

Exelon Generation Company, LLC
LaSalle County Station
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March 9, 2005

10 CFR 50.46

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Subject: Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report

Reference: Letter from S. R. Landahl (Exelon Generation Company, LLC) to U. S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated March 9, 2004.

In accordance with 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors," Exelon Generation Company (EGC), LLC, submits the enclosed attachments to fulfill the 30-day and annual reporting requirements for LaSalle County Station, Units 1 and 2. In the referenced letter, EGC reported the fuel peak cladding temperatures (PCTs) calculated based on an acceptable model to be 1380°F for General Electric (GE) fuel. The referenced letter also provided the PCT of 1807°F for the FANP fuel based on an acceptable model. Additionally, the referenced letter estimated the effects of changes that occurred since the PCT values had been calculated using an acceptable model. These changes resulted in an increase of 25°F to 1832°F for FANP fuel. Since the last evaluation, GE14 fuel has also been introduced into Unit 2. As a result, a new analysis was performed to bound both LaSalle units. Based on the new analysis, the PCT for GE fuel increased to a value of 1400°F. This is a change of 20°F from the last evaluation using an acceptable model. There were no changes for the FANP fuel and the PCT remains at 1832°F.

Unit 1 and Unit 2 employ a mixed core design containing co-resident GE and FANP fuel. The Loss of Coolant Accident (LOCA) analyses of record for both GE and FANP fuel are within all of the acceptance criteria set forth in 10 CFR 50.46.

Attachments 1 and 2 provide PCT information for the limiting LOCA evaluations for LaSalle County Station, Units 1 and 2, including all assessments as of February 1, 2005. The assessment notes are contained in Attachment 3 and provide a detailed description for each change or error reported.

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Should you have any questions concerning this letter, please contact Mr. Terrence W. Simpkin, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,

A handwritten signature in black ink that reads "Daniel J. Enright". The signature is written in a cursive style with a large, prominent initial "D".

Daniel J. Enright
Plant Manager
LaSalle County Station

Attachments

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector – LaSalle County Station

**Attachment 1
LaSalle Units 1 and 2 10 CFR 50.46 Report (GE Fuel)**

PLANT NAME: LaSalle Units 1 and 2
 ECCS EVALUATION MODEL: SAFER/GESTR LOCA
 REPORT REVISION DATE: February 1, 2005
 CURRENT OPERATING CYCLES: L1C11 and L2C11 *

ANALYSIS OF RECORD

Evaluation Model Methodology: NEDE-23785-1-PA, Rev. 1, "GESTR-LOCA and SAFER Models for the Evaluation of the Loss-of-Coolant Accident (Volume III), SAFER/GESTR Application Methodology", October 1984.

Calculation: "Project Task Report, Exelon LaSalle Unit 1 and 2 SAFER/GESTR Loss-of-Coolant Accident Analysis for GE 14 Fuel," GE report number GE-NE-0000-0022-8684-R1, dated December 2004.

Fuel: GE14

Limiting Single Failure: HPCS Diesel Generator

Limiting Break Size and Location: Double Ended Guillotine of Recirculation Pump Suction Piping

Reference PCT: 1400°F

MARGIN ALLOCATION

A. PRIOR LOCA MODEL ASSESSMENTS

A GE LOCA analysis was performed in December 2004 to support the introduction of GE 14 for L2C11. This analysis bounds both LaSalle Units and addressed all errors and issues.	Δ PCT = 0 °F
Net PCT	1400 °F

B. CURRENT LOCA MODEL ASSESSMENTS

None	N/A
Net PCT	1400 °F

* Currently Unit 2 in refueling. Expected startup – March 12, 2005

Attachment 2
LaSalle Units 1 and 2 10 CFR 50.46 Report (FANP Fuel)

PLANT NAME: LaSalle Units 1 and 2
 ECCS EVALUATION MODEL: EXEM BWR Evaluation Model
 REPORT REVISION DATE: February 1, 2005
 CURRENT OPERATING CYCLE: LIC11 and L2C11 *

ANALYSIS OF RECORD

Evaluation Model Methodology: Advanced Nuclear Fuels Corporation Methodology for Boiling Water Reactors EXEM BWR Evaluation Model, ANF-91-048(P)(A), January 1993.
 BWR Jet Pump Model Revision for RELAX, ANF-91-048(P)(A), Supplement 1 and Supplement 2, Siemens Power Corporation, October 1997.

