

April 27, 2005

Mr. D. M. Jamil
Vice President
Catawba Nuclear Station
Duke Energy Corporation
4800 Concord Road
York, SC 29745

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 RE: ISSUANCE OF
AMENDMENTS (TAC NOS. MC3823 AND MC3824)

Dear Mr. Jamil:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 223 to Renewed Facility Operating License NPF-35 and Amendment No. 218 to Renewed Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated July 19, 2004, as supplemented by letters dated March 8 and March 22, 2005.

The amendments revise TS 3.8.4, "DC Sources - Operating" and TS 3.8.6, "Battery Cell Parameters" to allow for the replacement of the existing nickel cadmium diesel generator batteries with conventional lead acid batteries.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Sean E. Peters, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 223 to NPF-35
2. Amendment No. 218 to NPF-52
3. Safety Evaluation

cc w/encls: See next page

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DISTRIBUTION: See next page

Package Number:

NRR-058

Amendment Number: ML050930033

Tech Spec No.:

*No Significant Changes to SE

* * No legal objection

OFFICE	PDII-1/PM	PDII-1/LA	EEIB/SC*	IROB/SC	OGC**	PDII-1/SC
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DATE	03/ 29 /05	04/ 06 /05	03/ 24 /05	04/15/05	04/ 22 /05	4/27/05

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SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 RE: ISSUANCE OF
AMENDMENTS (TAC NOS. MC3823 AND MC3824)

Date: April 27, 2005

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DUKE ENERGY CORPORATION
NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION
SALUDA RIVER ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-413
CATAWBA NUCLEAR STATION, UNIT 1
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 223
Renewed License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Renewed Facility Operating License No. NPF-35 filed by the Duke Energy Corporation, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated July 19, 2004, as supplemented by letters dated March 8 and March 22, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 223, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 27, 2005

DUKE ENERGY CORPORATION
NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1
PIEDMONT MUNICIPAL POWER AGENCY
DOCKET NO. 50-414
CATAWBA NUCLEAR STATION, UNIT 2
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 218
Renewed License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Renewed Facility Operating License No. NPF-52 filed by the Duke Energy Corporation, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated July 19, 2004, as supplemented by letters dated March 8 and March 22, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 218, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 27, 2005

ATTACHMENT TO LICENSE AMENDMENT NO. 223

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND LICENSE AMENDMENT NO. 218

RENEWED FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix A Technical Specifications and Associated Bases with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.8.4-2	3.8.4-2
3.8.4-3	3.8.4-3
3.8.6-1	3.8.6-1
3.8.6-2	3.8.6-2
3.8.6-3	3.8.6-3
3.8.6-4	3.8.6-4
-----	3.8.6-5
B 3.8.4-5	B 3.8.4-5
B 3.8.4-6	B 3.8.4-6
B 3.8.4-7	B 3.8.4-7
B 3.8.4-8	B 3.8.4-8
B 3.8.6-1	B 3.8.6-1
B 3.8.6-2	B 3.8.6-2
B 3.8.6-3	B 3.8.6-3
B 3.8.6-4	B 3.8.6-4
B 3.8.6-5	B 3.8.6-5
B 3.8.6-6	B 3.8.6-6
B 3.8.6-7	B 3.8.6-7

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 223 TO RENEWED FACILITY OPERATING
LICENSE NPF-35 AND
AMENDMENT NO. 218 TO RENEWED FACILITY OPERATING LICENSE NPF-52
DUKE ENERGY CORPORATION, ET AL.
CATAWBA NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated July 19, 2004, as supplemented by letters dated March 8 and March 22, 2005, Duke Energy Corporation, et al. (DEC, the licensee), submitted a request for changes to the Catawba Nuclear Station (Catawba), Units 1 and 2, Technical Specifications (TS). The requested changes would revise TS 3.8.4, "DC Sources - Operating" and TS 3.8.6, "Battery Cell Parameters" to allow for the replacement of the existing nickel cadmium diesel generator (DG) batteries with conventional lead acid batteries.

2.0 REGULATORY EVALUATION

The regulatory requirements that the Nuclear Regulatory Commission (NRC) staff applied in its review of the application include:

Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants," General Design Criterion (GDC) 17, "Electric power systems," requires, in part, that nuclear power plants have onsite and offsite electric power systems to permit the functioning of structures, systems, and components that are important to safety. The onsite system is required to have sufficient independence, redundancy, and testability to perform its safety function, assuming a single failure. The offsite power system is required to be supplied by two physically independent circuits that are designed and located so as to minimize, to the extent practical, the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. In addition, this criterion requires provisions to minimize the probability of losing electric power from the remaining electric power supplies as a result of loss of power from the unit, the offsite transmission network, or the onsite power supplies.

GDC 18, "Inspection and testing of electric power systems," requires that electric power systems that are important to safety must be designed to permit appropriate periodic inspection and testing.

