

June 23, 2005

Mr. Christopher M. Crane  
President and Chief Executive Officer  
AmerGen Energy Company, LLC  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - ISSUANCE OF  
AMENDMENT RE: REACTOR BUILDING CLOSED COOLING WATER PUMP  
AND SERVICE WATER PUMP TRIP CONDITIONS (TAC NO. MC6287)

Dear Mr. Crane:

The Commission has issued the enclosed Amendment No. 255 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station in response to your application dated February 24, 2005.

The amendment revised the Technical Specifications, Section 3.1.1, "Protective Instrumentation Requirements," notes aa and bb, correcting missed wording which led to incorrect statements of the as-designed Service Water pump and Reactor Building Closed Cooling Water system pump trip conditions. The amendment also made an editorial correction to pages 3.6-1 and 3.6-2.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

*/RA/*

Peter S. Tam, Senior Project Manager, Section 1  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures: 1. Amendment No. 255 to DPR-16  
2. Safety Evaluation

cc w/encls: See next page

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Accession Number: **ML051450582**

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AMERGEN ENERGY COMPANY, LLC

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 255  
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by AmerGen Energy Company, LLC, et al. (the licensee), dated February 24, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 255, are hereby incorporated in the license. AmerGen Energy Company, LLC, shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Richard J. Laufer, Chief, Section 1  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: June 23, 2005

ATTACHMENT TO LICENSE AMENDMENT NO. 255

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove

3.1-7  
3.1-18  
3.6-1  
3.6-2

Insert

3.1-7  
3.1-18  
3.6-1  
3.6-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 255

TO FACILITY OPERATING LICENSE NO. DPR-16

AMERGEN ENERGY COMPANY, LCC

OYSTER CREEK NUCLEAR GENERATING STATION (OCNGS)

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated February 24, 2005 (Accession No. ML050590085 in Agencywide Document Access and Management System (ADAMS)), AmerGen Energy Company, LLC (AmerGen, the licensee) requested changes to the Technical Specifications (TSs) for OCNGS. The licensee proposed to revise Table 3.1.1, note aa, which currently states that service water pump (SW) circuit breakers will be tripped in 10 seconds  $\pm 15\%$  during a loss-of-coolant accident (LOCA) by relays SK7A and SK8A, and note bb, which states that reactor building closed cooling water pump (RBCCW) circuit breakers will trip instantaneously during a LOCA. The proposed changes correct these notes by adding the phrase "with a concurrent Loss of Offsite Power (LOOP)" to both notes.

2.0 REGULATORY EVALUATION

The licensee's Amendment No. 42 to the OCNGS licensing application, dated October 23, 1968, concluded that the SW and RBCCW pumps are not required during accident conditions. Therefore, the tripping of these pumps during LOOP with LOCA is acceptable. Subsequently, operating license Amendment No. 60 (dated February 3, 1982; Accession No. 8202110372) authorized changes in the diesel generator load sequence timers for the SW and RBCCW pumps; this amendment also added notes aa and bb. As they stand, notes aa and bb do not correctly reflect the as-designed and as-licensed condition of OCNGS. Specifically, notes aa and bb imply that the SW and RBCCW pumps will trip upon a LOCA, despite the fact that by design, they only trip in a LOCA if there is concurrent LOOP.

3.0 TECHNICAL EVALUATION

The Nuclear Regulatory Commission (NRC) staff reviewed the licensee's application. In addition, the NRC staff reviewed the information about these pumps contained in Table 8.3-1, "Emergency Buses Automatic Loading Schedule," of the Oyster Creek Updated Final Safety Analysis Report. Table 8.3-1 shows that the SW pump and RBCCW pump are not loaded on to the emergency diesel generator (EDG) during a LOOP concurrent with a LOCA. The table

also shows that during a LOOP, the SW pump starts 120 seconds after close of the EDG breaker, and the RBCCW pump starts 166 seconds after close of the EDG breaker. Accordingly, the NRC staff confirms that notes aa and bb did inadvertently misrepresent the design of the SW and RBCCW systems.

Based on this review, the NRC agrees that the inadvertent wording errors introduced by Amendment No. 60 can be corrected by revising notes aa and bb, respectively, to read as follows (words added are shown in *italics*):

Pump circuit breakers will be tripped in 10 seconds  $\pm 15\%$  during a LOCA *with a concurrent Loss of Offsite Power (LOOP)* by relays SK7A and SK8A.

Pump circuit breakers will be tripped instantaneously during a LOCA *with a concurrent Loss of Offsite Power (LOOP)*.

The Bases description associated with the RBCCW and SW pump trip requirements (on page 3.1-7 of the TS) is also revised to reflect the new wording for notes aa and bb. The NRC staff finds the wording change consistent with the change to notes aa and bb.

The licensee also proposed to correct an administrative error on page 3.6-1. Specifically, the footnote belongs to TS Section 3.6.A.4.1, which is located on page 3.6-2. The proposed change involves relocating the footnote from page 3.6-1 to 3.6-2, with no change in any TS text. The NRC staff agrees that this change is purely editorial and is, therefore, acceptable.

Based on the above review, and the fact that the proposed amendment involves no design changes to the unit, and no changes to the methods of operation of plant equipment, the NRC staff finds the proposed amendment acceptable.

#### 4.0 STATE CONSULTATION

The State of New Jersey notified (letter, K. W. Tosch to NRC Document Control Desk, dated April 11, 2005; Accession No. ML051370114) the NRC staff that it had no comments on the licensee's proposed amendment. This letter satisfied NRC's regulatory requirement to notify the State official regarding the proposed issuance of the amendment.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes requirements with respect to use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (70 FR 15941). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Tam

Date: June 23, 2005

Oyster Creek Nuclear Generating Station

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