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RS-05-028

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U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

> Braidwood Station, Units 1 and 2 Facility Operating License Nos. NPF-72 and NPF-77 NRC Docket Nos. STN 50-456 and STN 50-457

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Byron Station, Units 1 and 2 Facility Operating License Nos. NPF-37 and NPF-66 NRC Docket Nos. STN 50-454 and STN 50-455

- Subject: Braidwood Station and Byron Station Commitment Change Notification: NRC Bulletin 2003-01, "Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized-Water Reactors"
- References: (1) Letter from George F. Dick, Jr. (U. S. NRC) to Christopher M. Crane (Exelon Generation Company, LLC), "Request for Additional Information Regarding Bulletin 2003-01 Byron Station, Units 1 and 2, and Braidwood Station, Units 1 and 2," dated May 4, 2004
 - (2) Letter from K. A. Ainger (Exelon Generation Company, LLC) to U. S. NRC, "Braidwood Station and Byron Station Response to Request for Additional Information Regarding Bulletin 2003-01," dated July 15, 2004

The purpose of this letter is to notify the NRC of a change in due dates for two actions proposed in Reference 1 associated with NRC Bulletin 2003-01, "Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized-Water Reactors." The actions required Braidwood Station and Byron Station to:

1. review the impact of injecting more than one refueling water storage tank (RWST) volume inside containment, and

 determine if level instrumentation in the Braidwood Station and Byron Station Unit 1 and Unit 2 containments can provide reliable, post-accident indication of sump level.

These actions were originally due January 31, 2005; however, due to Exelon Generation Company, LLC (EGC) efforts in responding to Generic Letter 2004-02, "Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors," these evaluations were not finalized by this date.

EGC previously notified the NRC Project Manager for Braidwood and Byron Stations of this change in due date. Preliminary evaluations indicate that the Braidwood Station and Byron Station design will allow more than one RWST volume inside containment and therefore the final action to modify procedures to allow injecting more than one RWST volume (i.e., identified in the original response as candidate operator action 6) will be completed by June 30, 2005, as originally indicated in Reference 2.

Should you have any questions concerning this letter, please contact David J. Chrzanowski at (630) 657-2816.

Respectfully,

Joseph A. Bauer

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