

April 6, 2005

MEMORANDUM TO: Biweekly Notice Coordinator

FROM: Stephen R. Monarque, Project Manager, Section 1 /RA/  
Project Directorate II  
Division of Licensing Project Management

SUBJECT: REQUEST FOR PUBLICATION IN BI-WEEKLY FR NOTICE -  
NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS  
TO FACILITY OPERATING LICENSES, PROPOSED NO  
SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION,  
AND OPPORTUNITY FOR A HEARING (TAC NOS. MC6435 AND  
MC6436)

Virginia Electric and Power Company, Docket Nos. 50-280 and 50-281, Surry Power  
Station, Unit Nos. 1 and 2, Surry County, Virginia

Date of amendment request: March 17, 2005

Description of amendment request: The proposed change would incorporate a license  
condition that would permit irradiation of the fuel assemblies to a lead rod average burnup  
of 62,000 MWD/MTU.

Basis for proposed no significant hazards consideration determination: As required by  
10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant  
hazards consideration, which is presented below:

1. The probability of occurrence or the consequences of an accident previously  
evaluated is not significantly increased.

For most of the accidents analyzed in the UFSAR [Updated Final Safety  
Analysis Report] (e.g., LOCA [loss-of-coolant accident], Steam Line Break,  
etc.) the fuel design has no impact on the likelihood of initiation of an  
accident. Fuel performance is evaluated as a consequence of the accident.  
The only accident where the fuel design may have an impact on the  
likelihood of a Chapter 14 accident is the Fuel Handling Accident discussed  
in Chapter 14.4.1 of the Surry UFSAR. The activity being evaluated is a  
slight increase in the lead rod average burnup limit for the fuel assemblies.  
No change in fuel design or fuel enrichment will be required to increase the  
lead rod average burnup. The fuel rods at the extended lead rod average

burnup will continue to meet the design limits with respect to fuel rod growth, clad fatigue, rod internal pressure and corrosion. Thus, there will be no impact on the capability to engage the fuel assemblies with the handling tools. Therefore, it is concluded that the change will not result in more than a minimal increase in the frequency of occurrence of any accident previously evaluated in the UFSAR. The impact of extending the lead rod average burnup to 62,000 MWD/MTU from 60,000 MWD/MTU on the Core Kinetics Parameter, Core Thermal-Hydraulics/DNBR [Departure from Nucleate Boiling Ratio], Specific Accident Considerations, and Radiological Consequences was considered. Based on the evaluation of these considerations, it is concluded that increasing the lead rod average burnup limit to 62,000 MWD/MTU will not result in a significant increase in the consequences of the accidents previously evaluated in the Surry UFSAR.

2. The possibility for a new or different type of accident from any accident previously evaluated is not created.

The fuel is the only component affected by the change in the burnup limit. The change does not affect the thermal hydraulic response to any transient or accident. The fuel rod design criteria [will] continue to be met at the higher burnup limit. Thus, the change does not create the possibility of an accident of a different type.

3. The margin of safety as defined in the Bases to the Surry Technical Specifications is not significantly reduced.

The operation of the Surry cores with a limited number of fuel assemblies with some fuel rods irradiated to a lead rod average burnup of 62,000 MWD/MTU will not change the performance requirements of any system or component such that any design criteria will be exceeded. The normal limits on core operation defined in the Surry Technical Specifications will remain applicable for the irradiation of the fuel to a lead rod average burnup of 62,000 MWD/MTU. Therefore, the margin of safety as defined in Bases to the Surry Technical Specifications is not significantly reduced.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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