

**Application for Renewed Operating License - Palisades Nuclear Plant
Precedent Tables for Section 3.1, Reactor Coolant System**

Table 3.1.2-1 Reactor Coolant System - Primary Coolant System - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 NUREG - 1801 Volume 1 Table Row (Table # - Row #)	Rev 1 NUREG - 1801 Notes
Alloy 600 Cladding	Shelter / Protection	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.D1.1-i	3.1.1-44	C			
					Water Chemistry Program	IV.D1.1-i	3.1.1-44	C			
Alloy 600 Safe Ends	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.C2.2-f	3.1.1-36	A, 123			
						IV.C2.5-h	3.1.1-36	A			
					Water Chemistry Program	IV.C2.2-f	3.1.1-36	A, 123			
						IV.C2.5-h	3.1.1-36	A			
Alloy 600 Thermal Sleeves	Shelter / Protection	Alloy 600/690	Treated Water (Ext)	Cracking	Alloy 600 Program	IV.C2.5-k	3.1.1-14	C, 124			
					Water Chemistry Program	IV.C2.5-k	3.1.1-14	C, 124			
Bolting and Fasteners	Fluid Pressure Boundary	Low Alloy Steel	Containment Air (Ext)	Loss of Preload	Bolting Integrity Program	IV.C2.4-g	3.1.1-27	101, A			

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				Loss of Material	Bolting Integrity Program			F	V.E-4	2-25	A
					Boric Acid Corrosion Program	IV.C2.1-d	3.1.1-39	C			
						IV.C2.2-d	3.1.1-39	C			
			Plant Indoor Air (Ext)	Loss of Preload	Bolting Integrity Program	IV.C2.4-g	3.1.1-27	A			
				Loss of Material	Bolting Integrity Program			F	V.E-4	2-25	A
					Boric Acid Corrosion Program	IV.C2.1-d	3.1.1-39	C			
						IV.C2.2-d	3.1.1-39	C			
Carbon Steel Nozzles	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.2-d	3.1.1-38	C			

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Carbon Steel Pipe (30" and 42")	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.1-d	3.1.1-38	A			
Flow Element (PCP Controlled Bleed)	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.2-f	3.1.1-36	C			
					Water Chemistry Program	IV.C2.2-f	3.1.1-36	C			
Non-CASS Valves in PCS and Connected Systems	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.2-f	3.1.1-36	C			
					Water Chemistry Program	IV.C2.2-f	3.1.1-36	C			
						IV.C2.2-f	3.1.1-36	C			
PCS Spray and Drain Nozzles	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.C2.5-k	3.1.1-14	C			

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						IV.C2.5-s	3.1.1-14	C			
					Water Chemistry Program	IV.C2.5-k	3.1.1-14	C			
						IV.C2.5-s	3.1.1-14	C			
PORV Isolation, Quench Tank Spray Manual Valves	Fluid Pressure Boundary	Cast Austenitic SS	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.4-b	3.1.1-36	A			
					Water Chemistry Program	IV.C2.4-b	3.1.1-36	A			
PORV Isolation Valves	Fluid Pressure Boundary	Cast Austenitic SS	Treated Water (Int)	Reduction of Fracture Toughness	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.4-c	3.1.1-23	A			
Pressurizer Alloy 600 Instrument Penetrations	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.C2.5-k	3.1.1-14	A			
					Water Chemistry Program	IV.C2.5-k	3.1.1-14	A			

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Pressurizer Heater Sleeves	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.C2.5-s	3.1.1-14	A			
					Water Chemistry Program	IV.C2.5-s	3.1.1-14	A			
Pressurizer Heaters	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Int)	Cracking	Alloy 600 Program	IV.C2.5-s	3.1.1-14	A			
					Water Chemistry Program	IV.C2.5-s	3.1.1-14	A			
Pressurizer Integral Support Weld	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.5-v	3.1.1-41	A			
				Loss of Material	Boric Acid Corrosion Program	IV.C2.5-u	3.1.1-38	A			
Pressurizer Manway and Flange Bolting	Fluid Pressure Boundary	Low Alloy Steel	Containment Air (Ext), Borated Water or Steam Leakage	Loss of Material	Boric Acid Corrosion Program	IV.C2.5-o	3.1.1-38	A			

