

4/17/05

8:29

DRP POLICY NO. 6  
PLANT TRANSIENT RESPONSE CHECKLIST

8:42

ALERT - EAL BAZ  
Px trip - s/o relief stroke  
open, SCAS/MSI

PURPOSE

To provide a broad checklist of topics (Attachment 1) for reference by regional staff and management following a plant transient or event of heightened interest.

REFERENCES

- Management Directive 8.3, ANRC Incident Investigation Program@
- NRC Inspection Procedure 71153, AEvent Followup@
- NRC Inspection Procedure 93812, ASpecial Inspection@
- NRC Inspection Procedure 93800, AAugmented Inspection Team@
- NUREG-1303, AIncident Investigation Manual@

GUIDANCE

The initial NRC reaction to a plant transient or event must be concise yet comprehensive to ensure an appropriate response. This checklist can help guard against becoming too consumed in the details to overlook the obvious. The intent is not to replace established procedures pertaining to NRC incident response, but rather to provide those involved with a list of broader topics that should be considered. Although the questions and areas of concern following an event may expand beyond this checklist, this should provide a sound base to help form the regional response.

IMPLEMENTATION

Resident Inspectors and DRP management should retain a copy of Attachment 1 for use during a plant transient or event with heightened interest. In addition, a copy should be retained for use in the incident response center and the regional duty officer brief case.

ATTACHMENT 1

EVENT FOLLOWUP CHECKLIST

Jeff Semancile & Steve Sover

Initial Event Response

- ✓ 1. Observe plant parameters and discuss with the licensee the current plant status. Is the safety significance of the situation fully understood?
- ✓ 2. Is the plant stable? Are plant conditions currently degrading or are conditions expected to degrade? *Plant is stable, previously there were complications,*
- ✓ 3. Review control board status, equipment indications, operator logs, and strip charts to determine current condition of the plant and if anything unusual has occurred.

A/20

4/17/05 8:29 am

ALERT DECLARED

See Notes.

4. Determine alarms or conditions preceding or indicating the event.

5. Obtain details about the event, including the initial assessment of the event by the licensee.

Yes. 6. Is there anything unusual or not understood about the event? Several things - initiator - TD AFWS - 1 train of ESAS

7. Was there any major degradation of fuel integrity, primary coolant pressure boundary, or primary containment boundary? Not Major - PWR/Leakage from pcc.

Yes. 8. Is there any identified or un-identified leakage, especially from the reactor coolant system? Leakage from PWRs & Safeties < uc. amount.

9. Is a radioactive release occurring or imminent? Were there any radiological consequences due to the event? - Leak in Aux Bldg - Steam Release & S/G samples to validate no 1<sup>0</sup>-2<sup>0</sup>

10. Has the event resulted in unexpected personnel exposure or contamination? No. Hand & foot prints if you were in the RCA.

Now Indicated. 11. Did the event involve a substantial breakdown of physical security or sabotage at the facility?

12. Did adverse weather cause the event or can weather conditions increase the significance of the event or cause any degradation to plant systems, structures or components? Transition to IP 71111.01, as needed.

13. Is the plant in an Emergency Action Level (EAL) per the site Emergency Plan? Should the site be in an EAL? Can the EAL escalate based on the progress of the event? yes. Yes. Not likely to escalate.

14. Were any technical specification safety limits exceeded?

15. Did the event involve an activity or operation that was in violation of the technical specifications, license conditions, or other regulatory requirements? No violation of TS. They determined later that they had 12 hours versus 6 hours. Exceeded AOT for time to Hot stop S/D?

16. Is the offsite grid and switchyard stable?

17. Verify that risk significant systems performed as expected. Did plant equipment perform as expected during the event? Transition into IP 71111.13, as needed. Not all eqpt. 1 train ESAS

18. What equipment is needed or available to achieve safe shutdown? ADV bypass valves.

19. What is the status of safety equipment that is currently out of service? What are potential adverse consequences of additional equipment failures? C No S/G sampling. CHG valve failed packing. S/G sample line.

20. For multi-unit sites, did the event have any effect on the opposite unit(s)? Opp. Unit is in outage. Drain down delayed.

