



U.S. NUCLEAR REGULATORY COMMISSION  
**STANDARD REVIEW PLAN**  
OFFICE OF NUCLEAR REACTOR REGULATION

## 16.0 TECHNICAL SPECIFICATIONS

### REVIEW RESPONSIBILITIES

Primary - ~~Licensing Guidance Branch (LGB)~~ Technical Specifications Branch (TSB)<sup>1</sup>

Secondary - All technical review branches and Licensing Project Manager  
Materials and Chemical Engineering Branch (EMCB),  
Mechanical Engineering Branch (EMEB),  
Civil Engineering and Geosciences Branch (ECGB),  
Electrical Engineering Branch (EELB),  
Instrumentation and Controls Branch (HICB),  
Human Factors Assessment Branch (HHFB),  
Quality Assurance and Maintenance Branch (HQMB),  
Emergency Preparedness and Radiation Protection Branch (PERB),  
Containment Systems and Severe Accident Branch (SCSB),  
Plant Systems Branch (SPLB),  
Probabilistic Safety Assessment Branch (SPSB),  
Reactor Systems Branch (SRXB), and  
Office of Nuclear Reactor Regulation (NRR) Project Manager<sup>2</sup>

### I. AREAS OF REVIEW

Section 50.36 of 10 CFR Part 50 requires that each operating license issued by the Commission contain technical specifications that set forth the limits, operating conditions, and other requirements imposed upon facility operation for the protection of the health and safety of the public. As part of the regulatory standardization effort, the staff has prepared generic<sup>3</sup> standard technical specifications (STSs)<sup>4</sup> for each of the light-water reactor nuclear steam supply systems and associated balance-of-plant equipment systems. These STSs<sup>5</sup> are subject to revision, and the

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#### USNRC STANDARD REVIEW PLAN

Standard review plans are prepared for the guidance of the Office of Nuclear Reactor Regulation staff responsible for the review of applications to construct and operate nuclear power plants. These documents are made available to the public as part of the Commission's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Standard review plans are not substitutes for regulatory guides or the Commission's regulations and compliance with them is not required. The standard review plan sections are keyed to the Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants. Not all sections of the Standard Format have a corresponding review plan.

Published standard review plans will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience.

Comments and suggestions for improvement will be considered and should be sent to the U.S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Washington, D.C. 20555.

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latest versions are available from the ~~Division of Technical Information and Document Control,~~ ~~NRC~~ Government Printing Office Sales Program and the National Technical Information Service.<sup>6</sup> The initial implementation of the STS program was made on the D. C. Cook operating license issued in October 1974. All subsequent operating licenses issued by the Commission will utilize the appropriate ~~generic~~<sup>7</sup> STS as the basis for issuance of Appendix A of licenses, "Technical Specifications." The current STS were developed as the result of the Commission's Technical Specifications Improvement Program in September 1992 and Crystal River Unit 3 was the first to implement them in October 1993.<sup>8</sup>

Applicants should use<sup>9</sup> the current ~~generic~~<sup>10</sup> STS as the basis for preparation of proposed Appendix A Technical Specifications items for Section 16.0 of preliminary safety analysis reports (PSARs). The proposed Appendix A Technical Specifications items will be reviewed to determine that the content and format are consistent with the applicable ~~generic~~<sup>11</sup> STS. Special attention will be given to those specifications which deviate from the ~~generic~~<sup>12</sup> STS to determine that proposed differences are justified on the basis of uniqueness in plant design or other considerations. Specifications so identified will be reviewed in detail to identify areas that may influence the acceptability of the final facility design. In particular, this portion of the review will determine the acceptability of proposed specification items that describe features affecting the type, capacity, number, or performance of surveillance activities involving safety-related systems.

At the PSAR stage, ~~n~~Numerical<sup>13</sup> values, graphs, and other data proposed will not be as complete as specified in the ~~generic~~<sup>14</sup> STS because of the preliminary nature of the plant design. The review of information that is provided in this area will be limited to determining that the values are in reasonable agreement with the expected operational capability of the plant.

