

April 14, 2005

Mr. William Levis  
Senior Vice President & Chief Nuclear Officer  
PSEG Nuclear LLC - X04  
Post Office Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NO. 1 - EVALUATION OF REQUEST TO UTILIZE 1998 EDITION WITH 2000 ADDENDA OF THE AMERICAN SOCIETY OF MECHANICAL ENGINEERS BOILER AND PRESSURE VESSEL CODE FOR THE THIRD 10-YEAR INTERVAL (TAC NO. MC4990)

Dear Mr. Levis:

By letter dated October 29, 2004, as supplemented by letter dated March 1, 2005, PSEG Nuclear, LLC (PSEG or the licensee) requested Nuclear Regulatory Commission (NRC) approval to use the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI 1998 Edition up to, and including, the 2000 Addenda for the Salem Nuclear Generating Station, Unit No. 1 (Salem 1). PSEG's request was made pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(g)(4)(iv). PSEG requested to use the later edition of the ASME Code for the remainder of the third 10-year inservice inspection (ISI) interval, which is scheduled to be completed by May 20, 2011, for all ASME Code, Section XI (Class 1, 2, and 3), Subsections IWB, IWC, IWD, and IWF, systems, structures, and components contained in IWX-2500-1.

PSEG stated that use of the later edition of the ASME Code will ensure a greater degree of implementation consistency between all stations (referring to Hope Creek Generating Station and Salem, Unit Nos. 1 and 2), decrease costs associated with surveillance procedure revisions, reduce opportunities for inadvertent human errors due to differences between ASME Code requirements and lessen administrative burdens and complexities associated with maintaining multiple ISI program long-term plans.

In its October 29, 2004, submittal, PSEG stated that they will comply with the following imposed additional requirements, as discussed in *Federal Register* Notice 69 FR 58804, dated October 1, 2004:

Exam Category B-D, items B3.40, B3.60, B3.120, and B3.140 for the use of ASME [Code] 1998 Edition for the inspection of Nozzle Inner Radii.

Exam Category B-G-2, item B7.80 for the use of ASME [Code] 1995 Edition to perform VT-1 of CRD [control rod drive] housing bolts, studs and nuts when disassembled.

Exam Category B-K, item B10.10 for the use of ASME [Code] 1995 Edition to examine welded attachments on at least one vessel in each group.

Additionally, in response to NRC staff questions, PSEG confirmed in its March 1, 2005, letter that it will comply with the additional requirements that the NRC has imposed for the use of the 1998 Edition, with 2000 Addenda of the ASME Code as discussed in *Federal Register* Notice 69 FR 58804:

1. PSEG will not use the provisions of paragraph IWA-4340 of the 2000 addenda with regard to mitigating defects by modification.
2. PSEG will comply with the provisions in IWA-4540(c), 1998 Edition [of Section XI,] for pressure testing Class 1, 2, and 3 mechanical joints.

PSEG further stated that ASME Code, Section XI, Subsections IWE and IWL will continue to be performed in accordance with PSEG's previously submitted and NRC-approved relief requests for IWE and IWL for Salem 1. The NRC's safety evaluation of that relief request is available in the Agencywide Documents Access and Management System under accession number ML003720636.

The limitations identified above are consistent with the limitations imposed by the implementation of the ASME Code up to the 2003 Addenda, as incorporated by reference in 10 CFR 50.55a. Given that PSEG will implement the 1998 Edition through the 2000 Addenda of ASME Code, Section XI consistent with its incorporation by reference in 10 CFR 50.55a, the NRC staff finds the licensee's request to be acceptable. Therefore, pursuant to 10 CFR 50.55a(g)(4)(iv), the NRC approves the licensee's request to use the 1998 Edition through the 2000 Addenda of the ASME Code, Section XI, with the limitations discussed above, for the remainder of the third 10-year ISI interval for Salem 1.

If you have any questions, please contact G. Edward Miller at 301-415-2481.

Sincerely,

***/RA by R. B. Ennis for/***

Darrell J. Roberts, Chief, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-272

cc: See next page

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