Yes. 21. Were appropriate actions taken by operations personnel during the event? Interview plant personnel, as necessary, to obtain information regarding the operators response to the event. Transition into IP 71111.14, as needed. No problems indicated.

22. Were any other human factor responsible for the event or abnormalities during the event? None we are aware of at this time.

23. Were the licensee's actions consistent with license requirements and approved procedures?

Yes. 24. Do station procedures or emergency operating procedures (EOPs) cover the plant conditions?

Kind of. Some recovery of BLD line to sample A & D S/Gs may not be covered.

25. Any other consequences due to the event?

26. Is there any media interest/interest from the local populace in the event?

*Yes. Channel 3. 2 news releases from licensee.*

27. Did the licensee make the proper notifications to the NRC and other agencies for the event?  
(See technical specifications and 10 CFR 20.2202 thru 2205, 50.72 and 73.71 criteria)

28. Does the licensee have a management team in place to address the event?

*Yes. SERO / EDF / TSC Manned.*

29. Have appropriate NRC management been briefed?

*[ - Jim Dyer / Elliot Marschiff ]  
Technical Assistant briefing  
(Commissioner TAs).*

Short Term Event Followup Actions

30. Evaluate the significance of the event using the SDP and the regional SRAs.

31. Does the licensee plan to issue a press release about the event?

*Yes. 2  
Yes*

32. Have the regional Public Affairs Office and State Liaison Office been informed?

33. Was the plants response during the event within the bounds of the FSAR analysis? Evaluate the plants data and compare it with the design data and FSAR descriptions.

34. Should plant equipment be quarantined?

35. Determine the event chronology.

36. Were there any event precursors? *- Possible: power supply replacement on 'B' s/w pressure instrument, could be related.*

37. Evaluate whether the licensee has appropriately resolved any abnormal issues from the event prior to restart.

38. Has the licensee adequately analyzed the event?

39. Has the licensee determined the cause(s) of the event and established initial corrective actions to prevent recurrence prior to restart?

40. Have regional and headquarters management been fully briefed with the details of the event, including the licensee's cause(s) for the event and corrective actions prior to restart?

41. Is there a need for additional or continued NRC response to the event?

Longer Term Event Followup Actions

42. Did the licensee's event evaluation include assessment of similar previous events?

43. Are there any generic safety concerns from the event that are potentially applicable to other facilities?

*317*

Max,

Observations from 8:00 PM - 11:45 PM

- Initial problems acquiring SG samples due to inability to re-open valve HV-22A. Sample found  $< MDA$  at 9:53 PM. Tritium sample on-going.
- Two packing leaks in CVCS ( valve 661 and CV-V-8511B).
- Licensee secured ERDS data at time of de-escalation from ALERT.
- Observed placing RHR in-service. Operators did a good job. Ensure TS 3.6.3 applied and adm. controls in place for suction valves.
- ERT focus:
  - 1) SSPS train 'A' power card.
  - 2) SG / Pressurizer Safety Valve perf.
  - 3) Cause of MS-V-5 Trip on AFW TT.
  - 4) 2 leaking charging valves.

Thx. Please call with any Q's. [Lita.]

Turnover w/

Pete Habighorst

610-420-0015

pp. 8-9 of my notes

- See TA Briefing Points. [SAS SSPS fault?]
- Priorities -
  - ① c/o the plant
  - ② s/b samples
  - ③ Investigate unexpected alarms during C/D → actually only RCP high range leakage flow low.
- Current Plant Conditions
- Tampering Investigation Result - 1:05 pm No tampering indicated.  
- 2-3 hours from 7:30 pm.
- Need to get on RHR (Mode 5)
- Region will stand down from LRC based on s/b samples or on RHR.

Long Term - Cause?

- PORVs reaction to water environment.
- Why did TOAFW trip?
- TS entries, evaluation.
- RCS LEAKAGE EAL evaluation - did they recognize the OAE for V.E. potential?
- Continuing to steam a s/b w/o knowing the chemistry of the s/b to ensure no 1<sup>o</sup>-2<sup>o</sup> leak.
- Radiological follow-up on leak in Aux Bldg from charging.
- "Release to the public" analysis by HP.
- Special Inspection Team.

