

May 9, 2005

Mr. Christopher M. Crane
President and Chief Executive Officer
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) RE: THREE MILE ISLAND
NUCLEAR STATION, UNIT 1 (TMI-1) RESPONSE TO GENERIC LETTER (GL)
2004-01, "REQUIREMENTS FOR STEAM GENERATOR TUBE INSPECTION,"
DATED AUGUST 30, 2004 (TAC NO. MC4859)

Dear Mr. Crane:

By letter dated October 29, 2004 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML043060328), AmerGen Energy Company, LLC (the licensee), submitted its response to GL 2004-01 for TMI-1.

The Nuclear Regulatory Commission staff has determined that it will need the additional information identified in the enclosure to this letter in order to complete its review of the subject GL. As discussed with and agreed to by your staff, please provide the requested information within 30 days of receipt of this letter. If you have any questions, please contact me at (301) 415-1402.

Sincerely,

/RA/

Timothy G. Colburn, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: RAI

cc w/encl: See next page

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ACCESSION NO.: ML051180042 *Input received. No substantive changes made.

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REQUEST FOR ADDITIONAL INFORMATION (RAI)
RELATED TO AMERGEN ENERGY COMPANY, LLC
THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1)
GENERIC LETTER (GL) 2004-01 RESPONSE
DOCKET NO. 50-289

By letter dated October 29, 2004, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML043060328) AmerGen Energy Company, LLC, the licensee for TMI-1, submitted a response to GL 2004-01, "Requirements For Steam Generator Tube Inspection," dated August 30, 2004. The Nuclear Regulatory Commission (NRC) staff has reviewed the licensee's letter and has determined that additional information is needed in order for the NRC staff to complete its review. Responses to the following questions are requested:

1. The October 29, 2004, response to GL 2004-01 indicated that the TMI-1 steam generator (SG) tube inspection approach/methods are consistent with the NRC staff's position provided in the GL. In Table 1, Note 6, you state that examination results from other once-through SG plants in the lower tubesheet sludge pile region have indicated that bobbin coil detection capability may be reduced in this region. You further state that there is no evidence to date, that TMI-1 is experiencing a reduction in the bobbin coil probability of detection (POD) in this region. Please provide the basis for this statement considering that licensees for other plants with similarly designed SGs have stated that tubes in the kidney region require the use of a rotating pancake coil probe for better detection of intergranular attack (IGA) due to sludge build up in the upper portion of the lower tubesheet crevice. In addition, for each indication within the sludge pile and in other areas of the lower tubesheet (excluding tube end cracking), provide a summary that includes the indication's location, the nature of the indication (IGA, stress-corrosion cracking (SCC), etc.), the indication's severity (length and depth), and whether the indication was detected by bobbin coil inspection, rotating pancake coil probe inspection, or both. Discuss whether rotating pancake coil probe inspections detected indications that were not detected by the bobbin coil probe, and if so, the implications of this finding. Discuss if the bobbin coil technique used in the lower tubesheet region meets the industry standards (e.g., Appendix H of the Electric Power Research Institute "Pressurized-Water Reactor Steam Generator Examination Guidelines: Revision 6," ADAMS Accession No. ML050980251). If this technique does not meet the industry standards, discuss your future plans for qualifying the inspection technique. If this technique does not meet industry standards and you have no plans to qualify the technique to an industry standard, provide the acceptance standard, specification or criteria you are using and the technical basis for this standard, specification, or criteria to ensure the adequacy of this technique.

Enclosure

2. If as a result of your response to the questions above, you conclude that full compliance with the TMI-1 Technical Specifications in conjunction with Criteria IX, XI, and XVI of Title 10 of the *Code of Federal Regulations*, Part 50, Appendix B, requires corrective actions, please discuss your proposed corrective actions as requested by GL 2004-01, Requested Information #2. In addition, if the inspections are not being performed consistent with the NRC position on the requirements, please submit a safety assessment as requested in GL 2004-01, Requested Information #3.

Three Mile Island Nuclear Station, Unit 1

cc:

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Three Mile Island Nuclear Station, Unit 1

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