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			Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.5-o	3.1.1-38	A			
				Loss of Preload	Bolting Integrity Program	IV.C2.5-p	3.1.1-26	101, A			
Pressurizer Manway and Flanges	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext), Borated Water or Steam Leakage	Loss of Material	Boric Acid Corrosion Program	IV.C2.5-o	3.1.1-38	A			
Pressurizer Quench Tank	Fluid Pressure Boundary	Epoxy Coated Carbon Steel	Containment Air (Ext)	Loss of Material	System Monitoring Program	IV.C2.6-b	3.1.1-38	E			
			Treated Water (Int)	Loss of Material	One-Time Inspection Program			F	VII.A3-8	3-22	E
Pressurizer Quench Tank Shell and Heads	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.6-b	3.1.1-38	A			
Pressurizer Spray Head	Spray Pattern	Alloy 600/690	Treated Water (Int)	Cracking	One-Time Inspection Program	IV.C2.5-j	3.1.1-12	A			

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					Water Chemistry Program	IV.C2.5-j	3.1.1-12	A			
Primary Coolant Pump Casing	Fluid Pressure Boundary	Cast Austenitic SS	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.3-b	3.1.1-36	A			
					Water Chemistry Program	IV.C2.3-b	3.1.1-36	A			
				Reduction of Fracture Toughness	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.3-c	3.1.1-23	A			
Primary Coolant Pump Closure Bolting	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Preload	Bolting Integrity Program	IV.C2.3-g	3.1.1-27	101, A			
				Loss of Material	Boric Acid Corrosion Program	IV.C2.3-f	3.1.1-39	A			
Primary Coolant Sample Heat Exchanger Shell	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.2-d	3.1.1-38	A			

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Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 NUREG - 1801 Volume 1 Table Row (Table # - Row #)	Rev 1 NUREG - 1801 Notes
					System Monitoring Program	VII.I.1-b	3.1.1-05	A			
			Treated Water (Int)	Loss of Material	Closed Cycle Cooling Water Program	V.D1.5-a	3.1.1-13	C			
Reactor Head Vent	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-g	3.1.1-07	C			
					Water Chemistry Program	IV.C2.1-g	3.1.1-07	C			
Reactor Head Vent Orifice	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-g	3.1.1-07	A			
					Water Chemistry Program	IV.C2.1-g	3.1.1-07	A			
Sample Point (Quench Tank Liquid, Loop 2 Hot Leg, Pressurizer Surge Line)	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-g	3.1.1-07	C			

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						IV.C2.2-h	3.1.1-07	C			
					Water Chemistry Program	IV.C2.1-g	3.1.1-07	C			
						IV.C2.2-h	3.1.1-07	C			
Small Bore Stainless Steel Pipe (PCS and Connected Systems)	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-g	3.1.1-07	A			
						IV.C2.2-h	3.1.1-07	A			
					Water Chemistry Program	IV.C2.1-g	3.1.1-07	A			
						IV.C2.2-h	3.1.1-07	A			
SS Cladding	Shelter/ Protection	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-c	3.1.1-36	A			

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						IV.C2.5-c	3.1.1-36	A			
						IV.C2.5-g	3.1.1-36	A			
						IV.C2.5-h	3.1.1-36	A			
						IV.C2.5-m	3.1.1-36	A			
					Water Chemistry Program	IV.C2.1-c	3.1.1-36	A			
						IV.C2.5-c	3.1.1-36	A			
						IV.C2.5-g	3.1.1-36	A			
						IV.C2.5-h	3.1.1-36	A			
						IV.C2.5-m	3.1.1-36	A			

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Stainless Steel Pipe (PCS and Connected Systems)	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.1-c	3.1.1-36	A			
Stainless Steel Safe Ends (Pressurizer and Connected Systems)	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.2-f	3.1.1-36	A			