START - 9:30am

Notified by Hoop  
ALERT - MS3

4/17/05: Millstone Unit 3

Steve Sauer  
Jeff Semanic

10:10 am  
Jim Keuzel - MOC

'B' s/g low steam line pressure → also MSI.

- 0829 Unit 3 Tripped - CAUSE?
- 0842 ALERT C-1, BAZ.
- 'B' & 'C' s/g <sup>code</sup> safety valves opened & stayed open. (Safety's closed, opened & closed). 15-20 min before they stabilized.
- MSI activation → low pressure.
- F.P. Barriers - OK
- No onsite protective actions or early dismissal
- Plant is stable.
- 'A' train SIAS (only), manually initiated SI.
- 'CH6' - CHV-661 packing leakage excessive; 'B' is in service
- EOP ES-1.1 → 'A' ch6 pump.
- No 50.54X → SI termination.
- 0849 State notified, no radio

EOP & TSC assumed

• Scott Smith is MANAGER of control room.

\* ALERT classification -

DSE0 - Steve Sauer, Bill Haffner.

- BAZ:
- Unisolable steam leak outside Unit.
  - Turbine started but tripped. Since been reset & running OK. Both MDAEW working.
  - PZR SAFETIES lifted, went solid on the pZR.
  - PZR level = 84
  - Pressure = 2100 # trending toward NOP
  - Temp @ NOT.

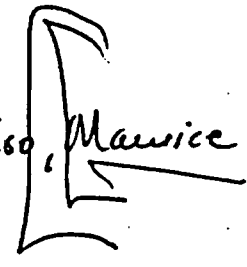
Auto Rx Trip / SIAS / MSI on low side line  
 pressure on 'A' train only. & then  
 did manual Rx Trip / SIAS / MSI.  
 (did replace a BS/<sup>power</sup>~~pressure~~ supply for ~~BS~~  
 Star pressure)

- ESF Activation TS.
- Rx Trip
- Initiator, chg leak, ferry turbine, S/G sensors didn't reset initially! (then further lifts). S/G pressure did not appear to be at left setpoint.
- ADVs were blocked by MSI signal.

Tampering Procedure.  
 Skip Jordan  
 Manager of Communications.

- SADS link is not currently working, IT is responding.
- Did not conclude whether a precautionary site dismissal was required.
- Looking at Unit 2 activities ~~not~~ for any work to put on hold.

Notified the residents to come in. Also, Maurice



1053. Heath.  
Felicia → ERC  
 Licensee briefing →  
 - PRT intact  
 - Did they re-open MSIVs?

Alan Price:

Status

0829 → Unit 3 Rx trip / SIAE / MSIV low 5/6 pressure.  
 → No reason to believe tampering is involved @ this time.

- Main Safety systems operated.
- Recovered per level & pressure.
- 64% per level, 2200# in RCS
- Transitioned to ES 1.1, terminated SI back in normal cty / shutdown.

0842 → ALERT - Unisolable. Steam Line Break outside Cont. due to 5/6 relief lifting below the setpoint.

- What would termination criteria be? MSI signal prevents dumping to condensers? on ADVs to remove decay heat.
- Looking at termination criteria for ALERT & looking @ UE.
- Personnel @ state armory.
- Corporate EOP is staffed.



Wayne  
Alan  
Not True

ERDS?  
Primary chemistry? Fuel Leakers?  
Sampled slgs, no indications of 1<sup>o</sup> - 2<sup>o</sup> leak.  
Water from racirc line or chg introduced into Aux BLDG.  
No fuel leakers.  
Max 1<sup>o</sup> pressure = 2300(?) Leakage from a  
PRV, 75% level in PRT.  
PRT rupture disc?  
PRT level ↑ slowly, 82%, pressure = 65<sup>psi</sup>  
Any other indication of leakage by way of sumps  
or otherwise?

Dan  
Alan

Primary left OR leakage?  
Indications of safety leakage but no 1<sup>o</sup> pressure  
at PORV subpoint.  
ERDS actuated.

