Company of Racine, Wisconsin, has applied for a partially exclusive license to practice the NASA inventions disclosed in (1) Patent No. U.S. 6,399,020 B1, entitled Aluminum-Silicon Alloy Having Improved Properties at Elevated Temperatures and Articles Cast Therefrom; (2) Patent No. U.S. 6,419,769 B1, entitled Aluminum-Silicon Alloy Having Improved Properties at Elevated Temperatures and Process for Producing Cast Articles Therefrom; and (3) PCT International Application No. PCT/US03/10372 entitled, High Strength Aluminum Alloy for High Temperature Application filed April 3, 2003, for European Patent Organizations (EPO), Australia, Brazil, Canada, Japan, Columbia, India, Indonesia, Mexico, Philippines, and Vietnam. All three inventions are assigned to the United States of America as represented by the Administrator of the National Aeronautics and Space Administration. Written objections to the prospective grant of a license should be sent to Mr. Jerry L. Seemann, Chief Patent Counsel/LS01, Marshall Space Flight Center, Huntsville, AL 35812. NASA has not yet made a determination to grant the requested license and may deny the requested license even if no objections are submitted within the comment period.

DATE(S): Responses to this notice must be received by May 23, 2005.

FOR FURTHER INFORMATION CONTACT: Sammy A. Nabors, Technology Transfer Department/ED02, Marshall Space Flight Center, Huntsville, AL 35812, (256) 544-5226.

Dated: March 15, 2005.

Keith T. Sefton,

Deputy General Counsel, Administration and Management.

[FR Doc. 05-5711 Filed 3-22-05; 8:45 am]

BILLING CODE 7510-13-P

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice 05-057]

Notice of Prospective Patent License

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of prospective patent license.

SUMMARY: NASA hereby gives notice that Phoenix Systems International, Inc. of Pine Brooke, NJ, has applied for an exclusive license to practice the invention described and claimed in NASA Case No. KSC-12664-3-CIP entitled "Emission Control System," which is assigned to the United States of America as represented by the Administrator of the National Aeronautics and Space Administration. Written objections to the prospective grant of an exclusive license to Phoenix Systems International, Inc. should be sent to Assistant Chief Counsel/Patent Counsel, NASA, Mail Code: CC-A, Office of the Chief Counsel, John F. Kennedy Space Center, Kennedy Space Center, FL 32899.

DATES: Responses to this Notice must be received within 15 days from date of publication in the Federal Register.

FOR FURTHER INFORMATION CONTACT: Randall M. Heald, Patent Counsel/ Assistant Chief Counsel, NASA, Office of the Chief Counsel, John F. Kennedy Space Center, Mail Code: CC-A, Kennedy Space Center, FL 32899, telephone (321) 867-7214.

Dated: March 14, 2005.

Keith T. Sefton,

Deputy General Counsel, Administration and Management.

[FR Doc. 05-5710 Filed 3-22-05; 8:45 am]

BILLING CODE 7510-13-P

NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Proposed Collection; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of pending NRC action to submit an information collection request to OMB and solicitation of public comment.

SUMMARY: The NRC is preparing a submittal to OMB for review of continued approval of information collections under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35).

Information pertaining to the requirement to be submitted:

1. The title of the information collection: 10 CFR part 75—Safeguards on Nuclear Material, Implementation of US/IAEA Agreement

2. Current OMB approval number: 3150-0055

3. How often the collection is required: Installation information is submitted upon written notification from the Commission. Changes are submitted as they occur. Nuclear material accounting and control information is submitted in accordance with specified instructions.

4. Who is required or asked to report: All persons licensed or certified by the Commission or Agreement States to possess source or special nuclear material at an installation specified on the U.S. eligible facilities list as determined by the Secretary of State or his designee and filed with the Commission, as well as holders of construction permits and persons who intend to receive source material.

5. The number of annual respondents: Seven, one of which perform the reporting and recordkeeping and the other six perform the recordkeeping only. The NRC-licensed facilities selected for inspection will be reporting or updating design information. This one facility and the six facilities selected pursuant to a separate protocol will maintain transfer and material balance records, but reporting to the IAEA will be through the U.S. State system (Nuclear Materials Management and Safeguards System).

6. The number of hours needed annually to complete the requirement or request: 2,800 (.2 hours for reporting and 2,800 hours for recordkeeping [400 hours per recordkeeper]).

7. Abstract: 10 CFR part 75 establishes requirements to implement the agreement between the United States and the International Atomic Energy Agency (IAEA). Under that agreement, NRC is required to collect information and make it available to the IAEA. Currently, the IAEA has selected and is inspecting two NRC-licensed facilities pursuant to 10 CFR 75.41.