The ~~Licensing~~<sup>15</sup> Project Manager will use the ~~generic~~<sup>16</sup> STS as a basis for determining that the proposed items are satisfactory; and ~~He~~<sup>17</sup> will consult the ~~Licensing Guidance Branch~~TSB<sup>18</sup> and ~~with~~<sup>19</sup> ~~Technical Reviewers~~<sup>20</sup> for resolution of novel or questionable aspects of the submittal including items deviating from the STS.<sup>21</sup>

Applicants should use the current ~~generic~~<sup>22</sup> STS as the basis for preparation of proposed Appendix A Technical Specifications for Section 16.0 of final safety analysis reports (FSARs). The Appendix A ~~t~~Technical ~~s~~Specifications<sup>23</sup> submitted in support of an operating license will be the finalized version of those technical<sup>24</sup> specifications originally identified in the PSAR and will reflect the final refinements in design, results of tests, and expected method of operation.

Each ~~generic~~<sup>25</sup> STS will be maintained current and updated periodically to reflect:

- (1) Changes in classes of plants or modifications of nuclear steam supply systems or balance-of-plant equipment systems.
- (2) Revised regulatory requirements.
- (3) Experience obtained by the NRC staff in reviewing proposed technical specification changes from licensees.

- (4) Operational experience obtained from licensees, <sup>26</sup> and the Office for Analysis and Evaluation of Operational Data, ~~the Divisions of Safety Technology and Licensing in NRR, and the Office of Inspection and Enforcement,~~ the Events Assessment and Generic Communications Branch (PECB), and/or the Inspection Program (PIPB) Branch in NRR. <sup>27</sup>

The ~~EGBT~~TSB<sup>28</sup> will determine the acceptability of the proposed technical<sup>29</sup> specifications.

Review Interfaces<sup>30</sup>

In addition, the ~~EGBT~~TSB<sup>31</sup> will coordinate secondary reviews performed by the technical review branches and by the ~~Licensing~~<sup>32</sup> Project Manager, as follows. <sup>33</sup>

1. <sup>34</sup> The technical review branches will be consulted, as necessary, to determine the validity of plant-specific features, methods, and numerical values proposed by the applicant.
2. The ~~Licensing~~<sup>35</sup> Project Manager shall integrate the technical specification effort into the licensing process and shall advise the ~~EGBT~~TSB<sup>36</sup> of any technical specifications; ~~different from the STS, that have been found to be necessary-related matters that have been identified as deficient during the licensing process.~~<sup>37</sup>
3. The secondary reviewers will report the results of their evaluations as indicated in subsection III of this SRP section.

II. ACCEPTANCE CRITERIA

- A. The proposed technical specifications will be considered to satisfy ~~Sections 10 CFR 50.34 and 10 CFR 50.36 of 10 CFR Part 50,~~<sup>38</sup> and therefore be acceptable, if they are consistent with the regulatory guidance contained in the following STS documents and contain plant-specific parameters ~~and additional technical specification requirements considered appropriate by the regulatory staff~~ at an appropriate level of detail:<sup>39</sup>

<u>NSSS Vendor</u>	<u>STS Publication</u>
Babcock and Wilcox Co.	NUREG-0103
Combustion Engineering, Inc.	NUREG-0212
General Electric Co.	NUREG-0123
Westinghouse Electric Corp.	NUREG-0452
	NUREG-1430, STS, Babcock and Wilcox Plants
	NUREG-1431, STS, Westinghouse Plants
	NUREG-1432, STS, Combustion Engineering Plants
	NUREG-1433, STS, General Electric Plants, BWR/4
	NUREG-1434, STS, General Electric Plants, BWR/6 <sup>40</sup>

For applicants referencing a certified design, the Standard Technical Specifications associated with the referenced design will provide the guidelines used as the basis for the evaluation of proposed plant-specific technical specifications. <sup>41</sup>

- B. The Technical Specifications must include limiting conditions for operation for all items meeting one or more of the following criteria as specified in 10 CFR 50.36(c)(2):
1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
  2. A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
  3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
  4. A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.<sup>42</sup>

### Technical Rationale

The technical rationale for application of these acceptance criteria to reviewing technical specifications is discussed in the following paragraphs:<sup>43</sup>

1. Compliance with 10 CFR 50.34 requires that, at the construction permit stage, the applicant identify and justify the selection of those variables, conditions, or other items identified through preliminary safety analysis as probable subjects for plant-specific technical specifications. Special attention should be given to those items that could significantly influence the final design. At the operating license stage, compliance with 10 CFR 50.34 requires that the applicant propose technical specifications prepared in accordance with 10 CFR 50.36.

