

June 13, 2005

LICENSEE: Nuclear Management Company, LLC

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON MARCH 3, 2005, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION (LRA) (TAC NOS. MC2099 AND MC2100)

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference on March 3, 2005, to discuss and clarify the staff's requests for additional information (RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application. The conference call was useful in clarifying the intent of the staff's RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

**/RA/**

Verónica M. Rodríguez, Project Manager  
License Renewal Section A  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

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DISTRIBUTION: Note to: Nuclear Management Co., Pt. Beach, Units 1 and 2, Re: Summary of telecon held on March 3, 2005 concerning RAIs, Dated: June 13, 2005

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**LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE  
TO DISCUSS THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2  
LICENSE RENEWAL APPLICATION**

**MARCH 3, 2005**

| <u>Participant</u> | <u>Affiliation</u>                       |
|--------------------|--|
| John Thorgersen    | Nuclear Management Company, LLC (NMC)    |
| Jim Knorr          | NMC                                      |
| Ann Marie Stone    | U.S. Nuclear Regulatory Commission (NRC) |
| Barry Elliot       | NRC                                      |
| Timothy Steingass  | NRC                                      |
| James Strnisha     | NRC                                      |
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| Verónica Rodríguez | NRC                                      |

REQUESTS FOR ADDITIONAL INFORMATION (RAIs)  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2  
LICENSE RENEWAL APPLICATION

March 3, 2005

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on March 3, 2005, to discuss and clarify the staff's requests for additional information (RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application (LRA). The following RAIs were discussed during the telephone conference call.

**Boraflex Monitoring Program**

**RAI B-2.1.5-1**

Boraflex coupon inspections provide information regarding the extent of Boraflex panel degradation in the spent fuel racks. The LRA states that 10 full-length Boraflex panels are tested at 5-year intervals (four accelerated panels and six random panels). It is unclear if it is intended to inspect Boraflex coupons in addition to inspecting the 10 full-length Boraflex panels. The staff requests the applicant to clarify this information.

**Discussion:** The applicant indicated and the staff agreed that this AMP has been previously reviewed by the Audit Team. This question will be WITHDRAWN. Instead, the Spent Fuel Pool Storage Rack Boraflex TLAA will be evaluated.

**RAI B-2.1.5-2**

LRA Page B-61, states that the EPRI RACKLIFE predictive code or its equivalent is used to trend and analyze the results of the silica level measurements in the spent fuel pool (SFP). The staff requests the applicant to indicate what other "equivalent predictive codes" could be used. If these codes significantly differ from the EPRI RACKLIFE predictive code, please describe these codes and discuss the significant differences. In addition, provide the criteria used for determining the frequency of silica level measurements in the SFP (i.e., monthly, quarterly, or annually).

**Discussion:** The applicant indicated and the staff agreed that this AMP has been previously reviewed by the Audit Team. This question will be WITHDRAWN. Instead, the Spent Fuel Pool Storage Rack Boraflex TLAA will be evaluated.

**RAI B-2.1.5-3**

The LRA indicates that enhancements to NUREG-1801 are to be completed prior to the period of extended operation. These enhancements involve the creation of "new procedures" for Boraflex areal density testing, blackness testing, trending and analysis of silica sampling results, and determination of accelerated exposure panels. The staff requests the applicant to provide specific information regarding each of these enhancements.

Enclosure 2

**Discussion:** The applicant indicated and the staff agreed that this AMP has been previously reviewed by the Audit Team. This question will be WITHDRAWN. Instead, the Spent Fuel Pool Storage Rack Boraflex TLAA will be evaluated.

#### **RAI B-2.1.5-4**

The Boraflex Monitoring Program at Point Beach performs the required scheduled surveillance program at a minimum frequency of 5-years. However, NUREG-1801 requires that "certain accelerated samples are tested every two years." The staff requests the applicant to justify this frequency difference and discuss any consequences of this less frequent surveillance program.

**Discussion:** The applicant indicated and the staff agreed that this AMP has been previously reviewed by the Audit Team. This question will be WITHDRAWN. Instead, the Spent Fuel Pool Storage Rack Boraflex TLAA will be evaluated.

### **Time Limited Aging Analysis**

#### **Section 4.4.3 - Reactor Coolant Pump Casing**

##### **RAI 4.4.3-1**

Section 4.4.3 of the application indicates that the applicant has re-evaluated the fracture mechanics analyses to ASME Code Case-481 documented in WCAP-13045 and WCAP-14705 for the PBNP Units 1 and 2 RCP casings and the analyses remain valid for the 60-year extended license operating period. The application indicates that these components are not susceptible to thermal aging because they satisfy the criteria in the NRC safety evaluation for WCAP-14575-A. The application also indicates that the fracture mechanics analysis will not be revised and resubmitted to the NRC for the extended period of operation because the code case has been superseded by the ASME Code and the analysis is no longer needed. The staff requests that the applicant evaluate the ASME Code Case-481 analysis to the criteria for time-limited aging analysis (TLAA) in 10 CFR 54.3 to determine whether the analysis satisfies the criteria and should be considered a TLAA. If it satisfies the TLAA criteria, the applicant is requested to identify the changes to the analysis that result from the proposed additional 20 years of facility operation and to provide the results of the analysis that satisfy 10 CFR 54.21(c)(i), (ii) or (iii).

**Discussion:** The applicant indicated and the staff agreed that the reactor coolant pump integrity analysis is not a TLAA for PBNP, as described in LRA Section 4.4.3. The code case has been superseded by the ASME code and the analysis is no longer required. The TLAA in LRA Section 4.4.3 will be deleted and therefore, this question will be WITHDRAWN.

#### **Section 4.4.8 - Component/Piping Subsurface Indication Analysis**

**Discussion:** The applicant indicated and the staff agreed that PBNP does not have any active TLAA's associated with component/piping subsurface indications. It was agreed that there is reason for withdrawing this TLAA, therefore, LRA Section 4.4.8 will be deleted by the applicant.

#### **Section 4.7.2 - Containment Accident Recirculation Heat Exchanger Tube Wear**

**Discussion:** The applicant indicated and the staff agreed that the containment accident recirculation heat exchanger tube wear is not a TLAA for PBNP, as described in LRA Section 4.7.2. Therefore, the TLAA will be deleted from LRA Section 4.7.2.