

Exelon Generation Company, LLC  
Dresden Nuclear Power Station  
6500 North Dresden Road  
Morris, IL 60450-9765

www.exeloncorp.com

10 CFR 50.4  
10 CFR 50.59

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U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Dresden Nuclear Power Station, Units 1, 2, and 3  
Facility Operating License No. DPR-2  
Renewed Operating License Nos. DPR-19 and DPR-25  
NRC Docket Nos. 50-01, 50-237 and 50-249

Subject: 10 CFR 50.59 Report

Enclosed is the report of completed changes, tests, and experiments in accordance with 10 CFR 50.59, "Changes, tests, and experiments," paragraph (d)(2) for Dresden Nuclear Power Station (DNPS). These evaluations use the criteria identified in 10 CFR 50.59 for determining whether a proposed change, test, or experiment shall require NRC approval prior to implementation of the proposed activities.

Should you have any questions regarding this report, please contact Mr. Pedro Salas, Regulatory Assurance Manager at (815) 416-2800.

Respectfully,



Danny G. Bost  
Site Vice President  
Dresden Nuclear Power Station

Enclosure

cc: Regional Administrator – Region III  
NRC Senior Resident Inspector – Dresden Nuclear Power Station

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## 10 CFR 50.59 Safety Evaluation Summary Report

Safety Evaluation	2002-03-010	Type of Safety Evaluation Reference	Modification ECs 335869, 335870, 335871,
Title:	Install Pull-To-Lock Control Switch for LPCI Valve 2(3)-1501-22A&B		
Description:	The design change provides a mechanism for overriding the Low Pressure Coolant Injection (LPCI) injection valves (2(3)-1501-22A&B) automatic injection function. The existing control switch (2(3)-1530-304A&B) on the 902(3)-3 panel will be replaced with a new switch having a PULL-TO-CLOSE feature. The new control switch will be installed in the same place as the existing switch. In addition, there will be wiring changes performed to include an alarm in Annunciator Section 2 of the 902(3)-3 Control Panel when the control switch is placed in Pull-To-Lock.		
Result:	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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Safety Evaluation	2002-03-011	Type of Safety Evaluation Reference	Modification EC 337644, 337645
Title:	Powerplex Upgrade		
Description:	The design change will upgrade the existing Powerplex II core monitoring system to a newer Powerplex III version. It also adds two new engineering workstations.		
Result:	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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Safety Evaluation	2003-01-001	Type of Safety Evaluation Reference	Modification EC DCPs 340263, 340264,
Title:	Removal of GL 96-06 Pressure Relief Devices		
Description:	Engineering Change (EC) Design Change Packages (DCPs) 340263, 340264, and 340265 will remove the pressure relief devices (i.e. check valve or pressure relief valve) and associated piping from Dresden Unit 2 that were installed to address post-accident, thermally-induced pressurization of sections of piping between the inboard and outboard isolation valves on the Reactor Water Clean Up (RWCU) (X-113 penetration), Shutdown Cooling (SDC) (X-111A/B penetrations), and Reactor Recirculation (RR) sampling (X-122 penetration) systems. Calculations performed using the plastic analysis method described in ASME Section III, Appendix F, have shown that the post-accident stresses developed in the piping sections and components are acceptable without the use of pressure relief devices. These pressure relief devices also currently function as containment isolation valves. The local leak rate (LLRT) taps on the discharge piping of the check valve on the RWCU system and the RR system will be relocated since the piping will be removed. Following installation, the LLRT taps will attach directly to the main RWCU piping upstream of the 2-1201-1 valve, and to the RR piping upstream of valve 2-0220-44. This 50.59 also addresses changes to procedures, the Technical Requirements Manual (TRM), and other owner-controlled documents such as the Updated Final Safety Analysis (UFSAR)		

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to reflect the removal of the pressure relief devices and associated piping.