Meeting the requirements of 10 CFR 50.34 provides assurance that those items requiring special attention at the construction permit stage are identified sufficiently early to preclude the need for any significant design change to support the facility's final technical specifications. Meeting the requirements of 10 CFR 50.34 at the operating license stage requires that the applicant propose technical specifications prepared in accordance with 10 CFR 50.36. The safety benefit of such a process is discussed in subsection II.2 of this SRP section.<sup>44</sup>

2. Compliance with 10 CFR 50.36 requires that the proposed technical specifications include the following:
  - a. Safety Limits and Limiting Safety System Settings. Safety limits apply to important process variables that are necessary to provide an appropriate level of protection for the integrity of certain physical barriers that guard against the uncontrolled release of radioactive material. Limiting safety system settings are

used for automatic protective devices affecting those variables having significant safety functions.

b. Limiting Conditions for Operation. Limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met. A technical specification limiting condition for operation of a nuclear reactor must be established for each item meeting one or more of the following criteria:

- Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
- A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

c. Surveillance Requirements. Surveillance requirements govern testing, calibration, or inspection of systems and components to ensure that an adequate level of quality is maintained.

d. Design Features. Design features affect aspects of the facility (e.g., construction materials and geometric arrangements) that, if altered or modified, would have a significant effect on safety and are not covered in the categories described above.

e. Administrative Controls. Administrative controls are provisions related to organization and management, procedures, recordkeeping, review and audit, and reporting — all of which are necessary to ensure the safe operation of the facility.

SRP Section 16.0 describes staff positions on the content and format of those technical specifications that are standardized for each major reactor vendor (i.e., as STS).

Meeting the requirements of 10 CFR 50.36 provides assurance that essential safety-related items and/or issues associated with facility design and operation (i.e., those derived from analyses and evaluations included in the SAR) will be identified.

Appropriate information related to these items and/or issues will then be appended to the operating license to ensure compliance and enforceability.<sup>45</sup>

### III. REVIEW PROCEDURES

The review of these<sup>46</sup> technical<sup>47</sup> specifications will be done on an item-by-item basis to determine the comparability with the applicability, format, and specific content of the current generic<sup>48</sup> STS. The technical<sup>49</sup> specification differences and supporting bases will be reviewed to determine their acceptability on the basis of the specific plant design and other pertinent considerations.

At the FSAR stage of the review, the<sup>50</sup> numerical values, graphs, tables, and other data proposed for each technical<sup>51</sup> specification should be as complete as specified in the generic<sup>52</sup> STS. This information will be reviewed to ensure conformance with material presented in applicable portions of the FSAR as summarized in the supporting basis for each technical<sup>53</sup> specification.

In performing the review, the reviewer shall consult with specialists in the technical review branches, as deemed necessary, to determine the acceptability of proposed plant-specific values.

At the completion of the review effort, a proof-and-review copy of the technical specifications will be issued by EGBTSB.<sup>54</sup> Each technical review branch will ascertain the acceptability of technical specifications within its area of specialty and shall advise the EGBTSB<sup>55</sup> by memorandum of its findings.

### IV. EVALUATION FINDINGS

The reviewer determines that sufficient information has been provided and that the<sup>56</sup> review supports conclusions of the following type, to be included in the staff's safety evaluation report:

#### PSAR Review

The applicant's submittal concerning technical specifications has been reviewed by the staff. As required by Section 50.34 of 10 CFR Part 50.34,<sup>57</sup> the applicant has provided an identification and justification for the selection of those variables, conditions, or other items which are determined as a result of the preliminary safety analysis and evaluation to be probable subjects of technical specifications for the facility, with special attention given for those items which may significantly influence the final design.

