

July 13, 2005

LICENSEE: Nuclear Management Company, LLC

FACILITY: Monticello Nuclear Generating Plant

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON JULY 6, 2005,
BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND
NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT
REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE
MONTICELLO NUCLEAR GENERATING PLANT LICENSE RENEWAL
APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference on July 6, 2005, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Monticello Nuclear Generating Plant license renewal application. The conference call was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/

Daniel J. Merzke, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-263

Enclosures: As stated

cc w/encls: See next page

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Monticello Nuclear Generating Plant

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Dated: July 13, 2005

ADAMS Accession No.: **ML051950060**

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TO DISCUSS THE MONTICELLO NUCLEAR GENERATING PLANT
LICENSE RENEWAL APPLICATION
JULY 6, 2005**

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NRC
Nuclear Management Company, LLC (NMC)
NMC
NMC
NMC

**DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAIs)
MONTICELLO NUCLEAR GENERATING PLANT
LICENSE RENEWAL APPLICATION
July 6, 2005**

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on July 6, 2005 to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Monticello Nuclear Generating Plant license renewal application (LRA). The following D-RAIs were discussed during the telephone conference call.

D-RAI 3.2-1

In Table 3.2.2-4 of the LRA, the applicant proposes to manage the aging effect of heat transfer degradation due to fouling of the copper alloy heat exchanger tubes in an external lubricating oil environment with a One-Time Inspection Program. The staff requests the applicant provide the following:

- (a) Specific material composition of the copper alloys.
- (b) Oil analysis program and/or other methods to ensure that the lubricating oil remains free of contaminants which might degrade the tubing.
- ©) Preventive maintenance procedures to ensure that heat transfer degradation does not reach unacceptable levels.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 3.2-2

In Table 3.2.2-4 of the LRA, the applicant states that the heat transfer degradation due to fouling of the copper alloy heat exchanger tubes in a steam environment will be managed with Plant Chemistry and One-Time Inspection Programs. The applicant further states that neither the components nor the material and environment combination are evaluated in NUREG-1801. The staff requests the applicant verify that the steam in the heat exchangers identified above originates from treated water. In addition, the applicant is requested to provide justification for not considering erosion and flow accelerated corrosion (FAC) as aging mechanisms for this environment and material combination.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 3.4-1

In Table 3.4.2-2 of the LRA, the applicant has identified no aging effects requiring management for rubber expansion joints intended to maintain the pressure boundary function in a plant indoor air environment. The applicant states that neither the components nor the material and environment combination are evaluated in NUREG-1801. The applicant further states that these elastomer components (neoprene, rubber, etc.) are indoors and not subject to ultra-violet rays or ozone, nor are they in locations that are subject to radiation exposure. These locations are also not subject to temperatures where changes in material properties or cracking could occur (>95 degrees F). Therefore, the applicant contends that no aging management is required.

However, it is industry experience that elastomeric expansion joints degrade due to oxidation in environments that are not necessarily harsh, as discussed in EPRI Report 1008035, "Expansion Joint Maintenance Guide" Revision 1, May 2003, and EPRI Report 1007933, "Aging Assessment Field Guide," December 2003. The staff therefore requests the applicant discuss their inspection procedures for the rubber expansion joints related to preventive maintenance, both for external and internal surfaces, of the elastomer.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 3.4-2

In Table 3.4.2-3 of the LRA, the applicant has identified the aging effects of changes in material properties and cracking due to irradiation and thermal exposure for rubber expansion joints in an internal steam environment. The intended function of the expansion joints is to maintain holdup of radioactive material. The applicant states that neither the components nor the material and environment combination are evaluated in NUREG-1801. The applicant further states that the aging effect/mechanism is applicable but does not require management since the intended function for this component is post-accident iodine plate-out and holdup. According to the applicant, main condenser structural integrity is continuously demonstrated during normal plant operation, thus the intended function is maintained. However, the staff position is that since this component type (rubber expansion joint) is within the scope of license renewal, its aging effects should be managed. Therefore, the applicant is requested to provide the appropriate aging management program to manage the aging effect of changes in material properties and cracking due to irradiation and thermal expansion of the rubber expansion joints in a steam environment.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.9-1

In Section 4.9 of the LRA related to the reactor building crane load cycles TLAA, the applicant contends that the current analysis of the fatigue life remains valid for the 60 year extended operating period. It is the staff's understanding that this crane will also handle spent fuel pool shipping casks. A refueling service platform with handling and grappling fixtures services the refueling area and the spent fuel pool. The applicant is requested to provide a fatigue analysis associated with lifts of the spent fuel casks and explain how the heavy load fatigue analysis provided in Section 4.9 of the LRA is the governing analysis for the TLAA.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.