10 CFR 50.36(c)(3), "Technical Specifications," requires a licensee's TSs to have Surveillance Requirements (SRs) relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operations are within safety limits, and that the limiting conditions for operation will be met.

Regulatory Guide 1.32, "Criteria For Safety-Related Electric Power Systems For Nuclear Power Plants," provides guidance for complying with GDC 17 and 18 with respect to design, operation, and testing of safety-related electric power systems of all types of nuclear power plants.

3.0 TECHNICAL EVALUATION

At Catawba, the 125 volt direct current (dc) Diesel Essential Auxiliary Power System provides a separate and independent train of 125 volt dc power to each DG. Each train consists of a 125 volt dc battery and a battery charger powered from its associated train of 600 volt essential auxiliary power. The present DG batteries are nickel cadmium Class IE batteries. These batteries are sized to carry their assigned loads for 2 hours. Each battery charger normally supplies its associated DG control panel loads while maintaining a float charge on its associated battery. Each DG battery is available to assume its associated loads upon the loss of its battery charger or alternating current (ac) power source.

In addition to its primary function of supplying dc power to the DG auxiliaries, the Catawba DG batteries have the capability to serve as an alternate dc supply for certain safety related distribution centers that support plant shutdown capability. To assure high availability of power and to protect against the loss of dc power due to a fire in the control complex, the loads required for plant shutdown are supplied power from two sources through auctioneering diode assemblies. These loads include the auxiliary feedwater pump turbine controls, which are required to have power available for a minimum of 2 hours. The two 125 volt dc sources are the 125 volt dc Vital Instrumentation and Control Power System and the 125 volt dc Essential Diesel Auxiliary Power System. The DG batteries therefore have the capability to serve two different safety related functions.

Catawba has a history of DG battery capacity degradation that has resulted in the replacement of the DG batteries every third refueling outage. According to the licensee, the current DG batteries have shown signs of excessive capacity degradation, as measured during the last several capacity tests. DEC performs battery capacity testing on each DG battery once per fuel cycle. The licensee attributes the excessive capacity degradation to the continuous elevated temperature environment in the DG rooms. The licensee attempted to solve the elevated temperature problem by adding ventilation to the DG batteries, however, the effort was unsuccessful in extending the battery life and improving battery reliability.

In an effort to enhance the reliability of the DG batteries, DEC has proposed replacing its nickel cadmium batteries with lead acid batteries. The licensee stated that the replacement batteries will be Exide Nuclear Class 1E stationary batteries model NCN-9 with a nominal ampere-hour capacity of 688 based on an 8-hour rate to 1.75 volts per cell end voltage at 77 EF.

The licensee stated that the replacement batteries will be housed in an air conditioned, seismically qualified enclosure within the DG rooms and that the new battery racks will meet the appropriate seismic requirements. While lead acid batteries are in operation, hydrogen and oxygen gases are produced. These gases result from the electrolysis of the water portion of

the electrolyte by the charging current. Therefore, the licensee stated that the heating, ventilation, and air conditioning (HVAC) system will be designed to ensure adequate ventilation to prevent the hydrogen concentration from resulting in an explosive mixture. The HVAC system design will also ensure even distribution of air throughout the battery enclosure to sufficiently cool the battery cells. Based on the above, the NRC staff finds that housing the replacement batteries in an air conditioned, seismically qualified enclosure is acceptable.

One significant electrical aspect associated with the conversion from nickel cadmium to lead acid DG batteries is ensuring that the batteries are sized properly for supplying the required loads during a design basis event. The design duty cycle requirement for the replacement batteries has been calculated by the licensee and will be incorporated into the Catawba TS Bases, similar to that for the existing batteries. The duty cycle represents the total load used for battery sizing, including the temperature correction factor and design margin. The licensee stated that the replacement batteries will have an overall end voltage that is greater than the minimum required overall battery terminal voltage during discharge and will allow for at least one cell to be jumpered out and the battery to remain capable of powering its respective loads during a design basis accident for the required 2-hour duration. Based on the above, the NRC staff finds that the replacement batteries will be adequately sized and, therefore, capable of supplying the required loads during a design basis event.

In its March 8, 2005, response to an NRC staff request for additional information (RAI), the licensee stated that the sizing of its DG battery chargers is based on the ability of the battery chargers to replace the ampere-hours of capacity actually removed during a design basis event within an 8-hour interval. This wording is not consistent with the licensee's design basis. Per the licensee's design basis, the DG battery charger supply is required to be based on the largest combined demands of the various steady state loads and the charging capacity to restore the battery from the minimum charge state to the fully charged state, irrespective of the status of the unit during these demand occurrences. The minimum required amperes and duration ensures that these requirements can be satisfied. Thus, the Catawba DG battery chargers are required to be capable of restoring their respective DG battery from the minimum charge state to the fully charged state within 8 hours while supplying the largest combined demands of the various steady state loads.