Jim  
Wiggins

Bill Haffner is the DSEO.  
No indications of chert sump leakage.  
Plan for word? PZR safeties don't liberate water.  
PZR solid conditions were 1<sup>st</sup> priority.  
Imagine further <sup>steps to</sup> cool down the primary.  
ERDS still not working.

Rayne

Brian

END of BRIEF  
Can they pump PRT?  
Is there only 1 relief leaking? How indicated?  
Results from 1<sup>o</sup> chemistry?

ENS line?  
HPN line?  
HOD - Horrible Communications / INITIALLY COULDN'T GET IN BLDG.

→ Larry Doeflein.  
 → Wayne →

→ s/g safety lifted first?

- Remains in ALERT
- 'B' s/g low steam line pressure
- Rx trip / SIAS / MSI
- 'A' train only

PZR PRT capture disc OK?

- Complications → Initiator
- 'A' train actuation only for SIAS/MSI/
  - TDAFW started but tripped
  - CHe level - 'A' CHe pump recirculation line had a packing leak, coolant into Aux Bldg.

- s/g press 1000#
- 3 AFW running
- ~~ADV~~ ADV bypasses - MOVs?
- PZR level 52%, PRESSURE NA/NOT.
- 1 CHe pump running

TA brief @ 1:00pm

- PRT level  $\uparrow \leftrightarrow$  PRESS?
- Any indication of PORV / Relief lifting still?
- 1<sup>o</sup> press?
- S/G safeties lifting below setpoint?
  
- What is the pressure in the PRT to rupture a rupture disc?
- Are they considering how to drain the PRT before rupture disc ruptures?
- Status of tampering procedure?
- ERDS status?
- <sup>Any</sup> Unit 2 outage work activities on hold?
- S/G safety lift sequence?
- Termination criteria for alert?
- Cont Sump<sup>W</sup>-leakage?

'B'

12:00  
2<sup>nd</sup>

- ERDS is established
- ~~ETA~~ Licensee brief on OPA.
- S/G code safeties
  - 'B' safety lifted @ 1140°F, setpoint = 1180°F.
  - We have LCO to get to 350°, temp is currently 550°
- TA brief @ 1:00 pm.
- Drawn a 1<sup>st</sup> sample, don't know results yet.
- Still in ES1.1, MSI reset is the only one remaining.

TA Brief 1:00pm

• SRI

• A + 0829, Millstone Unit 3 experienced an automatic RX trip, 'A' train SIAS, and 'A' train MSI from a 'B'  $\frac{1}{16}$  low steam line pressure. Operators followed up with manual initiation.

• S/G safeties on 'B' and 'C' opened & stayed open and then cycled until they stabilized about 15 minutes later. (5 safeties on each S/G).

0842. Operators declared an ALERT based on unisolable stm line leak outside ctmt, due to S/G safeties staying open.

• ~~Pressure has activated~~ One Pressurizer PORV & safety lifted & reseated due to going solid on the p2r from the SI. 2345# was max press reached.  $\frac{press \approx 70 \#}{}$

• PRT level increased to  $\approx 82\%$ , the rupture disc did not rupture.

• Complications to the event -

- "A" train of SI/MSI activated only
- TDAFW pump tripped when called open & has since been reset

- 1 S/G safety lifted  $\approx 40\#$  below setpoint (1140 versus 1180)  
- CH6 - 661, 'A' CH6 pump recirculation line had a packing leak, it has since been isolated. No airborne indications.

• Current Plant Conditions

- Remaining decay heat via the Atmospheric Dump Valve bypass valves.

- P2R level = 53%  $press = 2260\#$   $temp = 550^\circ$   
- progressing toward cooldown.  $sgwl = 69\%$   $press = 1100\#$

35 safeties  
2 PORVs.

- PRT level is 82%, 20#, licensee is in the procedure to reduce temperature <sup>in PRT</sup> and drain the PRT. Rupture disc goes from 86-100#.
- 1080# on all 4 S/G's
- Primary sample drawn, no results yet. Looking @ 5/6 samples.