The specific valves that will be removed are:

DCP 340263 - will remove relief valve 2-1099-29 on the SDC system  
(penetrations X-111A & B)

DCP 340264 - will remove check valve 2-1299-285 on the RWCU system  
(penetration X-113)

DCP 340265 - will remove check valve 2-0220-141 on the RR sampling system  
(penetration X-122)

**Result:** The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.

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Safety Evaluation	2003-01-002	Type of Safety Evaluation Reference	Modification EC 341118
<b>Title:</b>	3A Recombiner Dilution Steam Low Pressure - Revise Alarm Setpoint and Trip		
<b>Description:</b>	This design change revises the alarm setpoint of the 3A Off-Gas (OG) Recombiner Dilution Steam Low Pressure Switch, PSL 3-3041-21A, removes its trip function, and removes the interlock trip on the 3A OG Recombiner Inlet SV, 3-5499-76. These changes are a result of Extended Power Uprate (EPU) changes to the system.		
<b>Result:</b>	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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Safety Evaluation	2003-03-005	Type of Safety Evaluation Reference	Modification 2003-03-005
<b>Title:</b>	TR 86 Modification Implementation EC 340723		
<b>Description:</b>	Installation of New 138kV feed to Dresden Unit 2 138kV Reserve Auxiliary Transformer 22 (RAT22)		
	This design change installs a new 138kV feed to the Dresden Reserve Auxiliary Transformer #22 (RAT 22). Currently, the 138kV switchyard bus 1 (normal feed) supplies the source of offsite power through RAT 22. RAT 22 is connected to the 138kV switchyard by means of duplicate lines permitting connection to either of the two bus sections. The two connections are separated in the 138kV switchyard by one normally open and one normally closed circuit breaker, and via the 138kV transfer structure by two normally closed and two normally open disconnect switches on each line.		
<b>Result:</b>	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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intergranular stress corrosion cracking (IGSCC) resistant material. The new downcomer couplings, sparger clamps, and piping flexible joint assemblies are supported to ensure allowable loads are not exceeded.

**Result:** The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.

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Safety Evaluation	2004-03-008	Type of Safety Evaluation Reference	Modification EC 352013 AND EC 352012
<b>Title:</b>	REVISE THE U2(3) RECIRCULATING PUMP SPEED CONTROL CIRCUIT FOR OPEN LOOP OPERATION		
<b>Description:</b>	This proposed activity redesigns the RR Speed Control circuit. It removes the speed feedback signal and eliminates the noise problems associated with the speed signal from the tachometer causing the MG sets to be unstable and oscillate. The Rack Mounted Speed Controllers and Error Networks are being removed and the corresponding circuits rewired. The minimum and maximum speed limits are being increased slightly to prevent entering the MG fluid coupler unstable operating region. In addition, the "B" Manual/Automatic (M/A) Speed Transfer Station is being replaced with a Bias M/A Speed Transfer Station. This reduces the total load impedance on Master Controller since the current load impedance is exceeding the upper load impedances limit potentially causing the circuit to become unstable.		
<b>Result:</b>	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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Safety Evaluation	2004-04-010	Type of Safety Evaluation Reference	Temporary Alteration EC 352995
<b>Title:</b>	DISABLE 2D MOISTURE SEPARATOR HIGH LEVEL AUTOMATIC TURBINE TRIP		
<b>Description:</b>	This temporary alteration will defeat the automatic turbine trip from the 2D Moisture Separator level switches LS (2-3541-54D) and LS (2-3541-34D). The intent of this TCCP is to defeat the automatic turbine trip to prevent an unnecessary automatic turbine trip/reactor scram due to a degraded cable. These level switches provide two out of two logic with a 10-second time delay that automatically trips the turbine (with a subsequent reactor scram), protecting the low pressure turbine from potential damage due to moisture carryover from the 2D moisture separator. High level and High-High level alarms are also provided by these switches. Spurious false high level alarms have occurred due to degraded conductors within cable 21576. Based on recent troubleshooting efforts (WO 763381), it appears that the cable (21576) located inside the low pressure heater bay is degraded and will need to be replaced, which requires a unit shutdown due to high dose rates within the heater bay.		
<b>Result:</b>	The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.		

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Safety Evaluation      2005-01-001      Type of Safety      Temporary Alteration  
Evaluation Reference      TCCP 354622

Title:      Install Temporary Jumper on A37 Steam Line Resonance Compensator Card

Description:      A temporary jumper will be installed on the Unit 2 Electro-Hydraulic Control System Steam Line Resonance Compensator (SLRC) A37 circuit card in the 902-31 panel to bypass the "A" Main Steam Pressure Regulator (MSPR). The exact cause of the Unit 2 scram on March 24, 2005, could not be determined. Subsequent investigation and troubleshooting found the "A" MSPR as a suspected cause of the event. This jumper is being installed as a precaution to allow the "A" MSPR to be bypassed by operations during plant startup and at other times as needed.

Result:      The evaluation concluded that none of the eight criteria specified in 10 CFR 50.59(c)(2), as appropriate, were met. Therefore, prior NRC review and approval pursuant to 10 CFR 50.59(c)(1) was not required.