~~We have reviewed the p~~Proposed technical specification items presented in Section 16.0 of the preliminary safety analysis report have been reviewed with the objective of identifying those items that would require special attention at the construction permit stage to preclude the necessity for any significant change in design to support the final technical specifications. The proposed technical specifications items are similar to those developed by the staff as standard technical specifications for plants of a similar design. ~~We have not identified a~~Any items which require special attention at this stage of our review have not been identified.<sup>58</sup>

On this basis, ~~we have~~ it is concluded that the requirements of ~~Section 50.34~~ of 10 CFR Part 50.34 ~~has have~~<sup>59</sup> been met and that the proposed technical specifications items are acceptable.

## FSAR Review

The staff has concluded that the proposed technical specifications, as amended, satisfy the requirements of ~~Sections 50.34 and 50.36~~ of 10 CFR Part ~~50~~50.34 and 10 CFR 50.36.<sup>60</sup> This conclusion is based on the finding that the technical specifications conform to the guidance developed by the staff for plants designed by the (nuclear steam supply system ~~NSSS~~<sup>61</sup> vendor) as contained in NUREG-(number) with appropriate modifications for plant-specific considerations.

## V. IMPLEMENTATION

The following is intended to provide guidance to applicants and licensees regarding the NRC staff's plans for using this SRP section.

This SRP section will be used by the staff when performing safety evaluations of construction permit, operating license, and/or combined license applications submitted by applicants pursuant to 10 CFR 50 or 10 CFR 52.<sup>62</sup> Except in those cases in which the applicant proposes an acceptable alternative method for complying with specified portions of the Commission's regulations, the method described herein will be used by the staff in its evaluation of conformance with Commission regulations.

The provisions of this SRP section apply to reviews of applications docketed six months or more after the date of issuance of this SRP section.<sup>63</sup>

Implementation schedules for conformance to parts of the method discussed herein are contained in the referenced regulations and NUREGs.<sup>64</sup>

## VI. REFERENCES

1. 10 CFR ~~Section~~ 50.34, "Contents of Applications."
2. 10 CFR ~~Section~~<sup>65</sup> 50.36, "Technical Specifications."
3. ~~NUREG-0103, "Standard Technical Specifications for Babcock and Wilcox Pressurized Water Reactors."~~ NUREG-1430, "Standard Technical Specifications, Babcock and Wilcox Plants."<sup>66</sup>
4. ~~NUREG-0212, "Standard Technical Specifications for Combustion Engineering Pressurized Water Reactors."~~ NUREG-1432, "Standard Technical Specifications, Combustion Engineering Plants."<sup>67</sup>

5. ~~NUREG-0123, "Standard Technical Specifications for General Electric Boiling Water Reactors."~~NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4."<sup>68</sup>
6. ~~NUREG-0452, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors."~~NUREG-1431, "Standard Technical Specifications, Westinghouse Plants."<sup>69</sup>
7. NUREG-1434, "Standard Technical Specifications, General Electric Plants, BWR/6."<sup>70</sup>



## SRP Draft Section 16.0

### Attachment A - Proposed Changes in Order of Occurrence

Item numbers in the following table correspond to superscript numbers in the redline/strikeout copy of the draft SRP section.

Item	Source	Description
1.	Current primary review branch name and designation	Changed PRB to Technical Specifications Branch (TSB).
2.	Current secondary review branches and designations, Incorporation of PRB Comments	Added seven secondary review branch names and their designations: EMCB, EMEB, ECGB, EELB, HICB, SPLB, SRXB, and the NRR Project Manager. In response to PRB comments, also added PERB, SCSB, SPSB, HHFB, and HQMB (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
3.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
4.	Editorial, Implementation of PRB comment	Eliminated unnecessary pluralization of the acronym for standard technical specifications as recommended by the PRB (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
5.	Editorial, Implementation of PRB comment	Eliminated unnecessary pluralization of the acronym for standard technical specifications as recommended by the PRB (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
6.	Current source for copies of the STS	Changed the source for copies of STS to the GPO Sales Program and NTIS.
7.	Editorial modification	The term "generic" has been deleted. The STS are no longer referred to as the generic STS.
8.	Editorial, Implementation of PRB comment	Added clarification regarding the first use of current STS as recommended by the PRB (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
9.	Editorial modification	Changed the word "used" to "use" to correct the tense.
10.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
11.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
12.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
13.	Editorial	Provided clarification that the paragraph is discussing the PSAR stage information.
14.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.

