

Exelon Generation Company, LLC
LaSalle County Station
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June 3, 2005

10 CFR 50.73

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

LaSalle County Station, Unit 1
Facility Operating License No. NPF 11
NRC Docket No. 50-373

Subject: Licensee Event Report

In accordance with 10 CFR 50.73 (a)(2)(i)(B), Exelon Generation Company, (EGC), LLC, is submitting Licensee Event Report Number 05-002-00, Docket No. 050-373.

Should you have any questions concerning this letter, please contact Mr. Terrence W. Simpkin, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



Daniel Enright
Plant Manager
LaSalle County Station

Attachment: Licensee Event Report

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - LaSalle County Station

IE22

LICENSEE EVENT REPORT (LER)(See reverse for required number of
digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME LaSalle County Station, Unit 1**2. DOCKET NUMBER**

05000373

3. PAGE

1 of 3

4. TITLE Reactor Core Isolation Cooling (RCIC) Barometric Condenser Vacuum Pump Discharge Check Valve 1E51-F028 Failed Local Leak Rate Test**5. EVENT DATE****6. LER NUMBER****7. REPORT DATE****8. OTHER FACILITIES INVOLVED**

MO DAY YEAR

YEAR SEQUENTIAL
NUMBERREV
NO

MO DAY YEAR

FACILITY NAME

DOCKET NUMBER

04 07 2005

2005 - 002 - 00

06 03 2005

FACILITY NAME

DOCKET NUMBER

**9. OPERATING
MODE**

1

**10. POWER
LEVEL**

100

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)

<input type="checkbox"/>	20.2201(b)	<input type="checkbox"/>	20.2203(a)(3)(ii)	<input type="checkbox"/>	50.73(a)(2)(ii)(B)	<input type="checkbox"/>	50.73(a)(2)(ix)(A)
<input type="checkbox"/>	20.2201(d)	<input type="checkbox"/>	20.2203(a)(4)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(x)
<input type="checkbox"/>	20.2203(a)(1)	<input type="checkbox"/>	50.36(c)(1)(i)(A)	<input type="checkbox"/>	50.73(a)(2)(iv)(A)	<input type="checkbox"/>	73.71(a)(4)
<input type="checkbox"/>	20.2203(a)(2)(i)	<input type="checkbox"/>	50.36(c)(1)(ii)(A)	<input type="checkbox"/>	50.73(a)(2)(v)(A)	<input type="checkbox"/>	73.71(a)(5)
<input type="checkbox"/>	20.2203(a)(2)(ii)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(v)(B)	<input type="checkbox"/>	OTHER
<input type="checkbox"/>	20.2203(a)(2)(iii)	<input type="checkbox"/>	50.46(a)(3)(ii)	<input type="checkbox"/>	50.73(a)(2)(v)(C)	Specify in Abstract below or in NRC Form 366A	
<input type="checkbox"/>	20.2203(a)(2)(iv)	<input type="checkbox"/>	50.73(a)(2)(i)(A)	<input type="checkbox"/>	50.73(a)(2)(v)(D)		
<input type="checkbox"/>	20.2203(a)(2)(v)	<input checked="" type="checkbox"/>	50.73(a)(2)(i)(B)	<input type="checkbox"/>	50.73(a)(2)(vii)		
<input type="checkbox"/>	20.2203(a)(2)(vi)	<input type="checkbox"/>	50.73(a)(2)(i)(C)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)		
<input type="checkbox"/>	20.2203(a)(3)(i)	<input type="checkbox"/>	50.73(a)(2)(ii)(A)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)		

12. LICENSEE CONTACT FOR THIS LER

NAME

Bob Tjernlund, Design Engineering

TELEPHONE NUMBER (Include Area Code)

(815) 415-2918

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU- FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU- FACTURER	REPORTABLE TO EPIX
X	BN	ISV	Flowserve	Y					

14. SUPPLEMENTAL REPORT EXPECTED☒ YES
(If yes, complete EXPECTED SUBMISSION DATE)☐ NO**15. EXPECTED
SUBMISSION
DATE**MONTH
07DAY
15YEAR
05**16. ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)**

On 4/7/05, local leak rate testing (LLRT) was being performed on Unit 1 Reactor Core Isolation Cooling (RCIC) [BN] Barometric Condenser Vacuum Pump Discharge isolation valves 1E51-F069 and 1E51-F028. The procedurally required test pressure could not be achieved between these valves, and the measured leakage was therefore classified as infinite.

Acceptable test pressure and measured leakage were achieved when the test operator mechanically agitated check valve 1E51-F028. The 1E51-F028 check valve was declared inoperable and the 1E51-F069 motor-operated globe valve was closed.

The 1E51-F028 valve was replaced and tested acceptably. A root cause investigation is in progress to determine the cause of the valve failure.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET NUMBER	6. LER NUMBER			3. PAGE
LaSalle County Station, Unit 1	05000373	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 of 3
		05	- 002	- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor, 3489 Megawatts Thermal Rated Core Power

A. CONDITION PRIOR TO EVENT

Unit(s): 1 Event Date: 04/07/2005 Event Time: 1130 CDT
Reactor Mode(s): 1 Power Level(s): 100
Mode(s) Name: Run

B. DESCRIPTION OF EVENT

On 4/7/05, local leak rate testing (LLRT) was being performed on Unit 1 Reactor Core Isolation Cooling (RCIC) [BN] Barometric Condenser Vacuum Pump Discharge isolation valves 1E51-F069 and 1E51-F028. 1E51-F069 is a motor operated globe valve and the 1E51-F028 is a piston lift check valve. The LLRT was being performed as a follow up to previous failure of the 1E51-F028 valve in September 2004. The procedurally required test pressure could not be achieved between these valves, and the measured leakage was therefore classified as infinite. Acceptable test pressure and measured leakage were achieved when the test operator mechanically agitated check valve 1E51-F028.

The 1E51-F028 valve was declared inoperable. 1E51-F069 was closed in accordance with Technical Specification (TS) 3.6.1.3 Required Action A.1 and A.2, and was controlled with an equipment status tag.

Indications were that 1E51-F028 was stuck open and had not been capable of performing its specified safety function for a period of time longer than allowed by TS. This event is therefore reportable under 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT

The direct cause of this event was that the 1E51-F028-valve disc (piston) was stuck in the open position. Following a similar failure in September 2004, the piston and spring were replaced, and the valve internals were cleaned. A root cause investigation of the successive failures is in progress.

D. SAFETY ANALYSIS

The safety significance of this event was minimal. Isolation valve 1E51-F069 was fully operable as demonstrated with the local leak rate testing, so the primary containment integrity remained intact.

This was not a safety system functional failure.

E. CORRECTIVE ACTIONS

- Check valve 1E51-F028 was replaced on 5/5/05 and tested satisfactorily (Complete).
- A root cause investigation is in progress to identify corrective actions to prevent further occurrences (AT# 322203-03). A supplement to this LER will be submitted on completion of the root cause report (AT# 322203-15).

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		05	- 002	- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

F. PREVIOUS OCCURRENCES

<u>LER Number</u>	<u>Title</u>
373-04-001	Invalid Containment Isolation Valve Local Leak Rate Test Due to Inadequate Procedure

The corrective actions from this LER were directed at correcting errors in LLRT methodology. 1E51-F028 failed its LLRT after the procedure was corrected, and the piston and spring were replaced, and the body internals were cleaned.

G. COMPONENT FAILURE DATA

Piston Lift Check Valve, Flowserve (Rockwell Edwards), Fig 838YT