Current Activities

- Licensee has activated the Site Emergency Response Organization, TSC & EDF are activated.
  - Tampering investigation in progress, no results yet but no indications of tampering so far. (PSC)
  - Remain in ALERT, evaluating termination criteria.
  - Unit 2 Outage activities (PCS draindown below head flange) on hold until they terminate SERO.
  - <sup>ALWAYS</sup> Termination Criteria - Unit pressure back in band (currently @ 14#, normally ~13.5) [Unit chiller stop on an SI]
  - Reviewing TS 3.7.1 - S/G code safety's inoperable - 6 hrs to hot S/G, 6 hrs to hot S/G <sup>200-</sup> 350.
  - Need to investigate initiator, [electronic failure or actual low MS line pressure?]
- Handy:  
 - Continue to monitor plant parameters to cool down.  
 - Need to follow-up on TO AFW

- What is normal PRT level? (40-50%)
- How many S/G safeties lifted in each S/G? <sup>at least 1 S/G safety lifted on 'B' S/G.</sup>
- Location of main steam safeties 'B' & 'C' as compared to each other.
- PRTs operating under water environment - evaluation?
- TS

- Will not meet TS time frame to Hot S/D (<350°F)?

- Limiting 50 °F/HR because they are cooling down via Atmospheric Dump Valve bypass valves.

- 2:00 pm - [Paul Krohn & Pete Habigurst] to talk about site coverage.

[Kevin] call in ask to be put on ET bridge.

- Want to have site people until they get < 200 °F.

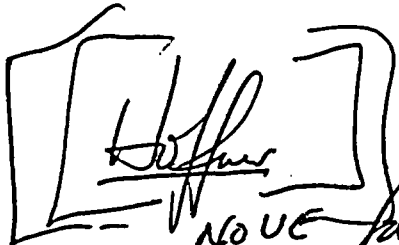
- Send Kevin home, Maurice home. <sup>Maurice back at 10:00pm, 5:00 am.</sup>

- Silas & I stay here until 8:00pm.

- Steve Shaffer drive down to cover the night shift.

2 hours to get to Millstone.

- 8:00 pm turnover - two people on-site.



NOUE for

failure to meet the time.

\* - RCS leakage past PORV, is it RCS leakage to be evaluated under EALs? [39ppm, <255ppm identified]

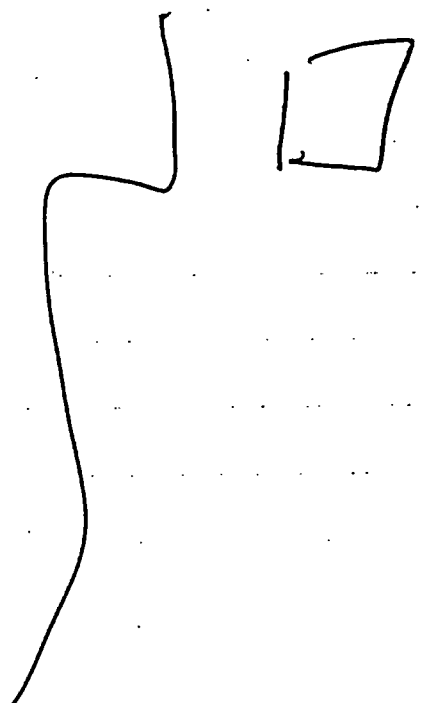
D S/G - normal

sample line CV's did not pass 4 surveillance last week, <sup>entered S/G</sup> isolated

- Utilizing <sup>alternate</sup> secondary sample flow path to get S/G sample (from the BLD line) which is isolated.

- A & D were under cautious purge.
- Can't sample from D S/G.

\* Why are they steaming from D S/G if they don't know the 1°-2° leak potential?





Bill Huffer: 3:20 pm

- Secured 2/4 RCPs & began ppt cooldown.
- Commencing c/D 60°F/1hr
- 4:00 pm evaluate c/D & termination criteria.
- Sampling S/G's - 3/4 S/G's
  - Sample on I was very low
  - D S/G is isolated, in-process of getting sample.
  - Terry Furber has a rad monitor supplied by A, B & D S/G's.
  - Alternate sample line is isolated

Wayne

Working on getting a sample from the B/D line.  
 WR or NR Rad Monitor on TOAD? Similar to MS line Rad Monitor.

