

June 29, 2005

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10 CFR 54

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
License Nos. DPR-24 and DPR-27

Final Update Information Regarding the
Point Beach Nuclear Plant License Renewal Application
(TAC Nos. MC2099 and MC2100)

By letter dated February 25, 2004, Nuclear Management Company, LLC (NMC), submitted the Point Beach Nuclear Plant (PBNP) Units 1 and 2 License Renewal Application (LRA). In accordance with 10 CFR 54.21(b), NMC is submitting the following update to the LRA at least three months prior to the scheduled completion of the LRA review by the NRC.

NMC has completed a review of PBNP current licensing basis (CLB) changes made since the time of the annual update to the PBNP LRA, including the Final Safety Analysis Report (FSAR) supplement, that could materially affect the content of the LRA. The following CLB changes have been made since the annual update to the PBNP LRA that materially affect the LRA.

The recent installation of a new reactor vessel head on PBNP Unit 2 did include some new structural components (missile shield and associated supports). Each of these components is covered as structural commodities in the LRA and is age-managed under the Boric Acid Corrosion Program and Structures Monitoring Program. The materials, environments, aging effects and the appropriate aging management programs are already contained in LRA Table 3.5.2-1 page 3-437. Therefore, there are no material effects to the PBNP LRA as a result of the new reactor vessel head.

The Unit 2 split pins (original material X-750) have also been replaced during the recent refueling outage on the reactor internals package with a 316 stainless steel material. Aging management of these components will be consistent with those of other reactor internals package components made of the same material under the Reactor Vessel

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Internals Program. There are no material effects to the PBNP LRA as a result of the split pin material change.

NMC has consolidated the various site-specific Quality Assurance (QA) programs into a common fleet QA program. NMC-1, "Quality Assurance Topical Report," was reviewed and approved by the NRC via Safety Evaluation Reports dated January 13 and March 24, 2005. Portions of Appendix A – FSAR Supplement to the LRA are revised by this letter to reflect this change. The FSAR Supplement change is as follows:

LRA APPENDIX A - FSAR SUPPLEMENT

BACKGROUND

A need for revision of the FSAR Supplement was identified as a result of the adoption of a common fleet QA program for PBNP. The new FSAR Chapter 15 discussion provides documentation of that change in the CLB.

DESCRIPTION OF CHANGES TO LICENSE RENEWAL APPLICATION

(additions are double-underlined; deletions are strikethrough)

Revised text as shown on LRA page A-22 and A-23.

FSAR CHAPTER 15

15.1 PROGRAMS THAT MANAGE THE EFFECTS OF AGING AND GENERIC QUALITY ASSURANCE PROGRAM REQUIREMENTS

Corrective Actions

Corrective actions are implemented in accordance with the requirements of 10 CFR 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," and ANSI N18.7-1976, "~~Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants,~~" as committed in Section 1.4 of the PBNP Final Safety Analysis Report (FSAR) NMC-1, "Quality Assurance Topical Report."

Confirmation Process

The confirmation process is part of the corrective action program, which is implemented in accordance with the requirements of 10 CFR 50, Appendix B, and ANSI N18.7-1976, as committed in Section 1.4 of the PBNP FSAR NMC-1, "Quality Assurance Topical Report."

Administrative Controls

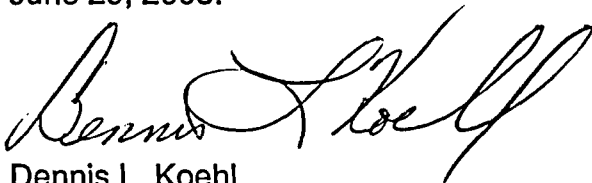
Aging management programs are implemented through various plant documents. These implementing documents are subject to administrative controls, including a formal review and approval process, in accordance with the requirements of 10 CFR 50, Appendix B, and ANSI N18.7-1976, as committed in Section 1.4 of the PBNP FSAR NMC-1, "Quality Assurance Topical Report."

Finally, NRC has granted license amendment Nos. 218 and 223 to PBNP Unit 1 and Unit 2, respectively, to revise Technical Specification 5.5.6, "Reactor Coolant Pump Flywheel Inspection Program," to increase the inspection interval from 10 years to 20 years. This change was approved by NRC by Safety Evaluation Report dated June 6, 2005. Therefore, there is no material effect on the LRA as a result of the revised reactor coolant pump flywheel inspection interval.

Should you have any questions concerning this submittal, please contact Mr. James E. Knorr at (920) 755-6863.

This letter contains no new commitments and no revisions to existing commitments.

I declare under penalty of perjury that the forgoing is true and correct. Executed on June 29, 2005.



Dennis L. Koehl
Site Vice-President, Point Beach Nuclear Plant
Nuclear Management Company, LLC

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
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