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Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 NUREG - 1801 Volume 1 Table Row (Table # - Row #)	Rev 1 NUREG - 1801 Notes
						IV.C2.5-h	3.1.1-36	A			
Stainless Steel Thermal Sleeves	Shelter / Protection	Stainless Steel	Treated Water (Ext)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.5-h	3.1.1-36	C			
					Water Chemistry Program	IV.C2.5-h	3.1.1-36	C			
			Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.5-h	3.1.1-36	C			
					Water Chemistry Program	IV.C2.5-h	3.1.1-36	C			
Stainless Steel Tubing	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.C2.2-f	3.1.1-36	A			
					Water Chemistry Program	IV.C2.2-f	3.1.1-36	A			
Vessels, Pressure (Pressurizer)	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.C2.5-b	3.1.1-38	A			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
CRDM Upper Pressure Housing & Flange	Pressure Boundary/ Fission Product Retention	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.2-b	3.1.1-36	A			
					Water Chemistry Program	IV.A2.2-b	3.1.1-36	A			
CRDM/Incore Instrument Bolting	Pressure Boundary/ Fission Product Retention	Stainless Steel	Containment Air (Ext)	Loss of Material	Bolting Integrity Program	IV.A2.2-f	3.1.1-26	A			
				Loss of Preload	Bolting Integrity Program	IV.A2.2-g	3.1.1-26	101, A			
				Cracking	Bolting Integrity Program	IV.A2.2-e	3.1.1-26	A			
Incore Instrument Closure Flanges	Pressure Boundary/ Fission Product Retention	Stainless Steel	Containment Air (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.2-f	3.1.1-26	C			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
			Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.2-b	3.1.1-36	C			
					Water Chemistry Program	IV.A2.2-b	3.1.1-36	C			
Internal SS Cladding	Pressure Boundary/ Fission Product Retention	Stainless Steel	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.4-b	3.1.1-36	C			
					Water Chemistry Program	IV.A2.4-b	3.1.1-36	C			
Reactor Vessel Column Support	Component Structural Support	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.8-b	3.1.1-38	119, 122, A			
Reactor Vessel Bottom Head	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
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Reactor Vessel Closure Head	Component Structural Support Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.5-f	3.1.1-40	C			
					Boric Acid Corrosion Program	IV.A2.1-a	3.1.1-38	A			
Reactor Vessel Closure Head Lifting Lugs	Component Structural Support	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.1-a	3.1.1-38	C			
Reactor Vessel Core Stabilizer Lugs	Structure Functional Support	Nickel-Based Alloys	Treated Water (Ext)	Cracking	Water Chemistry Program	IV.A2.6-a	3.1.1-12	A			
					Reactor Vessel Internals Inspection Program	IV.A2.6-a	3.1.1-12	A			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program			110, H	IV.B3-22	1-52	C

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
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Reactor Vessel CRDM Nozzles	Structure Functional Support Pressure Boundary/ Fission Product Retention	Nickel-Based Alloys	Treated Water (Int)	Cracking	Alloy 600 Program	IV.A2.2-a	3.1.1-35	A			
					Water Chemistry Program	IV.A2.2-a	3.1.1-35	A			
Reactor Vessel Flow Skirt	Reduce Flow Inequalities	Nickel-Based Alloys	Treated Water (Ext)	Cracking	Water Chemistry Program	IV.A2.6-a	3.1.1-12	E			
					Reactor Vessel Internals Inspection Program	IV.A2.6-a	3.1.1-12	A			
Reactor Vessel Head O-ring Leakage Monitoring	Pressure Boundary/ Fission Product Retention	Nickel-Based Alloys	Treated Water (Int)	Cracking	Alloy 600 Program	IV.A2.1-f	3.1.1-12	A			

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Reactor Vessel Head Vent Nozzle	Pressure Boundary/ Fission Product Retention	Nickel-Based Alloys	Treated Water (Int)	Cracking	Alloy 600 Program	IV.A2.7-b	3.1.1-35	A			
					Water Chemistry Program	IV.A2.7-b	3.1.1-35	A			
Reactor Vessel Incore Instrument Nozzles	Pressure Boundary/ Fission Product Retention	Nickel-Based Alloys	Treated Water (Int)	Cracking	Alloy 600 Program	IV.A2.7-b	3.1.1-35	121, A			
					Water Chemistry Program	IV.A2.7-b	3.1.1-35	A			
Reactor Vessel Intermediate Shell	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			
				Reduction in Fracture Toughness	Reactor Vessel Integrity Surveillance Program	IV.A2.5-c	3.1.1-05	109, A			