Calculation:

1. LaSalle LOCA-ECCS Analysis MAPLHGR Limits for ATRIUM™-9B Fuel, EMF-2175(P), March 1999.
2. LOCA Break Spectrum Analysis for LaSalle Units 1 and 2, EMF-2174(P), March 1999.
3. LaSalle Units 1 and 2 LOCA-ECCS Analysis MAPLHGR Limit for ATRIUM™-10 Fuel, EMF-2641(P), November 2001.
4. LaSalle Units 1 and 2 LOCA Break Spectrum Analysis for ATRIUM™-10 Fuel, EMF-2639(P), November 2001.

Fuel: ATRIUM™-9B and ATRIUM™-10
 Limiting Fuel: ATRIUM™-9B
 Limiting Single Failure: HPCS Diesel Generator
 Limiting Break Size and Location: 1.1 ft² Recirculation Pump Discharge Side Line Break
 1807 °F

Reference PCT:

MARGIN ALLOCATION

A. PRIOR LOCA MODEL ASSESSMENTS

10 CFR 50.46 report dated May 7, 1999 (See Note 2)	ΔPCT = 0 °F
10 CFR 50.46 report dated February 9, 2000 (See Note 3)	ΔPCT = 18 °F
10 CFR 50.46 report dated June 12, 2000 (See Note 4)	ΔPCT = 0 °F
10 CFR 50.46 report dated June 8, 2001 (See Note 5)	ΔPCT = 0 °F
10 CFR 50.46 report dated June 8, 2002 (See Note 6)	ΔPCT = 2 °F
10 CFR 50.46 report dated June 9, 2003 (See Note 7)	ΔPCT = 5 °F
10 CFR 50.46 report dated March 9, 2004 (See Note 1)	ΔPCT = 0 °F
Net PCT	1832 °F

B. CURRENT LOCA MODEL ASSESSMENTS

No errors/issues for this reporting period	ΔPCT = 0 °F
Net PCT	1832 °F

* Currently Unit 2 in refueling. Expected startup – March 12, 2005

Attachment 3
LaSalle Units 1 and 2 10 CFR 50.46 Report (FANP Fuel)

1. Prior LOCA model assessment for FANP fuel

During the startup of LaSalle Unit 1 Cycle 11 several evaluations were performed for FANP LOCA analysis as reported in the Reference. The net results of these evaluations were that there was a zero degree PCT impact. Additionally a problem was also identified by FANP pertaining with the transfer of RELAX coolant temperature data from PREHUXY to HUXY at the time of core spray. FANP determined that the impact this problem on the limiting break spectrums results was zero degree. This was also reported in the Reference.

[Reference: Letter from Susan R Landahl (Exelon) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated March 9, 2004.]

2. Prior LOCA Model Assessment for FANP fuel

The May 1999 LOCA model assessment was a new analysis of record for Framatome (Formerly Siemens) due to the introduction of ATRIUM-9B fuel into the Unit 2 Cycle 8 core. Therefore, there is no PCT change. Analysis was performed for a core power of 3722 MWt that bounds the current uprated power of 3489 MWt.

[Reference: Letter from J. A. Benjamin (ComEd) to U.S. NRC, "Report of Significant Change in Calculated Peak Cladding Temperature (PCT) – 10CFR 50.46 Report," dated May 7, 1999.]

3. Prior LOCA Model Assessment for FANP fuel

The February 2000 50.46 report assessed the impact of errors in the LOCA evaluation model.

[Reference: Letter from J. A. Benjamin (ComEd) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10CFR 50.46 Report," dated February 9, 2000.]

4. Prior LOCA Model Assessment for FANP fuel

The June 2000 10 CFR 50.46 report does not have any PCT assessment for ATRIUM-9B fuel.

[Reference: Letter from C. G. Pardee (ComEd) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated June 12, 2000.]

5. Prior LOCA model assessment for FANP fuel

The reference letter assessed impact of Unit 2 LPCS riser leakage, errors in FANP LOCA analysis model and Unit 2 Cycle 9 reload fuel.

[Reference: Letter from M. A Schiavoni (Exelon) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated June 8, 2001.]

6. Prior LOCA model assessment for FANP fuel

The referenced letter assessed impact of errors in FANP LOCA analysis model, Unit 1 Cycle 10 reload fuel and ATRIUM-9B exposure extension.

[Reference: Letter from M. A Schiavoni (Exelon) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated June 8, 2002.]

Attachment 3
LaSalle Units 1 and 2 10 CFR 50.46 Report (FANP Fuel)

7. Prior LOCA model assessment for FANP fuel

The June 2003 50.46 report assessed the impact of errors in the LOCA evaluation, Unit 2 jet pump leakage, Unit 2 Cycle 10 reload Fuel and the Unit 1 mid-cycle reload.

[Reference: Letter from Susan R Landahl (Exelon) to U.S. NRC, "Plant Specific ECCS Evaluation Changes – 10 CFR 50.46 Report," dated June 9, 2003.]