Duke Power

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June 14, 2005

U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001
ATTENTION: Document Control Desk

Subject: Duke Energy Corporation
Catawba Nuclear Station Unit 1
Docket No. 50-413
Response to Revised NRC Order (EA-03-009) which Established Interim Inspection Requirements for Reactor Pressure Vessel Heads at Pressurized Water Reactors

Revised NRC Order (EA-03-009) which established interim inspection requirements for reactor pressure vessel heads at Pressurized Water Reactors (PWR) required a bare metal visual (BMV) examination of 100 percent of the reactor pressure vessel (RPV) head surface (including 360° around each RPV head penetration nozzle). Additionally, revised NRC Order (EA-03-009) required that a report detailing the inspection results be submitted within 60 days of returning the plant to operation. Participating inspectors must be ASME XI, VT-2 qualified.

During the Catawba Unit 1 Refueling Outage (1EOC15), inspections described in response to IV.C.(3),Pressurized Water Stress Corrosion Cracking (PWSCC) Susceptibility Category (Low), of the revised order were completed. With respect to NRC Order EA-03-009, Catawba Unit 1 is considered a PWSCC low susceptibility plant based on reactor vessel head temperature and operational history. The inspection conformed to the requirements of EPRI Report 1007842, Visual Examination for Leakage of PWR Reactor Head Penetrations, Revision 2, and Order EA-03-009.

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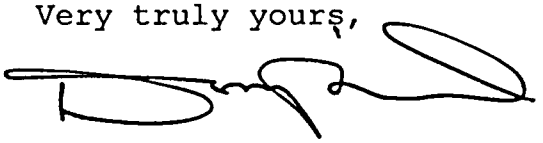
There were no concerns with the structural integrity of the reactor vessel head. The head is in very good condition with little oxidation. Localized areas were observed with minor surface discolorations; however, these have no effect on the structural integrity. Evidence of leakage was solely in the form of oxidation trails and was clearly associated with a historical leak path from above. These same trails were observed in the 1EOC13 inspection. In no case was a concentrated residue or deposit observed. There was no indication or evidence of leakage through the Alloy 600, J-groove weld of the Control Rod Drive Mechanisms or other Alloy 600 locations. The pressure retaining function of the vessel has not been degraded.

These BMV inspections were performed by VT-2 qualified inspectors

There are no NRC commitments contained in this letter.

Inquiries on this matter should be directed to A. Jones-Young at (803) 831-3051.

Very truly yours,



D.M. Jamil
Site Vice President

D. M. Jamil, being duly sworn, affirms that he is the person who subscribed his name to the foregoing statement, and that all matters and facts set forth herein are true and correct to the best of his knowledge.

D. M. Jamil, Site Vice President

Subscribed and sworn to me: 6-14-05

Michy Standridge, Notary Public

My commission expires: 7-10-2012

SEAL

xc:

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