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Reactor Vessel Lower Shell	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			
				Reduction in Fracture Toughness	Reactor Vessel Integrity Surveillance Program	IV.A2.5-c	3.1.1-05	109, A			
Reactor Vessel Nozzle Safe Ends	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			
Reactor Vessel Primary Coolant Nozzles	Pressure Boundary/ Fission Product Retention Structure Functional Support	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Reactor Vessel Seal Ledge Ring	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	C			
Reactor Vessel Stop Lugs	Prevent Core Displacement	Nickel-Based Alloys	Treated Water (Ext)	Cracking	Water Chemistry Program	IV.A2.6-a	3.1.1-12	A			
					Reactor Vessel Internals Inspection Program	IV.A2.6-a	3.1.1-12	A			
Reactor Vessel Studs, Nuts, Washers	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Crack Initiation and Growth	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.1-c	3.1.1-22	A			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.1-d	3.1.1-47	C			
					Boric Acid Corrosion Program	IV.A2.1-a	3.1.1-38	A			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Reactor Vessel Surveillance Capsule Holder	Structure Functional Support	Nickel-Based Alloys	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.A2.6-a	3.1.1-12	A			
					Water Chemistry Program	IV.A2.6-a	3.1.1-12	C			
Reactor Vessel Upper Shell	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.5-f	3.1.1-40	A			
					Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	A			
Reactor Vessel Upper Shell Flange	Pressure Boundary/ Fission Product Retention	Carbon Steel	Containment Air (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.5-f	3.1.1-40	A			
					Boric Acid Corrosion Program	IV.A2.5-e	3.1.1-38	A			

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Table 3.1.2-2 Reactor Coolant System - Reactor Vessel - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Under Head CRDM Support	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.A2.2-b	3.1.1-36	C			
					Water Chemistry Program	IV.A2.2-b	3.1.1-36	C			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Control Rod Shroud Assembly = IV.B3.2 - CEA Shroud Assemblies											
Control Rod Shroud	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.2-c	3.1.1-11	A, 113			
Control Rod Shroud, Shroud Top Support, Shroud Support Lug	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.2-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.2-a	3.1.1-45	A			
Control Rod Support Lug, Fuel Guide Pin, Fuel Guide Pin Nuts	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.2-c	3.1.1-11	A, 113			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Fuel Guide Pin, Fuel Guide Pin Nuts	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.2-b	3.1.1-45	A			
					Water Chemistry Program	IV.B3.2-b	3.1.1-45	A			
Fuel Guide Pin, Fuel Guide Pin Nuts, Fuel Plate Cap Screw	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Loss of Preload	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	V.B3.2-g	3.1.1-48	101, A			
Core Shroud Assembly = IV.B3.4 - Core Shroud Assembly											
Anchor Block, Centering Plate, Core Shroud Plate	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.4-b	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.4-a	3.1.1-45	A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
					Water Chemistry Program	IV.B3.4-a	3.1.1-45	A			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.4-c	3.1.1-43	A, 113			
Anchor Screw & Pin, Centering Screw & Pin, Positioning Screw, Shroud Bolt & Pin	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.4-e	3.1.1-45	A			
					Water Chemistry Program	IV.B3.4-e	3.1.1-45	A			
				Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.4-f	3.1.1-11	A, 113			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.4-g	3.1.1-43	A, 113			
				Loss of Preload	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.4-h	3.1.1-48	101, A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Core Support Barrel Assembly = IV.B3.3 - Core Support Barrel Assembly											
Core Support Barrel	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.3-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.3-a	3.1.1-45	A			
Core Support Barrel Integral Upper Flange	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.3-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.3-a	3.1.1-45	A, 113			
				Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.3-b	3.1.1-11	A, 113			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Incore Instrument Guide Tube (This group of components is not addressed in GALL IV.B3)											
Guide Tube Plug Screw	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Loss of Preload	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.2-g	3.1.1-48	E, 103			
Instrument Guide Tube, Guide Tube Bracket	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	C			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Instrument Guide Tube, Guide Tube Bracket, Guide Tube Plugs, Guide Tube Plug Screw, Guide Tube Support	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.1-b	3.1.1-11	C, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.1-a	3.1.1-45	C			
					Water Chemistry Program	IV.B3.1-a	3.1.1-45	C			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	C			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.3-a	3.1.1-45	C, 113			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Lower Internal Assembly (Integral with Core Barrel Assembly) = IV.B3.5 - Lower Internal Assembly											
Core Support Barrel Cap Screws	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.5-c	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.5-b	3.1.1-45	A			
					Water Chemistry Program	IV.B3.5-b	3.1.1-45	A			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.5-d	3.1.1-11	A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Core Support Barrel Snubber Lug	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.5-c	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.5-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.5-a	3.1.1-45	A			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.5-e	3.1.1-40	A			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.5-d	3.1.1-11	A			
Core Support Column	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.5-c	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.5-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.5-a	3.1.1-45	A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Core Support Column Support Beams and Tie Rods	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.5-c	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.5-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.5-a	3.1.1-45	A			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.5-d	3.1.1-11	A			
Core Support Plate	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.5-c	3.1.1-11	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.5-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.5-a	3.1.1-45	A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.5-d	3.1.1-11	A			
Upper Guide Structure - Not in GALL											
Instrument Sleeve	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.1-b	3.1.1-11	C, 113			
				Reduction in Fracture Toughness	Reactor Vessel Internals Inspection Program	IV.B3.2-e	3.1.1-43	F	IV.B3-7	1-13	C
Spacer Shim, Instrument Sleeve	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Cracking	Reactor Vessel Internals Inspection Program	IV.B3.1-a	3.1.1-45	C			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	C			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Upper Internal Assembly = GALL IV.B3.1 - Upper Internal Assembly											
Brace Grid Beam, Cross Brace Screw, Shroud Grid Ring	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.1-b	3.1.1-45	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.1-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.1-a	3.1.1-45	A			
Fuel Alignment Plate	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.1-b	3.1.1-45	A, 113			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.1-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.1-a	3.1.1-45	A			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	A			
Fuel Plate Align Lug, Fuel Plate Cap Screw, Fuel Plate Guide Pin	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Changes in Dimensions	Reactor Vessel Internals Inspection Program	IV.B3.1-b	3.1.1-45	A, 113			
				Cracking	Reactor Vessel Internals Inspection Program	IV.B3.1-a	3.1.1-45	A			
					Water Chemistry Program	IV.B3.1-a	3.1.1-45	A			
				Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	A			