- No longer steaming on D.
- Only running A & B RCPs. Not steaming on D S/G because secured OTRP.

Wayne

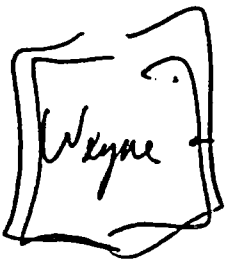
Update on initiating cause? Nothing yet, ERT assigned.  
 Media inquiries? Channel 3 went to Hartford. (CBS)  
 Any new press releases? 2nd press release will be going out shortly.

Paul

Rad Monitor on S/Gs shutdown in-service as you bring back B/D? RM is inop. A & D B/D in-service.



There are other indicators of no 1<sup>o</sup>-2<sup>o</sup> leak:  
• No SGWL A on 'D'  
• No TOAFW RAD MONITOR alarms.



Any RAD MONITOR TEAMS IN THE FIELD? Yes.  
Havent seen anything onsite or offsite.

TSC Brief:

- TSC is investigating unexpected alarms
  - RCP high range leakage flow low alarm (RCP seal leak?)
  - shutdown relief ~~line~~ <sup>valve temp</sup> high
- Temp readings on MSSVs to determine which s/b relief is leaking.

Vision

- No media inquiries into region.
- Still in Maintaining Mode.



Packing leak on chy line valve put out about 1000 gals of H<sub>2</sub>O?

(14)

Scwl = 72%

SG Press = 660 - 680

Res temp = 508

PRZ level = 88%

Res Press = 1390 #

ctmt Press = 13.8 # below TS of 14 #.

PRT = Temp = 180°F

Level = 68%

Press = 38 #.

→ 'A' PORV back valve is closed.

1000 gals

HP now deep in 'A' dry cubicle.

\* → Resetting of MSI signal?

\* → TSC priorities - slo. bld back in service for d/d samples

- Determine which PORV is leaking.

- Investigate unexpected alarms.

- Letdown relief line - both relief valve for letdown & PORV relieve to the PRT. they believe

(20° above ambient)

- Letdown press = 320 #, relief lifts > 6000 #.

4:37 pm  
Bill Helfer:

• c/d going slow, > 60°F/HR

~~termination criteria~~

• Exit Criteria - Initiating condition has cleared.

- 2029 time to hot s/d want to ensure they make this time.

• Not steaming 'D' s/g's, in progress to get sample from 'D' s/g.

\*

• Plant c/d - When they blocked the low pressure in the MSL, the <sup>MSL</sup> signal reset.

- 480°F, another 2 hours to get to < 350°

Chemistry Brief:

→ L/ld valves.

→ Establish open cycle B/D to all s/g's

→ Sample all s/g's, should take 2 hours or all

→ O<sub>2</sub>, H<sub>2</sub>, chemistry, NPDES parameters.

→ Remain in open cycle until hot enough temperature 250-300°F.

→ Maintaining s/g chemistry even though they

'A' PORV

- Still had an SI signal in.
- Bypass valves around ADDs.
- Blocked SI as part of CB procedure, which removed the SI initiator. Have not reset the MS I.

Bill Hoffer

Tailpiece temp - common piping  
 • EQQ dated logger - working on it right now.

- Criteria for exiting the ALERT
- Initiating event is over
- Who will be new communicator?
- They will be in Mode 4 before they deactivate zero.

Hoffer

Have not "reset" MSIV → don't want to lose info.

Acoustic monitors → RT

(5:30 pm)

Control Room Parameters Update.

SGWL = 76%

CD Rate = 60°F/HR.

