

June 16, 2005

LICENSEE: Nuclear Management Company, LLC

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON MARCH 28, 2005, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION (TAC NOS. MC2099 AND MC2100)

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on March 28, 2005, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application. The conference call was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/

Veronica M. Rodriguez, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

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DISTRIBUTION: Note to Nuclear Management Company, Re: Summary of telecon held on
March 28, 2005, Dated: June 16, 2005

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LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE CALL
TO DISCUSS THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

MARCH 28, 2005

<u>Participant</u>	<u>Affiliation</u>
Brad Fromm	Nuclear Management Company, LLC (NMC)
Jim Knorr	NMC
John Thorgersen	NMC
Mark Ortmyer	NMC
Todd Mielke	NMC
Johnny Eads	U.S. Nuclear Regulatory Commission (NRC)
Ken Chang	NRC
Kurt Cozens	NRC
Michael Morgan	NRC
Muhammad Razzaque	NRC
Robert Hsu	NRC
Samuel Miranda	NRC
Veronica Rodriguez	NRC

DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAI)
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

March 28, 2005

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on March 28, 2005, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application (LRA). The following D-RAIs were discussed during the telephone conference call.

D-RAI 3.1.1-2

The staff believes that the steam generator (SG) feedrings and associated J-tubes should be included in the scope of license renewal. Since this component is completely enclosed by safety-related pressure-boundary components, it is important to show that failures of this component would not impede its designed safety-related functions, as required by 10 CFR 54.4(a)(2).

Industry experience indicates that certain aging effects will degrade these components, and if permitted to continue, could result in loose parts and could interfere with its intended function. Examples of the operating experience information compiled by both the Institute of Nuclear Power Operations (INPO) and World Association of Nuclear Operators (WANO) are presented as follows:

- On April 25, 1989, a Westinghouse Model 51 steam generator was undergoing periodic testing and J-nozzles were found to be eroded/corroded.
- On January 14, 1995, a Westinghouse Model 51 steam generator was undergoing a special leak test of the feedring and water appeared to be flowing from a fault in either the piping or plug located at the support bracket, 180 degrees from the feedwater inlet. Two J-nozzles were removed from the top of the feedring, the area was cleaned, and the damaged weld in the bottom of the feedring was weld repaired. Although there was no failure on the J-nozzles, two existing J-nozzles were replaced with new style Inconel J-nozzles as a preventive measure.

The staff requests the applicant to justify why the SG feedrings and associated J-tubes should not be scoped into the license renewal. If such justification is not available, the staff requests the applicant to provide the aging management program that PBNP will be committing to manage these aging effects.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 3.5-13

- (1) In LRA Section 3.5.2.2.1.1, the applicant stated that concrete degradation in air due to aggressive rainwater is insignificant and the below-grade/lake water environment is

Enclosure 2

non-aggressive. The staff requests the applicant to provide sufficient data to support this statement.

- (2) During its AMR review, the staff was unable to identify how the LRA addresses the items described in the Interim Staff Guidance (ISG)-03. The staff requests the applicant to identify how its AMRs address all the items described ISG-03.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 2.3.1.2-2

The staff believes that the reactor vessel flange leakoff lines should be included in the scope of license renewal. The staff has taken the position (B&W Owners Group, ML993130195) that the reactor vessel flange pressure boundary is at the outboard flange seal. Therefore, the reactor vessel flange leakoff lines are part of the reactor vessel pressure boundary and should be treated as part of the reactor coolant system pressure boundary. As a result, the reactor vessel flange leakoff lines are within the scope of 10 CFR 54.4.

The staff requests the applicant to justify why the reactor vessel flange leakoff lines are not within the scope of 10 CFR 54.4 or to provide the aging management program that will be used to manage the lines during the period of extended operation.

Discussion: The applicant indicated that at Point Beach the reactor vessel flange leakoff lines are nonsafety-related components and therefore if it was to be considered within the scope of license renewal it will be screened under the 10 CFR 54.4(a)(2) criteria. Because the staff is not aware of any credible aging effect mechanism, there is no basis to conclude that this component will degrade and prevent any safety-related component from performing its intended function. Therefore, this question is WITHDRAWN and will not be sent as a formal RAI.