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Table 3.1.2-3 Reactor Coolant System - Reactor Vessel Internals - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Holddown Ring Plunger, Holddown Ring Strap, Holddown Ring	Structure Functional Support	Stainless Steel	Treated Water (Ext)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.B3.1-c	3.1.1-40	A			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Alloy 690 Tube Plugs	Fluid Pressure Boundary	Alloy 600/690	Treated Water (Ext)	Cracking	Steam Generator Tube Integrity Program	IV.D1.2-i	3.1.1-18	A			
					Water Chemistry Program	IV.D1.2-i	3.1.1-18	A			
			Treated Water (Int)	Cracking	Steam Generator Tube Integrity Program	IV.D1.2-i	3.1.1-18	A			
					Water Chemistry Program	IV.D1.2-i	3.1.1-18	A			
Handhole Cover	Fluid Pressure Boundary	Low-Alloy Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
			Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	C			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	C			
Tube Bundle Support Assembly	Structural Support for Safety Related	Stainless Steel	Treated Water (Ext)	Cracking	Water Chemistry Program			F	IV.D1-9	1-61	A
					Steam Generator Tube Integrity Program			118, F	IV.D1-9	1-61	A
		Carbon Steel	Treated Water (Ext)	Loss of Material	Water Chemistry Program	IV.D1.2-k	3.1.1-20	A			
					Steam Generator Tube Integrity Program	IV.D1.2-k	3.1.1-20	118, A			
Tube Bundle Wrapper	Direct Flow	Low Alloy Steel	Treated Water (Int)	Loss of Material	Water Chemistry Program			F	IV.D1-8	1-65	E

