10.

4254

TRANSMITTAL SLIP

Date 4/17/05

TO: Bill Gott

FROM: David Dodson - MILLSTONE POWER STATION

☒ For Your Information	☐ For Your Signature
☐ For Your Recommendation	☐ Please Note and File
☐ Take Appropriate Action	☐ Note and Return To Me
☐ Please See Me About This	☐ Additional Information is Requested
☐ As You Requested	
☐ Prepare Reply For My Signature	☐ Please Answer
☐ For Your OK (Please Return)	

COMMENTS:

5 ~~4~~ and 9 and pages including cover

[redacted] Ex 2

[redacted]

OS425

H/2

UPDATE 1916 HOURS 4/17/05

Attachment 8
NRC Event Notification Form
(Sheet 1 of 2)

NRC CONTACT (NAME) Bill Gott NRC EVENT NUMBER: 41607

NOTIFICATION TIME	FACILITY OR ORGANIZATION	UNIT	NAME OF CALLER	TELEPHONE NUMBER
1330	Millstone	3	DAVID BODSWORTH	(860) 443-2276

EVENT TIME (EST OR EDT)	EVENT DATE or IRF#	POWER (%) and MODE BEFORE	POWER (%) and MODE AFTER
0842	2005044	100% MODE 1	0% MODE 3

EVENT CLASSIFICATION	1-Hr Non-Emergency 10 CFR 50.72 (b)(1)	4-Hr Non-Emergency 10 CFR 50.72 (b)(2)	8-Hr Non-Emergency 10 CFR 50.72 (b)(3)
GENERAL EMERGENCY	TS Deviation	(i) TS Required Shutdown	(ii)(A) Degraded Condition
SITE AREA EMERGENCY		X (iv)(A) ECCS Discharge to RCS	(ii)(B) Unanalyzed Condition that Degrades Safety
X ALERT	60 Day Optional 10 CFR 50.73 (a)(1)	X (iv)(B) RPS Actuation (Scram)	X (iv)(A) Specified System Actuation
UNUSUAL EVENT	Invalid Specified System Actuation	(xi) Offsite Notification	(v)(A) Safe Shutdown Capability
X 50.72 NON-EMERGENCY			(v)(B) RHR Capability
PHYSICAL SECURITY (73.71)			(v)(C) Control of Rad Release
MATERIAL/EXPOSURE			(v)(D) Accident Mitigation
FITNESS FOR DUTY			(xii) Offsite Medical
OTHER UNSPECIFIED REQUIREMENT (IDENTIFY)			(xiii) Lost ENS
INFORMATION ONLY			(xiii) Lost Other Assessment/Offsite Comms
			(xiii) Emergency Siren INOP

OTHER UNSPECIFIED REQUIREMENT (IDENTIFY):

DESCRIPTION: (Fill in as available)

1. SYSTEM(S) AFFECTED: Reactor Trip, Safety Injection, AFW actuation
2. ACTUATIONS & THEIR INITIATION SIGNALS: Reactor Trip/SI/AFW - Low Steam line pressure
3. CAUSES (IF KNOWN): Cause is under investigation
4. EFFECT OF EVENT ON PLANT: Various Main Steam Safety Valves, Primary Safety valve actuated
5. ACTIONS TAKEN OR PLANNED: Stabilize plant, coddown to Mode 5 cold shutdown
6. ADDITIONAL INFORMATION: SEE ATTACHMENT

Level of Use Reference

Attachment 8
NRC Event Notification Form
(Sheet 2 of 2)

NOTIFICATIONS	YES	NO	WILL BE	ANYTHING UNUSUAL OR NOT UNDERSTOOD?	YES ☑ (EXPLAIN SHEET 1)	NO ☐
NRC RESIDENT	✓			DID ALL SYSTEMS FUNCTION AS REQUIRED?	YES ☐	NO ☑ (EXPLAIN SHEET 1)
STATE	✓			MODE OF OPERATION UNTIL CORRECTED: 5		
LOCAL	✓			ESTIMATED RESTART DATE: Unknown		
OTHER GOV AGENCIES	✓			ADDITIONAL INFO ON BACK: YES ☐	NO ☑	
MEDIA/PRESS RELEASE	✓					

RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS

☐ LIQUID RELEASE	☐ GASEOUS RELEASE	☐ UNPLANNED RELEASE	☐ PLANNED RELEASE	☐ ONGOING	☐ TERMINATED		
☐ MONITORED	☐ UNMONITORED	☐ OFFSITE RELEASE	☐ T. S. EXCEEDED	☐ RM ALARMS	☐ AREAS EVACUATED		
☐ PERSONNEL EXPOSED OR CONTAMINATED		☐ OFFSITE PROTECTIVE ACTIONS RECOMMENDED		State release path in description			

	Release Rate (Ci/sec)	%T.S. LIMIT	HOO GUIDE	Total Activity (Ci)	% T.S. LIMIT	HOO GUIDE
Noble Gas			0.1 Ci/sec			1000 Ci
Iodine			10 µCi/sec			0.01Ci
Particulate			1 µCi/sec			1 mCi
Liquid (excluding tritium & dissolved noble gases)			10 µCi/min			0.1 Ci
Liquid (tritium)			0.2 Ci/min		--	5 Ci
Total Activity						

	PLANT STACK	CONDENSER/AIR EJECTOR	MAIN STEAM LINE	SG BLOWDOWN	OTHER
RAD MONITOR READINGS:	1 E20	4.3 E-6	7.48E-2	isolated	
ALARM SETPOINTS					
% T.S. LIMIT (if applicable)					

RCS OR SG TUBE LEAKS: CHECK OR FILL IN APPLICABLE ITEMS:

LOCATION OF THE LEAK (e.g., SG#, valve, pipe, etc): None

LEAK RATE:	UNITS gpm/gpd	T. S. LIMITS:	SUDDEN OR LONG-TERM DEVELOPMENT
LEAK START DATE: TIME:		COOLANT ACTIVITY & UNITS: PRIMARY –	SECONDARY –

LIST OF SAFETY RELATED EQUIPMENT NOT OPERATIONAL:

SM/DSEO Signature : [signature] for B. Hoffer Time: 1320

Level of Use Reference

Attachment to April 17, 2005 Event Notification
Summary of Event

At approximately 0830 on April 17, 2005 Millstone Unit 3 experienced an automatic reactor trip from full power. The Reactor is in a stable shutdown condition and is proceeding to Mode 5 cold shutdown. No release of radioactivity other than those minor releases associated with normal plant operation has occurred. There were no personnel injuries associated with the event.

In addition to the Reactor Trip, half-train Safety Injection (SI) and half-train Main Steam Isolation (MSI) actuations occurred. Control room personnel were successful at manually initiating full SI and MSI. SI has since been terminated and RCS pressure control has been restored. Multiple steam line safety valves (MSSV) lifted with at least one MSSV indicating a failure to reclose. As a result, an ALERT was declared at approximately 0853 due to the failure of at least one MSSV failing to reclose. All MSSV's are currently closed. Auxiliary Feedwater (AFW) actuated automatically as expected, however; the turbine driven auxiliary feedwater pump (TDAFWP) tripped on start-up and was subsequently reset locally. Both motor driven auxiliary feedwater pumps (MDAFWPs) operated as expected to maintain steam generator water levels. Due to the additional inventory injected to the Reactor Coolant System (RCS) as a result of the SI actuation, the pressurizer filled and a primary system safety valve (PSSV) actuated at a lower than expected pressure. All PSSV's are currently closed, however; there is indication of leakage past either a PSSV or Pilot Operated Relief Valve (PORV). Upon alignment of the Charging system to Refueling Water Storage Tank recirculation flow path, 2 valves had indication of packing leakage. Those packing leaks have been terminated.

The cause of the event is under investigation.

Attachment to April 17, 2005 Event Notification
Summary of Event

Event No. 41607

Update to previous information:

The previously reported time of the ALERT declaration should be corrected to read 0842. Millstone Unit 3 entered Mode 4 at approximately 1903 on April 17, 2005. The event at Millstone Unit 3 was terminated at 1905. The cause of event is under investigation. A recovery plan is being formulated.