In letter dated March 22, 2005, the licensee provided additional information clarifying its March 8, 2005, RAI response. In its clarification letter, the licensee stated that it considered the actual loads that would be on the DG battery charger following an accident and determined that the post-accident plant loads on the DG battery charger would not change as a result of the proposed modification to replace the DG nickel cadmium batteries with lead acid batteries. Therefore, the actual ampere-hours removed from the new lead acid batteries during a design basis event would be equal to the number of ampere-hours removed from the present nickel cadmium batteries. Since the ampere-hours removed remains the same, Catawba's DG battery chargers will continue to be adequately sized (i.e., the DG battery chargers will remain capable of restoring their respective DG battery (nickel cadmium or lead acid) from the minimum charge state to the fully charged state within 8 hours while supplying the largest combined demands of the various steady state loads).

DEC also stated that each Catawba unit will have both trains of its DG batteries replaced during a refueling outage. The replacement of all four DG batteries will span a time period of greater than one year, thus, the Catawba TSs will need to reflect the use of both battery types (lead

acid and nickel cadmium) until all of the DG batteries are replaced. Therefore, the TSs for the existing nickel cadmium batteries will be preserved until the licensee submits a follow-up license amendment request requesting to delete those requirements. The TS requirements of Table 3.8.6-1 will be applicable to the replacement lead acid DG batteries.

For this amendment, the proposed TS changes are as follows:

- a. SR 3.8.4.2 requires a verification of battery cell voltage. The licensee proposed modifying this SR to be applicable to only the DG nickel cadmium batteries. This change is acceptable because SR 3.8.6.1 and SR 3.8.6.3 sets the requirements for float voltage verification for the lead acid batteries.
- b. The second option of SR 3.8.4.3 requires verification of battery connection resistance. DEC proposed making it applicable to both dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- c. SR 3.8.4.6 requires verification of dc channel battery connection resistance. DEC proposed a revision to make it applicable to both dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- d. Limiting condition for operation 3.8.6 established the limits for battery operation. The licensee proposed splitting this TS into three parts; one for the dc channel batteries, one for the DG lead acid batteries, and one for the DG nickel cadmium batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- e. Condition A of TS 3.8.6 requires the battery cell parameters to be within the Category A and B limits of TS Table 3.8.6-1. The licensee proposed making it applicable to both the dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- f. Condition B of TS 3.8.6 requires the electrolyte temperature of the battery cells to be within limits. DEC proposed making this requirement applicable to both dc channel batteries and DG batteries (both DG battery types). Condition B also requires the batteries to be within Category C limits of TS Table 3.8.6-1. DEC proposed making this requirement applicable to both dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- g. The licensee also proposed adding Required Action B.2 to require cascading to the applicable TS for inoperable DG batteries (both DG battery types). This change is acceptable because it adds clarification to the interpretation of the TSs.
- h. Condition C of TS 3.8.6 sets the conditions for the electrolyte level of the batteries. It is revised to make the first portion of the Condition applicable to only DG nickel cadmium batteries. This change is acceptable because the equivalent

requirement is being captured in Condition B and Required Action B.2 for the lead acid batteries. The second portion of the Condition is deleted in favor of moving this portion to the second portion of Condition B. This change is acceptable because the requirements are captured in the modification of Condition B.

- i. SR 3.8.6.1 requires that the licensee verify the battery parameters to the Category A limits. The licensee proposed making it applicable to both dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.
- j. SR 3.8.6.2 requires the verification of the battery electrolyte levels. It is revised so that it is only applicable to DG nickel cadmium batteries. This change is acceptable because SR 3.8.6.1 requires an identical surveillance for the lead acid batteries.
- k. SR 3.8.6.3 requires verification that the battery cell parameters meet the Category B limits. DEC proposed making it applicable to both dc channel batteries and DG lead acid batteries. Because this change adds requirements for the lead acid batteries, it is conservative.

The NRC staff finds that the licensee's proposed TS changes accurately reflect changes needed for the replacement of the DG nickel cadmium batteries with lead acid batteries. The licensee's implementation of the proposed changes will provide assurance that the DG batteries will continue to remain capable of supplying their intended design safety function. Therefore, the NRC staff finds the proposed changes acceptable.

Based on its review of the proposed amendment, the NRC staff concludes that the proposed changes do not affect Catawba's compliance with the applicable regulations; therefore, the proposed changes are acceptable. As discussed in the licensee's July 19, 2004, cover letter, implementation of the aforementioned battery modifications will impact the Catawba Final Safety Analysis Report (FSAR). Therefore, the licensee is required to submit an FSAR revision in accordance with 10 CFR 50.71(e).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such

finding (69 FR 76488). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Matthew McConnell

Date: April 27, 2005