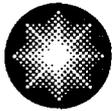


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Constellation Energy

R.E. Ginna Nuclear Power Plant

April 8, 2005

Ms. Donna M. Skay
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555-0001

Dear Mrs. Skay:

The purpose of this letter is to inform the NRC that certain regulatory commitments, as enumerated in Appendix A of the Ginna Station License Renewal Safety Evaluation Report, NUREG-1786, were not completed as stated. Specifically:

Item Number 1: Submit new pressure-temperature limit curves by December 2004.

The reason for this commitment was that Ginna desired to use revised methodology utilizing ASME Code Case N-641, as submitted via WCAP-15885. However, our current PTLR methodology listed in WCAP-14684 extends out to 32 EFPY (we are currently approaching 28 EFPY). Our current plan is to continue to use that methodology until a license amendment request using WCAP-15885 is submitted by Ginna and approved by the NRC.

Item Number 24: Perform hardness tests, if feasible, on emergency diesel generator jacket water coolers and lube oil coolers channel heads, during the 2005 RFO.

This test was planned to be performed in concert with maintenance of the diesel generator, originally scheduled for this refueling outage. Maintenance of the EDG's will be performed on-line during 2005. This commitment will be met at that time.

Item Number 34: Perform joint resistance tests when visual inspections of PVC boots or other materials of construction indicate that the joint may be overheating.

This commitment was made in our September 16, 2003 RAI response letter with a due date of "prior to September 2009". The July 2004 due date in the SER is a typographical error.

Item Number 38: Withdraw surveillance capsule in Spring 2005 and submit test report of results within one year, in accordance with 10 CFR 50, App. H, paragraph IV.A.

This commitment was made prior to the decision to pursue power uprate. The increased fluence associated with higher power level requires additional capsule residence time in the reactor vessel. A new withdrawal schedule, consistent with ASTM E-185 will be submitted after fluence levels associated with uprate are finalized.

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Item Number 39: Inspect to VT-1 quality the stainless steel fillet weld joining the bottom mounted instrument (BMI) guide tube to the end of each BMI penetration, as well as the 82/182 weld between the SS safe end and the lower penetration nozzle, each refueling outage beginning in 2005.

This commitment was made to complement the inspections which were planned to be performed on the reactor vessel bottom-mounted instrumentation penetrations per the requirements of Bulletin 2003-02. While performing these inspections, Ginna personnel were to inspect these welds for purposes of license renewal aging management. Inspection difficulties prevented the completion of these activities to VT-1 quality standards, though a general visual inspection of this area provided Ginna with confidence that there is no boric acid leakage from these welds.

As a result of our inspection activities on the BMI penetrations during the 2005 refueling outage, we are re-evaluating our inspection strategy in response to Bulletin 2003-02. This information will be provided to the NRC prior to the 2006 refueling outage.

Very truly yours,

Mary G. Kolsnick

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