UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

May 11, 2005

Carolina Power and Light Company
ATTN: Mr. James Scarola
Vice President - Harris Plant
Shearon Harris Nuclear Power Plant
P. O. Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: NOTIFICATION OF TRIENNIAL FIRE PROTECTION BASELINE INSPECTION (NRC INSPECTION REPORT 05000400/2005007)

Dear Mr. Scarola:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a triennial fire protection baseline inspection of the Shearon Harris Nuclear Power Plant Unit 1 in August 2005. The inspection team will be led by Mr. McKenzie Thomas, NRC Senior Reactor Inspector, of the Region II Office. The team will be composed of personnel from the NRC Region II Office. The inspection will be conducted in accordance with the NRC's baseline fire protection inspection procedure 71111.05T.

The inspection objective will be to evaluate your fire protection program implementation with emphasis on post-fire safe shutdown capability and the fire protection features provided to ensure at least one post-fire safe shutdown success path is maintained free of fire damage. The inspection team will focus their review on the separation of the systems and equipment necessary to achieve and maintain safe shutdown and fire protection features of selected fire areas.

On May 9, 2005, during a telephone conversation between Mr. Dave Corlett, Supervisor, Licensing/Regulatory Programs, and Mr. Thomas, our respective staffs confirmed arrangements for a three-day information gathering site visit and a two-week onsite inspection. The schedule for the inspection is as follows:

- Information gathering visit: Week of July 25-29, 2005
- Week 1 of onsite inspection: August 15-19, 2005
- Week 2 of onsite inspection: August 29 - September 2, 2005

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection and to become familiar with the Harris Nuclear Plant fire protection program features, post-fire safe shutdown capabilities, and plant layout. The types of documents the team will be interested in reviewing, and possibly obtaining, are listed in the Enclosure. Please contact Mr. Thomas prior to preparing copies of the materials listed in the Enclosure. The inspection team will try to minimize your administrative burden by specifically identifying those documents required for inspection preparation.

During the information gathering visit, the team will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for reactor site access (including radiation protection training, security, safety and fitness for duty requirements); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the onsite inspection weeks you ensure that copies of analyses, evaluations, or documentation regarding the implementation and maintenance of the Harris Nuclear Plant fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the Harris Nuclear Plant fire protection program and its implementation.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection or the inspection team's information or logistical needs, please contact Mr. Thomas at (404) 562-4673, or me at (404) 562-4669.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Should you have any questions concerning this letter, please contact us.

Sincerely,

**\\RA**

D. Charles Payne, Chief
Engineering Branch 2
Division of Reactor Safety

Docket No.: 50-400
License No.: NPF-63

Enclosure: Triennial Fire Protection Inspection Support Documentation

cc w/encl: (See page 3)

cc w/encl:
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**Triennial Fire Protection Inspection Support Documentation**

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit. The current version of these documents is expected unless specified otherwise. Electronic media is preferred, if readily available. (The preferred file format is searchable ".pdf" files on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Please provide 5 copies of each CDROM submitted.) Information in "lists" should contain enough information to be easily understood by someone who has a knowledge of the technology. The lead inspector will discuss specific information needs with the licensee staff and may request additional documents.

1. The current version of the Fire Protection Program and Fire Hazards Analysis.

2. The fire protection program implementing procedures (e.g., administrative controls, surveillance testing, fire brigade).

3. Fire brigade training program document and the pre-fire plans for the selected fire areas/zones (to be determined during information gathering visit).

4. The post-fire safe shutdown analysis, including system and separation analyses.

5. The alternative shutdown analysis.

6. Piping and instrumentation (flow) diagrams for the fire suppression systems.

7. Piping and instrumentation (flow) diagrams for the systems and components used to achieve and maintain hot standby, and cold shutdown, for fires involving shutdown from the control room.

8. Piping and instrumentation (flow) diagrams for the systems and components used to achieve and maintain hot standby, and cold shutdown, for fires in areas requiring alternative or dedicated shutdown capability.

9. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.

10. Plant layout drawings which identify plant fire area and/or fire zone delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment for the selected fire areas/zones (to be determined during information gathering visit).

11. Plant layout drawings which identify the general location of the post-fire emergency lighting units.

12. Plant operating procedures used to, and describing, shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room.

13. Plant operating procedures used to implement the alternative shutdown capability from outside the control room with a fire in either the control or cable spreading room (or any other alternative shutdown area).

14. Maintenance and surveillance testing procedures for alternative shutdown capability (including Appendix R emergency lights and communication systems) and fire barriers, detectors, fire pumps and fire suppression systems.

15. Calculations and/or justifications that verify fuse/breaker coordination for the selected fire areas/zones (to be determined during information gathering visit) that are fed off the same electrical buses as components in the protected train. Also, a list of the maintenance procedures used to routinely verify fuse/breaker coordination in accordance with the post-fire safety shutdown coordination analysis.

16. A list of the significant fire protection and post-fire safe shutdown design change descriptions (including their associated 10 CFR 50.59 evaluations) and Generic Letter (GL) 86-10 evaluations.

17. A list of the protection methodologies (as identified in 10 CFR Part 50, Appendix R, Section III.G) used to achieve regulatory compliance for the selected fire areas/zones (to be determined during information gathering visit). That is, please specify whether 3-hour rated fire barriers (Section III.G.2.a), 20-foot separation along with detection and suppression (Section III.G.2.b), 1-hour rated fire barriers with detection and suppression (Section III.G.2.c), or alternative dedicated shutdown capability (Section III.G.3) is used for the selected fire areas/zones.

18. Procedures or instructions that govern the implementation of plant modifications, temporary modifications, maintenance, and special operations, and their impact on fire protection.

19. Organization chart(s) of site personnel down to the level of the fire protection staff.

20. Procedures or instructions that control the configuration of the fire protection program, features, and post-fire safe shutdown methodology and system design.

21. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of code deviations (i.e., a listing of the NFPA code editions committed to (Code of Record)).

22. The three most recent fire protection QA audits and/or fire protection self-assessments.

23. A list of open and closed fire protection problem identification and resolution reports [also know as action requests/condition reports/problem reports/problem investigation reports/NCRs/EARs] associated with fire protection or Appendix R safe shutdown for the past three years.

24. A list of plant fire protection licensing basis documents (i.e., a list of the SERs and change evaluations which form the licensing basis for the facility's post-fire safe shutdown configuration).

25. A list of fire protection or Appendix R calculations.

26. A list of fire impairments identified during the previous year.

27. A list of abbreviations/designators for plant systems.