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June 10, 2005
BW050053

10 CFR 50.54 (f)

United States Nuclear Regulatory Commission
ATTN: Document Control Desk
11555 Rockville Pike
Rockville, Maryland 20852

Braidwood Station, Unit 2
Facility Operating License No. NPF-77
NRC Docket No. STN 50-457

Subject: Braidwood Station, Unit 2 Sixty-Day Response to NRC Bulletin 2004-01, "Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Connections at Pressurized-Water Reactors"

On May 28, 2004, the NRC issued NRC Bulletin 2004-01, "Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Connections at Pressurized-Water Reactors." Requested Information section 2(a) of this bulletin requires the following information be submitted to the NRC within 60 days after plant restart following the next inspection of the Alloy 82/182/600 pressurizer penetrations and steam space piping connections:

> "...a statement indicating that the inspections described in the licensee's response to item (1)(c) of this bulletin were completed and a description of the as-found condition of the pressurizer shell, any findings of relevant indications of through-wall leakage, followup NDE performed to characterize flaws in leaking penetrations or steam space piping connections, a summary of all relevant indications found by NDE, a summary of the disposition of any findings of boric acid, and any corrective actions taken and/or repairs made as a result of the indications found."

Pursuant to 10 CFR 50.54, "Conditions of licenses," paragraph (f), Attachment 1 to this letter provides the Braidwood Station, Unit 2 Sixty-Day response. This response is due to the NRC by July 4, 2005.

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Please direct any questions you may have regarding this submittal to Mr. Dale Ambler, Regulatory Assurance Manager, at (815) 417-2800.

I declare under penalty of perjury that the foregoing is true and correct.

Respectfully,

Executed on 6/9/05

Keith J. Polson
Site Vice President
Braidwood Nuclear Generating Station

Enclosures: Attachment, Braidwood Station Unit 2 Sixty-Day Response to NRC Bulletin 2004-01

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - Braidwood Station

ATTACHMENT

Braidwood Station Unit 2

Sixty-Day Response to NRC Bulletin 2004-01

" Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Connections at Pressurized-Water Reactors "

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> *"...a statement indicating that the inspections described in the licensee's response to item (1)(c) of this bulletin were completed and a description of the as-found condition of the pressurizer shell, any findings of relevant indications of through-wall leakage, followup NDE performed to characterize flaws in leaking penetrations or steam space piping connections, a summary of all relevant indications found by NDE, a summary of the disposition of any findings of boric acid, and any corrective actions taken and/or repairs made as a result of the indications found."*

Response

Summary of the Inspections Performed, Extent of the Inspections, and Methods Used

During A2R11 (Spring 2005 refuel outage), a bare metal visual (BMV) examination was performed on all five of the pressurizer steam space penetrations and adjacent pressurizer shell surfaces. The examination was conducted in accordance with Exelon corporate procedures ER-AA-335-015, "VT-2 Examination", ER-AP-331-1001, "Boric Acid Corrosion Control (BACC) Inspection Locations, Implementation, and Inspection Guidelines," and ER-AP-331-1002, "Boric Acid Corrosion Control Program Identification, Assessment, and Evaluation."

The examination was performed after insulation was removed from the top of the pressurizer and the five associated penetrations. The examination was a direct visual examination within two feet of the area of interest, ensuring any boric acid leakage would be easily identified.

All five penetrations were examined 360 degrees around. The examination was performed real time by certified VT-2 examiners.

Description of the As-Found Condition, Findings of Relevant Indications, and Summary of the Disposition of any Findings

The examination of the five penetrations and adjacent pressurizer vessel surfaces identified no evidence of any boric acid deposits associated with reactor coolant leakage.

Corrective Actions Taken

Based on the acceptable as-found condition of the five penetrations and adjacent pressurizer vessel surfaces there was no followup NDE, disposition of findings, or corrective actions required.