SG Press = 320#

RCS Temp = 430°F

RZR level = 71%

RCS Press = 730#

Chut Press = 13.8

PRT Temp = 185°F

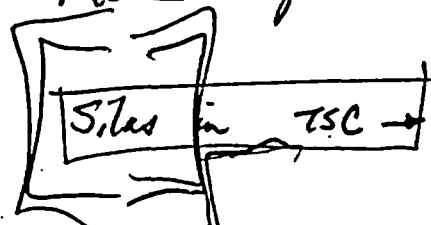
RCP A & B running

Level = 68%

Press = 42#

PRV Valve Condition - 'A' still closed.

MSI Signal - removed but not reset.



Dont report as absolute, if you receive conflicting statements. Report, this is what I knew last & I'll find out. (PRT drain procedure.)

→ If someone asks about something about you have already briefed, simply rebrief. Dont say I already discussed that with the group.

Obviously one of the group was not on line, otherwise they would have asked. All responders need the info. James

Not related to event. SRI to RI technique discussion.

Not Related to event.  
SRI to RI technique info.

(18)

if you don't have enough info.

TSC priorities is a standard briefing subject.  
After you get a detail bundle on things,  
Review their priority list and brief their  
priorities.

- 'A' SSRS fault.
- Have not met MSI yet due to SSRS issue.
- Still don't know why TDAF tripped. feeling using BND AFW pumps.
- 1900 ppm boron currently in RCS.
- Steaming to condense - 2 MSIV bypass don't work
  - 1 S/C doesn't have a RCP
- C403 debrief for <sup>running</sup> off going shift.
- ERT forming.

6:30 pm Update

SOWL = 78%

SG Press = 160#

RCS Temp = 370°F

PZR Level = 70%

RCS Press = 440#

Chant Press = 13.8

CLD Rate = 60°F/HR.

PRT Temp = 185

Level = 68%

Press = 30#

RCP A & B running

PRV conditions - 'B' PRV was isolated

- Downstream fail piece @ 20°F in 25 min

- 'B' determined to be leaking.

- 'B' remains isolated.

- ARP - guidance.

S/G → A, B, C; D to be sampled.

→ D & A will be priorities.



B.M. Offner 7:00 pm

- RCS Temp = 353°
- Termination will go out @ 350° (from ALERT)
- Will make reg'd notifications & phone calls.

7:03 PM Licenses in Mode 4, terminated ALERT.

- S/G samples.
- On RHR, stable & heading toward 200°F.
- Stable C/D.

I'll call Maurice.

1909 - De-escalated from ALERT. HQ notified. ENS secured. 7:03 in Mode 4. SERO/rsc demanned. OCC is the coordination center for follow-up activities.

1950 Peter Habighorst on-site.

- SG Sample B/D alignment working. D's/G Samples.

Contact Reactor Safety Counterpart Link.

(21)

← Results of S/G Sample 'D' K\*

← Shutdown Cooling →

← Region still in monitoring mode. →

[Kevin MacNan] 1:00pm - ~~not~~ report →

2000 HV-22A Problems to reopen to establish  
Blowdown.

2005 - SG WR level. (Walkdown of boards)

82% SG#1

80% SG#2

81% SG#3

80% SG#4

WR - 53% WR

C/D Rate - 31°F/HR.

RCS Temp - 312°F.

RCS Pressure - 348 psig.

AFW flow - D - 8 gpm

76 gpm

90 gpm

86 gpm

B CCP in-board.

(22)

2020-

- 'A' SG aligned for sample. - B/D Sample.  
260°F preps for RHR (312°F) 31°F/HR.  
heps.

2020

ET Briefing

(E. Henschoff) / Dyer -  
(T. McGinty)  
(S. Collins)

Purpose of Call

- Terminate ALERT 7:05.
- Monitoring Mode

Criteria for normal mode.

1. SD cooling. (couple of hours)
  2. Results from chemistry sampling
- D' - No 1°/2° (steaming from this loop)

what are the problems.

- Turbine AFW - No RM alarm.

Site Teams Monitoring (Nothing above background)

'D' SG revised procedures.

Water in letdown Room + ?

