

July 5, 2005

Mr. L. M. Stinson
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT (FNP), UNIT 1 - 10 CFR PART 50, APPENDIX H, TEST REPORT REQUIREMENTS: REVIEW OF TOPICAL REPORT WCAP-16221-NP, "ANALYSIS OF CAPSULE V FROM SOUTHERN NUCLEAR OPERATING COMPANY, JOSEPH M. FARLEY, UNIT 1 REACTOR VESSEL RADIATION SURVEILLANCE PROGRAM" (TAC NO. MC3056)

Dear Mr. Stinson:

By letter dated April 16, 2004, Southern Nuclear Operating Company (the licensee) submitted report number WCAP-16221-NP, "Analysis of Capsule V from the Southern Nuclear Operating Company, Joseph M. Farley, Unit 1 Reactor Vessel Radiation Surveillance Program." The report is applicable to the evaluation of the reactor vessel (RV) at FNP, Unit 1, and provides the applicable fracture toughness test data for surveillance capsule V which was removed from FNP, Unit 1, at 20.16 effective full power years of plant operation.

Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix H, "Reactor Vessel Material Surveillance Program Requirements" (the rule), provides the Nuclear Regulatory Commission (NRC) surveillance and testing requirements for ferritic components in the RVs of operating domestic light-water reactors. The rule requires licensed owners of domestic light-water reactor facilities to install a number of surveillance capsules within the cavities of their RVs and to remove capsules and test the capsule materials in accordance with the withdrawal schedule and testing requirements of American Society for Testing and Materials (ASTM) Standard Practice E185-82. Paragraph IV.A. of the rule requires the RV material surveillance capsule test results to be the subject of a summary technical report that is required to be submitted to the NRC within one year of the capsule withdrawal date. Paragraph IV.B. of the rule specifies that these topical reports shall include all data required by ASTM Standard Practice E185 and the results of all fracture toughness tests conducted on the RV beltline materials in both the unirradiated and irradiated condition.

The NRC staff has performed a preliminary review of report number WCAP-16221-NP and has confirmed that the report includes all of the data and test results that are required by Paragraph IV.B of 10 CFR Part 50, Appendix H and by ASTM Standard Practice E185-82. Furthermore, based on this preliminary review, the NRC staff has not identified any immediate safety issues associated with the information provided in report number WCAP-16221-NP. Therefore, the NRC staff intends to forward this report to the Pacific Northwest National Laboratory for the purpose of officially updating the surveillance data in the NRC staff's Reactor Vessel Integrity

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Database. The licensee will be expected to incorporate the updated surveillance data into the next revision of the plant's pressure-temperature limits, as required by 10 CFR Part 50, Appendix G.

This completes the NRC staff's activities under TAC No. MC5068. Please contact me at (301) 415-1493, if you have any other questions on these issues.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-348

cc: See next page

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Joseph M. Farley Nuclear Plant, Units 1 & 2

cc:

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