From:Headquarters Operations Officer 1To:John White; Region 1 Incident Response Center; Tim McGintyDate:4/17/05 2:17PMSubject:documents sent to HOO from Millstone

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>>> Region 1 Incident Response Center 04/17/05 01:44PM >>>

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					t1 of 2)			
	NRC CONTACT (NA	ME)	Bill Gott		NRC EVEN	TI	NUMBER: 41607	
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L	·						100 44322	. 16
	EVENTTIME (ESTORE	EVENT DATE or IRF#		FORE 160% MODE 1		POWER (56) and MODE AF. TER 07. NOVE3		
			2003071		100%		010 10005	
	EVENT CLASSIFICATION		-Hr Non-Emergency 10 CFR 50.72 (b)(1)	Τ	4-Rr Non-Emergency 10 CFR 50.72 (b)(2)	Γ	8-Hr Non-Emergency 10 CFR 50.72 (b)(3)	
	GENERAL EMERGENCY		TS Deviation	Τ	(i) TS Required Shut- down		(ii)(A) Degraded Condition	
	SITE AREA EMERGENCY		· · · · · · · · · · · · · · · · · · ·	k	(iv)(A) ECCS Discharge to RCS		(ii)(B) Unanalyzed Condi- tion that Dogrades Safety	
X	ALERT		50 Day Optional 10 CFR 50.73 (a)(1)	X	(iv)(B) RPS Actuation (Scram)	X	(iv)(A) Specified System Ac- tuation	
	UNUSUAL EVENT		nvalid Specified System Ac- uation	Γ	(xi) Offsite Notification		(v)(A) Safe Shutdown Ca- pability	
V	50.72 NON-EMERGENCY	╈					(v)(B) RHR Capability	
	PHYSICAL SECURITY (73.71)			Γ			(v)(C) Control of Rad Re- lease	
_	MATERIAL/EXPOSURE	┿		┢			(v)(D) Accident Mitigation	
	FITNESS FOR DUTY	1					(xii) Offsite Medical	
	OTHER UNSPECIFIED RE- QUIREMENT (IDENTIFY)	T					(xiii) Lost ENS	
	INFORMATION ONLY						(xiii) Lost Other Assess- ment/Offsite Comms	
1							(xiii) Emergency Siren INOP	
01	HER UNSPECIFIED REQUIRE	MEN	IT (LDENTIFY):					
	····							
$\frac{DE}{I.}$	SCRIPTION: (Fill in us available	:)						
	Reactor Tri	Р	, Safety In	ìć	ction, AFWe	20	tuation	
2.	Reactor Trip		SIL AFW		Low Stean	2	line pressure	
J.	CAUSES (IF KNOWN): COUSE 15	- -	ider invest	-17	ation			
J .	EFFECT OF EVENT ON PLA			. 7			C. C. L. Value an	
5.	ACTIONS TAKEN OR PLAN	YED;		\mathbf{J}			y Safety valve ac	
5.	ADDITIONAL INFORMATION SEE ATTAC	N:	MENT	<u>س</u>	n to Mode	Ľ	5 cold shutdow	
	Level of Use Reference]	STOP		ACT REVIEW		RAC 14 Rev. 001-04 98 of 115	