**SRP Draft Section 16.0**  
Attachment A - Proposed Changes in Order of Occurrence

Item	Source	Description
15.	Editorial modification	Deleted "Licensing." The title "Licensing Project Manager" no longer exists.
16.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
17.	Editorial modification	Changed to eliminate gender-specific reference.
18.	Current primary review branch name	Changed PRB to TSB.
19.	Editorial modification	Deleted "with" to be consistent with a preceding phrase.
20.	Editorial modification	The term "technical reviewers" should not be capitalized since it is not a formal title.
21.	Editorial	Revised to better characterize the items that may be referred to technical reviewers or TSB by the Project Manager.
22.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
23.	Editorial modification	Capitalized "Technical Specifications" to be consistent with the first sentence in this paragraph.
24.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
25.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
26.	Editorial modification	Deleted "and" and inserted a comma because additional phrases have been added to this sentence.
27.	Editorial modification	Changed the names of the branches that currently provide events assessment and operational experience: PECB and PIPB. Deleted the reference to the Office of Inspection and Enforcement.
28.	Current primary review branch designation	Changed PRB to TSB.
29.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
30.	SRP-UDP format item	Added "Review Interfaces" under AREAS OF REVIEW.
31.	Current primary review branch designation	Changed PRB to TSB.
32.	Editorial modification	Deleted "Licensing." The title "Licensing Project Manager" no longer exists.
33.	Editorial modification	Added "as follows" to set up a list of review interfaces.

**SRP Draft Section 16.0**  
Attachment A - Proposed Changes in Order of Occurrence

Item	Source	Description
34.	SRP-UDP format item	Reorganized one paragraph addressing review interfaces into numbered paragraphs 1, 2, and 3. Order and text were preserved and branch designations have been updated.
35.	Editorial modification	Deleted "Licensing." The title "Licensing Project Manager" no longer exists.
36.	Current primary review branch designation	Changed PRB to TSB.
37.	<b>Integrated Impact 1202,</b> Incorporation of PRB Comment	Revised to reflect the process under which technical specification requirements are reviewed. The STS are considered acceptable approach for satisfying the relevant requirements of 10 CFR 50.34 and 10 CFR 50.36 as indicated in subsection II. The amended requirements of 10 CFR 50.36(c)(2) now address explicit criteria for appropriate LCOs in technical specifications (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
38.	Editorial modification	Simplified citation format used to reference the Code of Federal Regulations.
39.	<b>Integrated Impacts 475 and 1202,</b> Incorporation of PRB Comment	Revised to reflect the process under which technical specification requirements are reviewed. The STS are considered acceptable approach for satisfying the relevant requirements of 10 CFR 50.34 and 10 CFR 50.36 as indicated in subsection II. The amended requirements of 10 CFR 50.36(c)(2) now address explicit criteria for appropriate LCOs in technical specifications (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
40.	Integrated Impact No. 475	Updated the NUREG references to improved STS.
41.	<b>Integrated Impact # 475</b>	Added coverage of certified designs which may include new Standard Technical Specifications (e.g., CE System 80+).
42.	<b>Integrated Impact 1202</b>	Revised to reflect that 10 CFR 50.36 has been amended in 1995 to specify criteria for determining appropriate LCOs and that LCOs for items meeting criteria now specified in 10 CFR 50.36 must be provided regardless of the content of the STS until which time the STS have been revised to explicitly consider and address the current requirements of 10 CFR 50.36(c)(2).
43.	SRP-UDP format item	Added subheading and introductory paragraph for "Technical Rationale."
44.	SRP-UDP format item	Added technical rationale for 10 CFR 50.34.

