIO	<u>IN</u> INDICA	ATION DOCUMENTATION
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Comments: See figure 1 for lin	mitations	
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	Analyst	12 Date: 11/12/04

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OMPC	ONENT Lowe	er Shell to	o Lower H	ead					
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		WELD	VOLUME	WELD	VOLUME	WELD	VOLUME	WELD	VOLUME
	Perpendicular	WELD 100.00	100.00	WELD 83.56	VOLUME 88.35	79.45 79.45	VOLUME 86.04	WELD	VOLUME

RE	ACTOR VESSE	L WELD RE	SULTS SUMM	IARY
PLANT NAME	Kewaunee	Unit		1
WELD NO.	RV-W4	COMPONENT	Lower Shel	II to Lower Head
LIMITATIONS:	NO X	YES	86.14 % Comp See Coverage Brea	olete (Two Directions) okdown Sheet
RESULTS		NO. OF	INDICATIONS	N/A
NI	<u> </u>	STATU	S	N/A
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COMPONENT Lower Shell to Lower Head	LANT	NAME	Kewa	aunee						
International Internationa							We	esDy	ne	
COMPONENT Lower Shell to Lower Head										
COMPONENT Lower Shell to Lower Head	/ELD I		ΚV	-VV4						
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BEAM ANGLE BREAK DOWN TWO DIRECTIONAL COVERAGE BEAM DIRECTION										
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AVERAGE 88.15		Parallel	100.00	92.55	100.00	92.55	79.45	87.86		
		AVERAGE	38 24 25 2	315 46 4	87	88	82	2.38		

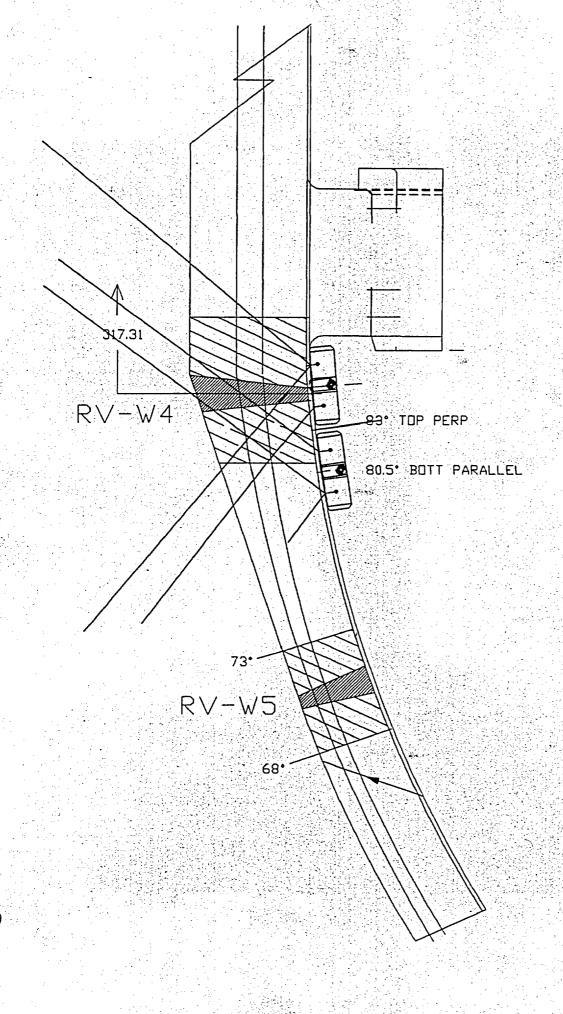


Figure 1