

July 12, 2005

MEMORANDUM TO: Darrell J. Roberts, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Victor Nerses, Senior Project Manager, Section 2 /RA/
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SEABROOK STATION, UNIT NO. 1, DRAFT REQUEST FOR
ADDITIONAL INFORMATION (TAC NO. MC6548)

The enclosed draft request for additional information (RAI) was transmitted by facsimile on July 12, 2005, to Mr. Michael O'Keefe of FPL Energy, LLC (the licensee). This draft RAI was transmitted to facilitate the technical review being conducted by the Nuclear Regulatory Commission (NRC) and to support a conference call with the licensee in order to clarify certain items.

This RAI is related to the licensee's amendment request for Seabrook Station, Unit No. 1 dated March 28, 2005. The proposed amendment would revise the expiration date of the operating license to recapture zero-power and low-power testing time.

Review of the RAI would allow the licensee to determine and agree upon a schedule to respond to the RAI. This memorandum and the attachment do not convey or represent an NRC staff position regarding the licensee's request.

Docket No. 50-443

Enclosure: Draft Request for Additional Information

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DRAFT REQUEST FOR ADDITIONAL INFORMATION
RELATED TO RECAPTURE ZERO-POWER AND LOW-POWER TESTING TIME

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

By letter dated March 28, 2005, FPL Energy Seabrook, LLC (licensee) submitted an amendment request for Seabrook Station, Unit No. 1 (SS). The proposed amendment would revise the expiration date of the operating license to recapture zero-power and low-power testing time.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information the licensee provided that supports the proposed amendment and requests the following additional information to clarify the submittal. If you believe that the requested information has been previously docketed, please provide a specific reference to the document where the information may be found.

1. The proposed amendment would revise the expiration date of the SS operating license to recapture zero- and low-power testing time. In the Technical Analysis section of the license amendment request, the licensee discussed neutron embrittlement effects on the reactor vessel materials. Title 10 of the *Code of Federal Regulations*, Section 50.61, Paragraph (b)(1), (10 CFR 50.61(b)(1)) requires: "For each pressurized water nuclear power reactor for which an operating license has been issued, other than a nuclear power reactor facility for which the certifications required under Sec. 50.82(a)(1) have been submitted, the licensee shall have projected values of RT_{PTS} accepted by the NRC, for each reactor vessel beltline material for the EOL (End of License) fluence of the material." Additionally, 10 CFR 50.61(b)(1) requires: "This assessment must be updated...upon request for a change in the expiration date for the facility."
 - a. Please identify the portion of the License Amendment Request that describes the update required by 10 CFR 50.61. If this information is not included or identified in your March 28, 2005 License Amendment Request, provide an assessment of the RT_{PTS} value for each beltline material at the proposed license expiration date. This assessment should include sufficient information to demonstrate how RT_{PTS} was calculated from materials property data for the SS beltline materials and the projected neutron fluence ($E > 1.0$ MeV) at the proposed license expiration date.
2. Please provide a description of effect the proposed change would have on the reactor vessel surveillance program capsule withdrawal schedule, as required by 10 CFR Part 50, Appendix H. Specifically, identify whether the withdrawal schedule would need to be changed to remain in compliance with the edition of American Society for Testing and Materials (ASTM) E-185 "Standard Practice for Conducting Surveillance Tests for Light-Water Cooled Nuclear Power Reactor Vessels," that is applicable to SS.

Enclosure