May 26, 2005

LICENSEE: Tennessee Valley Authority (TVA)

FACILITY: Browns Ferry Nuclear Plant (BFN), Units 1, 2, and 3

SUBJECT: SUMMARY OF A CONFERENCE CALL HELD ON MAY 4, 2005, BETWEEN THE NRC STAFF AND TENNESSEE VALLEY AUTHORITY (TVA) LICENSE RENEWAL STAFF, CONCERNING INFORMATION ON BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3, LICENSE RENEWAL APPLICATION (TAC NOS. MC1704, MC1705, AND MC1706)

On May 4, 2005, the U.S. Nuclear Regulatory Commission (NRC) staff, conducted a conference call with TVA, the applicant. This was a follow-up call concerning the applicant's responses to the staff's requests for additional information (RAIs) previously issued in December 2004 in Sections 2.4 - Scoping and Screening Results: Structures and Section 3.5 - Aging Management of Containment, Structures and Component Supports of the Browns Ferry Nuclear Plant (BFN) license renewal application (LRA). Enclosure 1 provides a list of conference call participants. Enclosure 2 is a summary of the RAIs discussed during the conference call.

The applicant agreed to provide written responses on the above-mentioned RAIs. Many items that were raised during the conference call remained unresolved. These issues are identified in Enclosure 2.

The NRC staff emphasized the importance of adequate advance preparation by the applicant to adequately respond to staff's questions. The applicant assured the staff that TVA will be better prepared for the staff's questions in the future.

The applicant had an opportunity to comment on the meeting summary.

/RA/

Ram Subbaratnam, Project Manager License Renewal Section A License Renewal and Environmental Impacts Program Division of Regulatory Improvement Programs Office of Nuclear Reactor Regulation

Docket Nos.: 50-259, 50-260 and 50-296

Enclosures: As stated

May 26, 2005

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- FACILITY: Browns Ferry Nuclear Plant (BFN), Units 1, 2, and 3
- SUBJECT: SUMMARY OF A CONFERENCE CALL HELD ON MAY 4, 2005, BETWEEN THE NRC STAFF AND TENNESSEE VALLEY AUTHORITY (TVA) LICENSE RENEWAL STAFF, CONCERNING INFORMATION ON BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3, LICENSE RENEWAL APPLICATION (TAC NOS. MC1704, MC1705, AND MC1706)

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ADAMS Accession No.: ML051460418

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DATE:	5/25/05	5/24/05	5/17/05	5/26/05

LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE CALL TO DISCUSS BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2 AND 3 LICENSE RENEWAL APPLICATION MAY 4, 2005

Participants	Affiliation		
Ram Subbaratnam	U.S. Nuclear Regulatory Commission (NRC)		
Yoira Diaz	NRC		
David Jeng	NRC		
Hansraj Ashar	NRC		
Arnold Lee	NRC		
Tim Abney	TVA (Tennessee Valley Authority)		
Ken Brune	TVA		
Joe McCarthy	TVA		
Garry Smith	TVA		
Kevin Green	TVA		
Erick Locker	TVA		
Vick Shivoni	TVA		
Micky Hamby	TVA		

SUMMARY OF REQUESTS FOR ADDITIONAL INFORMATION (RAIs) DISCUSSED DURING THE CONFERENCE CALL HELD BETWEEN U.S. NUCLEAR REGULATORY COMMISSION (NRC) AND TENNESSEE VALLEY AUTHORITY (TVA) LICENSE RENEWAL STAFF MAY 4, 2005

I. Sections 2.4 - Scoping and Screening Results: Structures

<u>RAI-2.4-2</u>: The applicant has not explicitly included the components internal to drywell and suppression chambers within the scope of license renewal. The applicant agreed that everything inside the primary containment should be shown as a commodity group, and will be included into the LRA scope. This information will be provided in a formal response to the RAI.

RAI-2.4-3: The staff had a concern related to drywell shell corrosion experienced in BFN, Units 1, 2, and 3. This is described by the applicant in the program element for operating experience in the Aging Management Program (AMP) B.2.1.32, "ASME Section XI Subsection IWE Program." The staff is concerned about the degradation and requested the applicant to evaluate if this experience was a result of an inefficient refueling cavity seal. If it is, the staff requested that the seals be included into the LRA scope and the applicant properly evaluate their operating experience. The applicant agreed to evaluate this issue further. This information will be provided in a formal response to the RAI.

II. Section 3.5 - Aging Management Review of Containment, Structures and Component Supports

RAI-3.5-1: The staff had a concern regarding aging management of the vent line bellows inside the containment which is currently being done via use of Appendix J program Type -A testing. The staff contended that this test alone is inadequate for this component. The staff still needs additional information supporting how the integrity of the vent line bellows are maintained and details of relevant operating experience at BFN. This information will be provided in a formal response to the RAI.

RAI-3.5-4: The staff had concerns regarding the degradation of the drywell shells (inside and outside). The applicant's response dated January 31, 2005, did not address a number of questions that are pertinent to the period of extended operation. The staff requested additional clarifications to resolve the issue. This information will be provided in a formal response to the RAI.

<u>RAI-3.5-5</u>: The applicant's response did not fully address staff concerns regarding degradation of pedestals supporting the reactor vessels and seismic restraints anchored to the sacrificial shields and the drywell. The staff requested additional clarifications to resolve the issue. This information will be provided in a formal response to the RAI.

RAI-3.5-14: The staff requested the applicant to clarify the key elements for periodic inspection program for the boral coupon test specimens, some of which had shown blisters. These details should include an objective, scope, frequency and inspection approach to the program. During the conference call, the applicant stated that they have an internal plant procedure to inspect these test coupons, but do not have an AMP. The applicant keeps track of the degradation of

these neutron absorber plates inside the test coupons. The applicant will provide the NRC staff with additional information which will include a history of these inspections. This information will be provided in a formal response to the RAI

Follow-up to RAI B.2.1.33-2: During the call the applicant stated that 10 CFR 50.55a (g)(4) excluded the inspection of supports for Class MC components while using Section XI IWF ISI Program. The NRC staff clarified that, subsequent to the 50.55a rulemaking the staff is re-thinking the inspection requirement and is engaged in an ongoing dialogue with NEI/Industry. However for license renewal, the applicant needs to demonstrate how aging effects of supports for class MC components will be managed. This information will be provided in a formal response to the RAI.

Follow-up to RAI B.2.1.36: The staff had another question regarding evaluation of inspection personnel qualification based on Industry Guidance ACI 349.3R-96 as stated in the structures monitoring program. The staff stated that this industry guidance alone will not be adequate to qualify the inspectors for the examination of steel supports for the AMP B.2.1.36. The staff requested that the applicant reevaluate the program element from previous staff positions and submit the description for staff review. This information will be provided in a formal response to the RAI.

DISTRIBUTION: Summary of a conference call on May 4, 2005 with TVA, Dated: May , 2005 Adams Accession No.: ML051460418

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BROWNS FERRY NUCLEAR PLANT

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BROWNS FERRY NUCLEAR PLANT

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