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						NR	IC Ev		loti et 2			on Fo	orm						
NOTIFICATIONS YES NO						· *	VILL BE	ANYTHING UNUSUAL OR NOT UNDERSTOOD?					OT Y	ES M EXPLAIN SHE	ET 1)	NO [NO		
NRC RESIDENT						+		DID ALL SYSTEMS FUNCTION AS REQUIRED?					Y NC	F2S 🗋		NO [(EXI	NO E (EXPLAIN SHEET 1)		
STATE V								MODE OF OPERATION UNTIL CORRECTED: 5											
LOCAL							ESTIM	ESTIMATED RESTART DATE: UNKINS WY											
OTHER GOV AGENCIES						ADDIT	IONAL	LINF	00	NBACI	K: YES		N0	<u>9</u>					
	DIA/PRESS RELE	_		V									_						
RA	DIOLOGICAL RI				R FI		_		TEM			<u></u>							
	LIQUID GASEON RELEASE RELEAS					INPLAN RELEASE			PLANNE				ONGOING			TE	ERMINATED		
	MONITORED UNMON		TTOREI			OFFSITE	:		T.S EXC		DED		RM ALARMS			AREAS EVACUATED			
	PERSONNEL EXPOSED OR OFFSITE PROTECTIVE ACTIONS State release path in description RECOMMENDED																		
							Releas (Cl/sci	e Rate :)	%T.S.	&T.S. LIMIT		ноо с	JUIDE	Total Activity (Ci)		rs. Lim	IT	HOO GUIDE	
Noh	le Gas											0.1 Ci/	-					1000 Ci	
Iodi												10 µCi/sec		ļ				0.01Ci	
	Particulate										1 μCi/s				l mCi				
	Liquid (excluding tritlum & dissolved noble gases)										10 µCi						0.1 Ci 5 Ci		
	iquid (tritium)								0.2 Ci/	min									
100	Total Activity PLANT STACK CON EJEC							NDENSER/AIR CTOR			LIN	ne l		SG .OWDOWN			OTHER		
	RAD MONITOR READ- INGS:			20 4.3			3 <i>E</i> -	E-6		7.48E		~2	ī£0(ated					
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% T. ble)	S. LIMIT (if applie	2.																	
RCS	S OR SG TUBE LI	eaks	S: CIIEC	K OR F	TLL	IN AI	PPLICA	BLE ITE	MS:										
LOC	CATION OF THE L	EAK	(c.g., SG	#, valve,	pipe,	e(C):	No	ne											
LFAK RATE: UNITS gpm/gpd							T. S. LIMITS:						SUDDEN OR LONG-TERM DEVELOPMENT						
LEA	LEAK START DATE: TIME:								COOLANT ACTIVITY & UNITS: PRIMARY - SECONDARY -										
LIST	r of safety rei	LATE	D EQUIP	MENT N	NOT	OPER	ATIONA	L:	I ,										
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APR-17-2005 SUN 01:44 PM MP MANAGER PUBLIC INFO.

Attachment to April 17, 2005 Event Notification Summary of Event

At approximately 0830 on April 17, 2005 Millstone Unit 3 experienced an automatic reactor trip from full power. The Reactor is in a stable shutdown condition and is proceeding to Mode 5 cold shutdown. No release of radioactivity other than those minor releases associated with normal plant operation has occurred. There were no personnel injuries associated with the event.

In addition to the Reactor Trip, half-train Safety Injection (SI) and half-train Main Steam Isolation (MSI) actuations occurred. Control room personnel were successful at manually initiating full SI and MSI. SI has since been terminated and RCS pressure control has been restored. Multiple steam line safety valves (MSSV) lifted with at least one MSSV indicating a failure to reclose. As a result, an ALERT was declared at approximately 0853 due to the failure of at least one MSSV failing to reclose. All MSSV's are currently closed. Auxiliary Feedwater (AFW) actuated automatically as expected, however; the turbine driven auxiliary feedwater pump (TDAFWP) tripped on start-up and was subsequently reset locally. Both motor driven auxiliary feedwater pumps (MDAFWPs) operated as expected to maintain steam generator water levels. Due to the additional inventory injected to the Reactor Coolant System (RCS) as a result of the SI actuation, the pressurizer filled and a primary system safety valve (PSSV) actuated at a lower than expected pressure. All PSSV's are currently closed, however; there is indication of leakage past either a PSSV or Pilot Operated Relief Valve (PORV). Upon alignment of the Charging system to Refueling Water Storage Tank recirculation flow path, 2 valves had indication of packing leakage. Those packing leaks have been terminated.

The cause of the event is under investigation.