**SRP Draft Section 16.0**  
Attachment A - Proposed Changes in Order of Occurrence

Item	Source	Description
45.	SRP-UDP format item, Incorporation of PRB Comment	Added technical rationale and summary of current relevant requirements of 10 CFR 50.36. The STS are considered acceptable approach for satisfying the relevant requirements of 10 CFR 50.34 and 10 CFR 50.36 as indicated in subsection II. It should also be noted that the amended requirements of 10 CFR 50.36(c)(2) now address explicit criteria for appropriate LCOs in technical specifications (see November 27, 1995 Memorandum to R.W. Borchardt from C.I. Grimes (TAC #M92875)).
46.	Editorial modification	Changed "these" to "the" for readability.
47.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
48.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
49.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
50.	Editorial	Revised to reflect up front that the paragraph applies only to an FSAR review.
51.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
52.	Editorial modification	Deleted the term "generic." The STS are no longer referred to as generic STS.
53.	Editorial modification	Added "technical" before the word "specifications" for clarity and consistency.
54.	Current primary review branch designation	Changed PRB to TSB.
55.	Current primary review branch designation	Changed PRB to TSB.
56.	Editorial modification	Changed to eliminate gender-specific reference.
57.	Editorial modification	Simplified citation format used to reference the Code of Federal Regulations.
58.	Editorial modification	Changed to eliminate use of first person (i.e., we).
59.	Editorial modification	Changed to eliminate use of first person (i.e., we) and to simplify citation format for the Code of Federal Regulations.
60.	Editorial modification	Simplified citation format used to reference the Code of Federal Regulations.
61.	Editorial modification	Defined "NSSS" as "nuclear steam supply system."

**SRP Draft Section 16.0**  
Attachment A - Proposed Changes in Order of Occurrence

Item	Source	Description
62.	SRP-UDP Guidance, Implementation of 10 CFR 52	Added standard sentence to address application of the SRP section to reviews of applications filed under 10 CFR Part 52, as well as Part 50.
63.	SRP-UDP Guidance	Added standard paragraph to indicate applicability of this section to reviews of future applications.
64.	Editorial	Added standard paragraph alluding to implementation schedules and information in listed references.
65.	Editorial modification	Provided correct citation format for the Code of Federal Regulations.
66.	Integrated Impact No. 475	Deleted reference to NUREG-0103 and added to NUREG-1430, the improved version of the B&W STS.
67.	Integrated Impact No. 475	Deleted reference to NUREG-0212 and added to NUREG-1432, the improved version of the CE STS.
68.	Integrated Impact No. 475	Deleted reference to NUREG-0123 and added to NUREG-1433, the improved version of the GE STS, BWR/4.
69.	Integrated Impact No. 475	Deleted reference to NUREG-0452 and added to NUREG-1431, the improved version of the W STS.
70.	Integrated Impact No. 475	Added reference to NUREG-1434, the improved version of the GE STS, BWR/6.

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**SRP Draft Section 16.0**  
Attachment B - Cross Reference of Integrated Impacts

Integrated Impact No.	Issue	SRP Subsections Affected
475	Update referenced NUREGS for the improved STS, and delete references to the original STS.	Subsection II, ACCEPTANCE CRITERIA, II.A  Subsection VI, REFERENCES, References 3, 4, 5, 6, and 7.
476	Develop review procedures to include the four criteria for technical specifications contained in the Commission's Final Policy Statement on Technical Specification Improvements.	No changes have been made for this Integrated Impact.
479	Develop review procedures for review of line-item improvements to technical specifications.	No changes have been made for this Integrated Impact.
480	Develop review procedures to provide guidance for review of shutdown risk model technical specification improvements.	No changes have been made for this Integrated Impact.
1202	Revise the SRP as appropriate to reflect the current requirements of 10 CFR 50.36 which have been recently amended to provide criteria for determining appropriate LCOs to be included in Technical specifications.	AREAS OF REVIEW, Review Interface 2  ACCEPTANCE CRITERIA, subsection II.A first paragraph and subsection II.B