June 17, 2005

LICENSEE: Nuclear Management Company, LLC

FACILITY: Monticello Nuclear Generating Plant

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON JUNE 9, 2005, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE MONTICELLO NUCLEAR GENERATING PLANT LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference on June 9, 2005, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Monticello Nuclear Generating Plant, license renewal application. The conference call was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/ (S. Lee for)

Daniel J. Merzke, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures: As stated

cc w/encls: See next page

June 17, 2005

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DISTRIBUTION: Note to NMC , Re: Teleconference held on June 9, 2005, Dated: June 17, 2005
ADAMS Accession No.: ML051680443

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TO DISCUSS THE MONTICELLO NUCLEAR GENERATING PLANT
LICENSE RENEWAL APPLICATION
JUNE 9, 2005

Participants	Affiliations
Patrick Burke	Nuclear Management Company, LLC (NMC)
Mike Aleksey	NMC
Joe Pairitz	NMC
Ron Siepel	NMC
Daniel Merzke	U.S. Nuclear Regulatory Commission (NRC)
John Fair	NRC

Enclosure 1

DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAIs)
MONTICELLO NUCLEAR GENERATING PLANT
LICENSE RENEWAL APPLICATION

June 9, 2005

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on June 9, 2005, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Monticello Nuclear Generating Plant, license renewal application (LRA). The following D-RAIs were discussed during the telephone conference call.

D-RAI 2.3.1-1

The Control Rod Velocity Limiters are described in Section 6.4 of the USAR. The Control Rod Velocity Limiters are provided as an integral part of each control rod. They provide hydraulic damping to reduce the free fall velocity of the rod and thereby reduce the consequences in the event the control rod became detached from its drive and dropped from the core. The LRA does not mention this component, nor does it appear to refer to USAR Section 6.4 in the text. Please provide justification.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 2.5.1-1

The Safety Parameter Display System is described in Section 7.13 of the MNGP USAR. The purpose of the Safety Parameter Display System (SPDS) is to provide a concise display of critical plant variables to control room operators to aid them in rapidly and reliably determining the safety status of the plant. The LRA does not mention this system, nor does it appear to refer to USAR Section 7.13 in the text. Please provide justification.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.3.1-1

Section 4.3.1 of the license renewal application (LRA) discusses the evaluation of the reactor pressure vessel (RPV) components. The LRA indicates that a reanalysis of the fatigue usage of RPV components was performed as part of the 1998 power uprate. The LRA also indicates that revised fatigue usage factors were determined from the MNGP fatigue monitoring program (FMP). Describe how these revised fatigue usage factors were calculated. Provide a copy of SASR 89-77, "Accumulated Fatigue Usage for the Monticello Nuclear Generating Station Reactor Pressure Vessel," December 1989.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.3.2-1

Section 4.3.2 of the LRA discusses the evaluation of the reactor vessel internals (RVI). The LRA indicates that the 60-year fatigue usage was estimated by multiplying original fatigue usage by a factor of 1.5. Confirm that this extrapolation bounds the number of startup/shutdown design cycles listed in Section 4.3.1 of the LRA.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.3.3-1

Section 4.3.3 of the LRA discusses the evaluation of reactor coolant pressure boundary (RCPB) piping. The LRA indicates that portions of the RCPB were required to be analyzed for fatigue in accordance with the ASME Code Section III for Nuclear Class 1 piping. Provide the basis for this requirement. The LRA further indicates that the design fatigue usage at the limiting location for RCPB core spray piping is less than 0.65 (core spray valve joint). The LRA estimates the 60-year fatigue usage by multiplying the design value by 1.5. Table 4.3.1-1 of the LRA indicates that the projected 60-year fatigue usage of the core spray nozzle is 0.65 based on the FMP. Indicate whether number of thermal transient cycles used to estimate the 60-year fatigue usage of the core spray valve joint is consistent with the number of thermal transient cycles, obtained from the FMP, used to estimate the 60-year fatigue usage of the core spray nozzle.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.5-1

Section 4.5 of the LRA discusses the evaluation of the impact of the reactor water environment on the fatigue life of the locations identified in NUREG/CR-6260, "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components." The environmental fatigue usage for the core spray nozzle (safe end) is much lower than the fatigue usage of the core spray nozzle (without environmental effects) provided in Table 4.3.3-1 of the LRA. Provide the basis for the reported usage factors. In addition, discuss the calculation of the Fen multipliers used for each of the NUREG/CR-6260 locations. Provide the calculated Fen multipliers.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI 4.6.1-1

Section 4.6.1 of the LRA discusses the evaluation of the suppression chamber. The LRA indicates that the maximum fatigue usage for the torus shell was calculated to be 0.98. Provide the number of safety relief valve (SRV) lifts used in this analysis.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.