0835 +

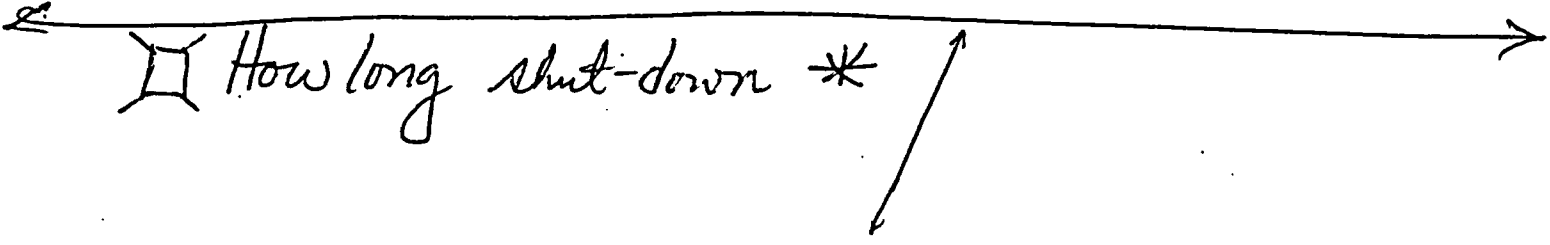
'A' RHR pump in-service.

Quarantine

☐ Cards replaced SG low pressure. - (non)

← Plan to review cards →

☐ PRT parameters → [ ]

☐ How long shut-down \* 

MSIV Bypass.  
PORV  
MS Safety Value.

0500 → RHR in-line  
← Mode 5. →

0850 - 302°F C/D - 11°F/HR

A RHR pump in-service - bron equalization

D → SG → Sampling.

☐ Confirm PRT parameters 0855  
| 160°F  
| 72%  
| 16 psig.  
B PORV Tailpipe - 180°F  
cooldown incl.

(24)

Conf. Call

1- Safety MS.  
Reviewing 2 other safeties  
Eval. PORV - Thu-tomorrow

Plan for Tonight

leakseals.

Min -

like to do -

Consider

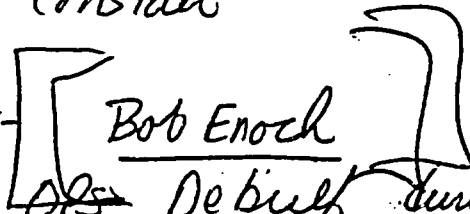


10:30

Mngt Team

Scope Mtg.

ERT-



Bob Enoch

~~OPS~~ Debrief during transient

IC - SSPS - A' Train Card -

SG/Passenger Safety

AFWTT. MS-V'S trip closed <sup>700psig</sup>

2 leaking charging valve - transient (system)

- Do not disturb As-found - / document.

OPS Crew generally positive transient.

Listing of problems

- Cause of R Trip - (EOC)

~ RHR in-service ~ 1hr.

\* 2110 \* SG B/D in-service / samples in progress

Delith Bed working.

25

B' PORV configuration. □ ← Block/PORVs. → VCT →

Charging Values - Quantify water - □ 1,000 gal →  
A/C - 661 Manual → CH-V-8511 B 2' MOV

2114 - 'A' PORV BLOCK OPEN / PORV closed. ✓

2140. -

9:53 pm 'D' S/G. 1 liter sample. ✓  
← (2153) → - No Activity. →  
Tritium still in progress. ↔

10:30. ↔ Call IRC.

← SDC. →

↑ RCS < 350 psig Working toward Start RHR →  
260°F 2nd suction relief for UOP in-service

A RHR →

□ ERDS - Up Running / Last Update / 7:30. ? →  
when devalued licence-sealed data.  
John Rogge / Don Jackson. 11:00pm.

26

OSC. staffing → □

Op. Support Center - U-2 RFD  
Support if necessary  
ERT - over the night.

# 2258 - Started 'A' RHR pump for SDC, working to  
↑ flow to 1,000 gpm then ↑ 500 gpm until to  
4,000 gpm.

2335

2000 gpm. -  
SDC flow.  
'A' RHR pump.

277 °F      365 psig  
← 20 °F/HR cooldown →

Restate: SM. Difficulty → T.

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2345 - Final Discussion with IRC, concluded  
to stand down.  
- RHR in-service  
- RCS cooldown stable.  
left U-3 Control Room.