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Lower Head	Fluid Pressure Boundary	Carbon Steel w/SS cladding	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	A			
			Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-i	3.1.1-44	C			
					Water Chemistry Program	IV.D1.1-i	3.1.1-44	C			
Primary Manway Cover	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-k	3.1.1-38	A			
Manway Cover Diaphragm	Fluid Pressure Boundary	Stainless Steel	Treated Water (Int)	Cracking	Water Chemistry Program	IV.D1.1-i	3.1.1-44	E			
Nozzle Safe Ends	Fluid Pressure Boundary	Carbon Steel w/ SS Cladding	Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-i	3.1.1-44	A			
					Water Chemistry Program	IV.D1.1-i	3.1.1-44	A			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
			Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
Primary Divider Plate	Fluid Pressure Boundary	Stainless Steel	Treated Water (Ext)	Cracking	Water Chemistry Program	IV.D1.1-i	3.1.1-44	C			
Primary Inlet and Outlet Nozzles	Fluid Pressure Boundary	Carbon Steel w/SS cladding	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
			Treated Water (Int)	Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-i	3.1.1-44	A			
					Water Chemistry Program	IV.D1.1-i	3.1.1-44	A			
Feedwater Inlet Nozzles and Thermal Sleeves	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
			Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	C			
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	C			
					Flow Accelerated Corrosion Program	IV.D1.1-d	3.1.1-25	117, A			
Steam Outlet Nozzle and Flow Limiter, Blowdown Nozzle	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
			Treated Water (Int)	Loss of Material	Water Chemistry Program	IV.D1.1-c	3.1.1-02	E			
					Flow Accelerated Corrosion Program	IV.D1.1-d	3.1.1-25	117, A			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
Secondary Side 2" Inspection Port Cover	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-k	3.1.1-38	C			
			Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	A			
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	A			
Wide and Narrow Range Water level Nozzles, Sampling and Instrument Nozzles	Fluid Pressure Boundary	Low Alloy Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
			Treated Water (Int)	Loss of Material	Water Chemistry Program	IV.D1.1-c	3.1.1-02	C			
				Cracking	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	E			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	E			
Shells (Lower, Upper, Transition)	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	A			
			Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	A			
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	A			
					Steam Generator Tube Integrity Program	IV.D1.1-c	3.1.1-02	B			
Tubesheet	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	114, C			
			Treated Water (Int) (Secondary)	Loss of Material	Steam Generator Tube Integrity Program	IV.D1.1-c	3.1.1-02	C			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
					Water Chemistry Program	IV.D1.1-c	3.1.1-02	C			
		Nickel Based Alloy	Treated Water (Int) (Primary)	Cracking	Steam Generator Tube Integrity Program	IV.D1.2-a	3.1.1-18	C			
					Water Chemistry Program	IV.D1.2-a	3.1.1-18	C			
Upper Head	Fluid Pressure Boundary	Carbon Steel	Containment Air (Ext)	Loss of Material	Boric Acid Corrosion Program	IV.D1.1-g	3.1.1-38	C			
			Treated Water (Int)	Loss of Material	ASME Section XI IWB, IWC, IWD, IWF Inservice Inspection Program	IV.D1.1-c	3.1.1-02	A			
				Loss of Material	Water Chemistry Program	IV.D1.1-c	3.1.1-02	A			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
U-Tubes	Fluid Pressure Boundary Heat Transfer	Alloy 600/690	Treated Water (Ext)	Cracking	Steam Generator Tube Integrity Program	IV.D1.2-b	3.1.1-18	A			
						IV.D1.2-c	3.1.1-18	A			
					Water Chemistry Program	IV.D1.2-b	3.1.1-18	A			
						IV.D1.2-c	3.1.1-18	A			
				Loss of Material	Water Chemistry Program	IV.D1.2-e	3.1.1-18	A			
					Steam Generator Tube Integrity Program	IV.D1.2-e	3.1.1-18	A			

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Table 3.1.2-4 Reactor Coolant System - Replacement Steam Generators - Summary of Aging Management Evaluation											
Component Type	Intended Function	Material	Environment	Aging Effect Requiring Management	Aging Management Programs	NUREG - 1801 Volume 2 Line Item	Table 1 Item	Notes	Rev 1 NUREG - 1801 Volume 2 Line Item	Rev 1 Table 1 Item	Rev 1 Notes
			Treated Water (Int)	Cracking	Steam Generator Tube Integrity Program	IV.D1.2-a	3.1.1-18	A			
					Water Chemistry Program	IV.D1.2-a	3.1.1-18	A			
Manway Fasteners	Fluid Pressure Boundary	Carbon Steel / Low Alloy Steel	Containment Air (Ext)	Crack Initiation and Growth	Bolting Integrity Program	IV.D1.1-l	3.1.1-26	A			
				Loss of Material	Boric Acid Corrosion Program	IV.D1.1-k	3.1.1-38	A			
				Loss of Preload	Bolting Integrity Program	IV.D1.1-f	3.1.1-26	101, A			

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