ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One or more Functions with two required channels inoperable.	C.1 Restore one channel to OPERABLE status.	7 days
D. Required channel of Functions 18, 21, 24, or 25 inoperable.	D.1 Restore required channel to OPERABLE status.	7 days
E. Required Action and associated Completion Time of Condition C or D not met.	E.1 Enter the Condition referenced in Table 3.3.11-1 for the channel.	Immediately
F. As required by Required Action E.1 and referenced in Table 3.3.11-1.	F.1 Be in MODE 3. AND F.2 Be in MODE 4.	6 hours 12 hours
G. As required by Required Action E.1 and referenced in Table 3.3.11-1.	G.1 Prepare and submit a Special Report to the NRC in accordance with Specification 5.7.2.	30 days

SURVEILLANCE REQUIREMENTS

----------NOTE----------
These SRs apply to each PAMI Function in Table 3.3.11-1, with exceptions noted.

	SURVEILLANCE	FREQUENCY
SR 3.3.11.1	Perform CHANNEL CHECK for Function 9.	12 hours
SR 3.3.11.2	Perform CHANNEL CHECK for each required instrumentation channel, except Function 9, that is normally energized.	31 days
SR 3.3.11.3	Perform CHANNEL FUNCTIONAL TEST for function 9.	31 days
SR 3.3.11.4	Perform CHANNEL CALIBRATION, for functions 2,3,14,15,16,17, and 20.	18 months
SR 3.3.11.5	Perform CHANNEL CALIBRATION for functions 1,4,5,6,7,8,9,11,12,13,18, 19,21,22,23,24,25,26, and 27.	24 months

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

	FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1.	Excore Neutron Flux	2	F
2.	Reactor Coolant System Hot Leg Temperature	2 (1 per steam generator)	F
3.	Reactor Coolant System Cold Leg Temperature	2 (1 per steam generator)	F
4.	Reactor Coolant System Pressure (wide range)	2	F
5.	Reactor Vessel Water Level	2[d]	G
6.	Containment Water Level (wide range)	2	F
7.	Containment Pressure (wide range)	2	F
8.	Containment Isolation Valve Position	2 per penetration flow path[a][b]	F
9.	Containment Area Radiation (high range)	2	G
10.	Deleted		
11.	Pressurizer Level	2	F
12.	Steam Generator Water Level (wide range)	2 per steam generator	F
13.	Condensate Storage Tank Level	2	F
14.	Core Exit Temperature – Quadrant 1	2[c]	F
15.	Core Exit Temperature – Quadrant 2	2[c]	F
16.	Core Exit Temperature – Quadrant 3	2[c]	F
17.	Core Exit Temperature – Quadrant 4	2[c]	F
18.	Auxiliary Feedwater Flow	1 per steam generator	F
19.	Containment Pressure (narrow range)	2	F
20.	Reactor Coolant System Subcooling Margin Monitor	2	F
21.	Pressurizer Safety Valve Position Indication	1 per valve	F
22.	Containment Temperature	2	F
23.	Containment Water Level (narrow range)	2	F
24.	HPSI Flow Cold Leg	1 per cold leg	F
25.	HPSI Flow Hot Leg	1 per hot leg	F
26.	Steam Line Pressure	2 per steam generator	F
27.	Refueling Water Storage Tank Level	2	F

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

5.7 Reporting Requirements (continued)

5.7.2 Special Reports

Special Reports may be required covering inspection, test, and maintenance activities. These special reports are determined on an individual basis for each unit and their preparation and submittal are designated in the Technical Specifications.

Special Reports shall be submitted to the U. S. Nuclear Regulatory Commission, Attention: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, in accordance with 10 CFR 50.4 within the time period specified for each report.

The following Special Reports shall be submitted:

a. When a pre-planned alternate method of monitoring post-accident instrumentation functions is required by Condition B or Condition G of LCO 3.3.11, a report shall be submitted within 30 days from the time the action is required. The report shall outline the action taken, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the function to OPERABLE status.

b. Any abnormal degradation of the containment structure detected during the tests required by the Pre-Stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.

c. Following each inservice inspection of steam generator (SG) tubes, in accordance with the SG Tube Surveillance Program, the number of tubes plugged and tubes sleeved in each SG shall be reported to the NRC within 15 days. The complete results of the SG tube inservice inspection shall be submitted to the NRC within 12 months following the completion of the inspection. The report shall include:

   1. Number and extent of tubes and sleeves inspected, and

   2. Location and percent of wall-thickness penetration for each indication of an imperfection, and

   3. Identification of tubes plugged and tubes sleeved.

(continued)