May 12, 2005

L-MT-05-035
10 CFR Part 50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Monticello Nuclear Generating Plant
Docket No. 50-263
License No. DPR-22

LER 2005-004, - Voluntary LER for "Control Rod Drive Insert Line Leakage"

A Voluntary Licensee Event Report for this occurrence is attached.

This letter makes no new commitments or changes any existing commitments.

John T. Conway for TJP

Thomas J. Palmisano
Site Vice President, Monticello Nuclear Generating Plant
Nuclear Management Company, LLC

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Monticello, USNRC
Resident Inspector, Monticello, USNRC

NRC FORM 366 (6-2004) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 EXPIRES 6-30-2007

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Monticello Nuclear Generating Plant	05000263	1 of 2

TITLE (4) Voluntary LER for "Control Rod Drive Insert Line Leakage"

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER 05000
03	09	2005	2005	– 004	– 00	05	12	2005	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	N
POWER LEVEL (10)	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) (11)

	20.2201(b)	20.2203(a)(3)(ii)	50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)
	20.2201(d)	20.2203(a)(4)	50.73(a)(2)(iii)		50.73(a)(2)(x)
	20.2203(a)(1)	50.36(c)(1)(i)(A)	50.73(a)(2)(iv)(A)		73.71(a)(4)
	20.2203(a)(2)(i)	50.36(c)(1)(ii)(A)	50.73(a)(2)(v)(A)		73.71(a)(5)
	20.2203(a)(2)(ii)	50.36(c)(2)	50.73(a)(2)(v)(B)	X	OTHER Specify in Abstract below or in NRC Form 366A
	20.2203(a)(2)(iii)	50.46(a)(3)(ii)	50.73(a)(2)(v)(C)		
	20.2203(a)(2)(iv)	50.73(a)(2)(i)(A)	50.73(a)(2)(v)(D)		**Voluntary**
	20.2203(a)(2)(v)	50.73(a)(2)(i)(B)	50.73(a)(2)(vii)		
	20.2203(a)(2)(vi)	50.73(a)(2)(i)(C)	50.73(a)(2)(viii)(A)		
	20.2203(a)(3)(i)	50.73(a)(2)(ii)(A)	50.73(a)(2)(viii)(B)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Ron Baumer	763-295-1357

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX		CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
B	AA	PSP		No						

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT

A voluntary License Event Report is being provided for the following event:

On March 5, 2005 with the Monticello Nuclear Generating plant in a refueling outage, during a scheduled under-vessel walk down, a small water leak was noted on a control rod drive flange. On March 9, Nuclear Management Company (NMC) determined that the leakage was from the top surface of the flange adjacent to the Control Rod Drive (CRD) insert line. The leakage was determined to be active and the result of a flaw in the insert line to CRD flange weld. Metallurgical examination determined that the leak initiated at a small weld void in the weld between the insert line and the CRD housing flange created during original construction. Corrective action taken included performance of a metallurgical, chemical and visual examination of the leaking section of the CRD insert line, replacement of a section of the insert line to re-make the weld at the CRD flange, and a 100 percent visual inspection of the other control rod drive flanges for leakage.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Monticello Nuclear Generating Plant	05000263	2005	– 004	– 00	2 of 2

TEXT *(If more space is required, use additional copies of NRC Form 366A)* **(17)**

Description

On March 5, 2005 with the Monticello Nuclear Generating plant in a refueling outage, during a scheduled primary containment under vessel [RPV] walk down, a small water leak was noted on a control rod drive (CRD) [AA] flange. On March 9, Nuclear Management Company (NMC) determined that the leakage was from the top surface of the flange adjacent to the CRD insert line [PSP]. The leakage was determined to be active and the result of a flaw in the insert line to flange weld. The CRD was isolated to prevent movement and the drive was considered inoperable until the leak was repaired. A portion of the insert piping was removed for examination and replaced under a station modification. Following repairs, the CRD was tested and restored to service.

Event Analysis

The event was not reportable in accordance with 10 CFR 50.72 or 10 CFR 50.73, however in accordance with NUREG-1022 revision 2, a voluntary Licensee Event Report is being made for this event. The event is not classified as a safety system functional failure.

Safety Significance

The safety significance of this issue was low since the CRD was always capable of inserting into the reactor. In the event that the insert line severed at reactor power, the CRD could be inserted by using reactor pressure alone. The safety function of the CRD system would not be affected. The CRD insert lines are not part of the reactor coolant pressure boundary. A risk analysis determined that the effect on the Core Damage Frequency is less than 1 X 10-7 per year. This event had no adverse affect on the health and safety of the pubic.

Cause

Metallurgical examination of the pipe determined that the leak was the result of a small weld void in the weld between the insert line and the CRD housing flange created during original construction.

Corrective Action

A short section of pipe was removed and replaced under a station modification. A visual inspection of all CRD insert and withdrawal piping connections at the CRD flange was performed and no other leakage was observed.

Failed Component Identification

Pipe Manufacturer: Greenville Tubes, Inc (Greenville, PA)

Previous Similar Events

A review of the station corrective action program and industry databases found the following similar events: Monticello (LER 1998-03), Brunswick (LER 2-88-007), and Duane Arnold (LER 88-019). Each of these events involved leakage from either the CRD insert or withdrawal lines, however in each case, the leakage was attributed to a corrosion failure mechanism of the pipe. The event discussed in LER 2005-04 was due to a weld void created during original construction.