

September 7, 2005

Mr. Christopher M. Crane
President and Chief Executive Officer
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - REFUELING OUTAGE
T1R16 STEAM GENERATOR TUBE INSPECTION (TAC NO. MC8120)

Dear Mr. Crane:

Inservice inspections of steam generator (SG) tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. As required by the plant technical specifications, such reporting requirements range from submitting a special report, within 15 days following completion of each inservice inspection of SG tubes, identifying the number of tubes plugged and/or repaired, to submitting a special report, within 12 months following completion of the inspection, providing complete results of the SG tube inservice inspection.

We have requested a telephone conference with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Refueling Outage T1R16. This discussion will occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Enclosed is a list of discussion points to facilitate this phone conference. Upon completion of the discussion we plan to prepare a brief summary of the conference call and docket any supporting material that you may have provided in support of the call.

If you need clarification of this matter, please feel free to contact me at 301-415-1451.

Sincerely,

/RA/

Peter S. Tam, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: Discussion Points

cc w/encl: See next page

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ACCESSION NO.: **ML052300459**

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DATE	8/26/05	8/26/05	8/31/05	9/07/05

OFFICIAL RECORD COPY

STEAM GENERATOR (SG) TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMERGEN ENERGY CO., LLC

THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1)

DOCKET NO. 50-289

The following discussion points have been prepared to facilitate the telephone conference arranged with AmerGen Energy Co., LLC, to discuss the results of the SG tube inspections to be conducted during the upcoming Three Mile Island Nuclear Station, Unit 1, Refueling Outage T1R16. This phone call is expected to occur towards the end of the planned SG tube inspection interval, but before the licensee completes the inspections and repairs.

The Nuclear Regulatory Commission (NRC) staff plans to prepare a brief summary of the conference call and docket any material that the licensee may have provided in support of the call.

- (1) Discuss any trends in the amount of primary-to-secondary leakage observed during the recently completed cycle.
- (2) Discuss whether any secondary side pressure tests were performed during the outage and the associated results.
- (3) Discuss any exceptions taken to the industry guidelines.
- (4) For each SG, provide a description of the inspections performed including the areas examined and the probes used (e.g., dents/dings, sleeves, expansion-transition), the scope of the inspection (e.g., 100% of dents/dings greater than 5 volts and a 20% sample between 2 and 5 volts), and the expansion criteria. Also, discuss the extent of the rotating probe inspections performed in the portion of tube below the expansion transition region (reference NRC Generic Letter 2004-01, "Requirements for Steam Generator Tube Inspections").
- (5) For each area examined (e.g., tube supports, dent/dings, sleeves, etc), provide a summary of the number of indications identified to-date of each degradation mode (e.g., number of circumferential primary water stress corrosion cracking indications at the expansion transition). For the most significant indications in each area, provide an estimate of the severity of the indication (e.g., provide the voltage, depth, and length of the indication). In particular, address whether tube integrity (structural and accident-induced leakage integrity) was maintained during the previous operating cycle. In addition, discuss whether any location exhibited a degradation mode that had not previously been observed at this location at this unit (e.g., observed circumferential primary water stress corrosion cracking at the expansion transition for the first time at this unit).

Enclosure

- (6) Describe repair/plugging plans.
- (7) Describe in-situ pressure test and tube pull plans and results (as applicable and if available).
- (8) Provide the schedule for SG-related activities during the remainder of the current outage.
- (9) Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts
 - a description of any loose parts detected and their location within the SG
 - if the loose parts were removed from the SG
 - indications of tube damage associated with the loose parts
 - the source or nature of the loose parts if known
- (10) Once-through SGs - If you have welded plugs provided by Babcock and Wilcox (B&W) installed in the SGs, be prepared to discuss the actions taken in response to Framatome's notification of the effect of tubesheet hole dilation on the service life of B&W welded plugs.
- (11) Once-through SGs - Discuss any actions taken in response to the severed tube issue during the outage (NRC Information Notice (IN) 2002-02 and IN 2002-02, Supplement 1). If actions are complete, please so indicate.

Three Mile Island Nuclear Station, Unit 1

cc:

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