Submit, by May 23, 2005 comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room located at One White Flint North, 11555 Rockville Pike, Rockville, MD. OMB clearance requests are available at the NRC Worldwide Web site (*http://www.nrc.gov/public-involve/doc-comment/omb/index.html*). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T-6 E 6,

Washington, DC 20555–0001, by telephone at (301) 415–7233, or by Internet electronic mail at *infocollects@nrc.gov.*

Dated at Rockville, Maryland, this 14th day of March 2005.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 05–5680 Filed 3–22–05; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72–20]

Department of Energy; Three Mile Island 2 Independent Spent Fuel Storage Installation; Notice of Docketing of Materials License SNM–2508 Amendment Application

AGENCY: Nuclear Regulatory Commission.

ACTION: License Amendment.

FOR FURTHER INFORMATION CONTACT: Joseph M. Sebrosky, Senior Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone: (301) 415–1132; fax number: (301) 415–1179; e-mail: *jms3@nrc.gov.*

SUPPLEMENTARY INFORMATION: By letter dated January 31, 2005, the Department of Energy (DOE or licensee) submitted an application to the U.S. Nuclear Regulatory Commission (NRC or the Commission), in accordance with Title 10 of the Code of Federal Regulations (10 CFR) 72.56, requesting the amendment of the Three Mile Island 2 (TMI–2) Independent Spent Fuel Storage Installation (ISFSI) license for the ISFSI located in Butte County, Idaho. DOE proposes to change the technical specification corrective actions if the 5 year leak test of the dry shielded canisters fails.

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72–20 and will remain the same for this action. Upon approval of the Commission, the TMI–2 ISFSI license, SNM–2508, would be amended to allow this action.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) regarding the proposed amendment or, if a determination is made that the proposed amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the proposed amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

For further details with respect to this amendment, see the application dated January 31, 2005, which is publically available in the records component of NRC's Agencywide Documents Access and Management System (ADAMS). The NRC maintains ADAMS, which provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the Internet at *http://www.nrc.gov/reading-rm/adams.html.* If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, (301) 415–4737 or by email to *pdr@nrc.gov.*

Dated at Rockville, Maryland, this 15th day of March 2005.

For the Nuclear Regulatory Commission.

John D. Monninger,

Chief, Licensing Section, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 05–5681 Filed 3–22–05; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–271]

Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc. Vermont Yankee Nuclear Power Station; Exemption

1.0 Background

Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc. (Entergy or the licensee) are the holders of Facility Operating License No. DPR–28 which authorizes operation of the Vermont Yankee Nuclear Power Station (VYNPS). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of a boiling-water reactor located in Vernon, Vermont.

2.0 Request/Action

Title 10 of the Code of Federal Regulations (10 CFR), section 50.54(o), requires primary reactor containments for water-cooled power reactors to be subject to the requirements of Appendix J to 10 CFR part 50. Appendix J specifies the leakage test requirements, schedules, and acceptance criteria for tests of the leak-tight integrity of the primary reactor containment and systems and components which penetrate the containment. Option B of Appendix J is titled "Performance-Based Requirements." Option B, section III.A., "Type A Test," requires that the overall integrated leakage rate must not exceed the allowable leakage rate (La) with margin, as specified in the Technical Specifications (TSs). The overall integrated leakage rate, as specified in the 10 CFR part 50, Appendix J, Option B, definitions, means the total leakage rate through all tested leakage paths. The licensee is requesting a permanent exemption from Option B, section III.A., requirements to permit exclusion of the main steam pathway leakage contributions from the overall integrated leakage rate Type A test measurement. Main steam leakage includes leakage through all four main steam lines and the main steam drain line.

Option B, Section III.B of 10 CFR part 50, Appendix J, "Type B and C Tests," requires that the sum of the leakage rates of all Type B and Type C local leak rate tests be less than the performance criterion (La) with margin, as specified in the TSs. The licensee also requests exemption from this requirement, to permit exclusion of the main steam pathway leakage contributions from the sum of the leakage rates from Type B and Type C tests.

The main steam leakage effluent has a different pathway to the environment, when compared to a typical containment penetration. It is not directed into the secondary containment and filtered through the standby gas treatment system as is other containment leakage. Instead, the main steam leakage is collected and treated via an alternative leakage treatment (ALT) path having different mitigation characteristics.

In performing accident analyses, it is appropriate to group various leakage effluents according to the treatment they receive before being released to the environment (*e.g.*, from main steam pathways). The proposed exemption would more appropriately permit ALT pathway leakage to be independently grouped with its unique leakage limits. In this manner, the VYNPS containment leakage testing program will be made more consistent with the limiting assumptions used in the associated accident consequence analyses.

The licensee has analyzed the main steam leakage pathway (with an increase in leakage from 62 standard cubic feet per hour (scfh) to 124 scfh at