

DISTRIBUTION CONTROL LIST

Document Name: ITS/BASES/TRM

CC_NAME	NAME	DEPT	LOCATION
1	OPS PROCEDURE GROUP SUPV.	OPS PROCEDURE GROUP	IP2
3	PLANT MANAGER'S OFFICE	UNIT 3 (UNIT 3/IPEC ONLY)	IP2
5	CONTROL ROOM & MASTER	OPS (3PT-D001/6 (U3/IPEC)	IP3 (ONLY)
11	RES DEPARTMENT MANAGER	RES (UNIT 3/IPEC ONLY)	45-4-A
16	SICARD MIKE	I&C OFFICE (SUPERVISOR)	45-2-A
19	STEWART ANN	LICENSING	GSB-2D
20	CHEMISTRY SUPERVISOR	CHEMISTRY DEPARTMENT	45-4-A
21	TSC (IP3)	EEC BUILDING	IP2
22	SHIFT MGR. (LUB-001-GEN)	OPS (UNIT 3/IPEC ONLY)	IP3
23	LIS	LICENSING & INFO SERV	OFFSITE
25	SIMULATOR	TRAIN (UNIT 3/IPEC ONLY)	48-2-A
28	RESIDENT INSPECTOR	US NRC 88' ELEVATION	IP2
32	EOF	E-PLAN (ALL EP'S)	EOF
47	CHAPMAN N	BECHTEL	OFFSITE
50	TADEMY L. SHARON	WESTINGHOUSE ELECTRIC	OFFSITE
55	GSB TECHNICAL LIBRARY	A MCCALLION/IPEC, IP2/IP3	GSB-3B
61	SIMULATOR	TRAIN (UNIT 3/IPEC ONLY)	48-2-A
69	CONROY PAT	LICENSING/ROOM 205	GSB-2D
99	BARANSKI J (ALL)	ST. EMERG. MGMT. OFFICE	OFFSITE
102	BILYOU CHRISTINE I&C ONLY	I&C SUPPORT	45-2-D
106	SIMULATOR INSTRUCT AREA	TRG/3PT-D001-D006 ONLY)	#48
164	CONTROL ROOM & MASTER	OPS (3PT-D001/6 (U3/IPEC)	IP3 (ONLY)
207	TROY M	PROCUREMENT ENG.	1A
273	FAISON CHARLENE	NUCLEAR LICENSING	WPO-12
319	L.GRANT (LRQ-OPS TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
354	L.GRANT (LRQ-OPS/TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
357	L.GRANT (ITS/INFO ONLY)	TRAINING - ILO CLASSES	48-2-A
424	GRANT LEAH (9 COPIES)	(UNIT 3/IPEC ONLY)	#48
474	OUELLETTE P	ENG., PLAN & MGMT INC	OFFSITE
483	SCHMITT RICHIE	MAINTENANCE ENG/SUPV	45-1-A
484	HANSLER ROBERT	REACTOR ENGINEERING	72' UNIT 2
489	CLOUGHNESSY PAT	PLANT SUPPORT TEAM	GSB-3B
491	ORLANDO TOM (MANAGER)	PROGRAMS/COMPONENTS ENG	45-3-G
492	FSS UNIT 3	OPERATIONS	K-IP-I210
493	OPERATIONS FIN TEAM	33 TURBIN DECK	45-1-A
494	AEOF/A.GROSJEAN (ALL EP'S)	E-PLAN (EOP'S ONLY)	WPO-12D
495	JOINT NEWS CENTER	EMER PLN (ALL EP'S)	EOF
496	L.GRANT (LRQ-OPS/TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
497	L.GRANT (LRQ-OPS/TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
500	L.GRANT (LRQ-OPS TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
501	L.GRANT (LRQ-OPS TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
512	L.GRANT (LRQ-OPS TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
513	L.GRANT (LRQ-OPS TRAIN)	LRQ (UNIT 3/IPEC ONLY)	#48
518	DOCUMENT CONTROL DESK	NRC (ALL EP'S)	OFFSITE
527	MILIANO PATRICK	NRC/SR. PROJECT MANAGER	OFFSITE
529	FIELDS DEBBIE	OUTAGE PLANNING	IP3/OSB

A001

	IPEC SITE MANAGEMENT MANUAL	QUALITY RELATED ADMINISTRATIVE PROCEDURE	IP-SMM-AD-103 Revision 0
		INFORMATIONAL USE	Page 13 of 21

ATTACHMENT 10.1

SMM CONTROLLED DOCUMENT TRANSMITTAL FORM

SITE MANAGEMENT MANUAL CONTROLLED DOCUMENT TRANSMITTAL FORM - PROCEDURES

Page 1 of 1

		CONTROLLED DOCUMENT TRANSMITTAL FORM - PROCEDURES	
TO: DISTRIBUTION		DATE: 03/07/2005	TRANSMITTAL NO:
FROM: IPEC DOCUMENT CONTROL: EEC or IP2 53'EL		<small>(Circle one)</small>	PHONE NUMBER: 271-7440
<p>The Document(s) identified below are forwarded for use. In accordance with IP-SMM-AD-103, please review to verify receipt, incorporate the document(s) into your controlled document file, properly disposition superseded, void, or inactive document(s). Sign and return the receipt acknowledgement below within fifteen (15) working days.</p>			
AFFECTED DOCUMENT: IP3 TECH SPEC BASES			
DOC #	REV #	TITLE	INSTRUCTIONS
INDEX - REMOVE YOUR CURRENT COPY AND REPLACE WITH ATTACHED UPDATED INDEX			
THE FOLLOWING PROCEDURE HAS BEEN REVISED, PLEASE REMOVE YOUR CURRENT COPY AND REPLACE WITH ATTACHED UPDATED REVISION:			
SECTION 3.9.6 REV 1 - EFFECTIVE IMMEDIATELY			
<p align="center">*****PLEASE NOTE EFFECTIVE DATE*****</p>			
RECEIPT OF THE ABOVE LISTED DOCUMENT(S) IS HEREBY ACKNOWLEDGED. I CERTIFY THAT ALL SUPERSEDED, VOID, OR INACTIVE COPIES OF THE ABOVE LISTED DOCUMENT(S) IN MY POSSESSION HAVE BEEN REMOVED FROM USE AND ALL UPDATES HAVE BEEN PERFORMED IN ACCORDANCE WITH EFFECTIVE DATE(S) (IF APPLICABLE) AS SHOWN ON THE DOCUMENT(S).			
<hr/> NAME (PRINT)	<hr/> SIGNATURE	<hr/> DATE	<hr/> CC#

INDIAN POINT 3 TECHNICAL SPECIFICATION BASES

INSTRUCTIONS FOR UPDATE: 14-03/07/05

REMOVE

- a) List of Effective Sections;
4 pages (Rev. 13)
- b) Section 3.9.6; Rev. 0
4 pages

INSERT

- a) List of Effective Sections;
4 pages (Rev. 14)
- b) Section 3.9.6; Rev. 1
3 pages

**TECHNICAL SPECIFICATION BASES
LIST OF EFFECTIVE SECTIONS**

BASES SECTION	REV	NUMBER OF PAGES	EFFECTIVE DATE
Tbl of Cnt	1	4	05/18/2001
B 2.0 SAFETY LIMITS			
B 2.1.1	0	5	03/19/2001
B 2.1.2	0	4	03/19/2001
B 3.0 LCO AND SR APPLICABILITY			
B 3.0	1	15	09/30/2002
B 3.1 REACTIVITY CONTROL			
B 3.1.1	0	6	03/19/2001
B 3.1.2	0	7	03/19/2001
B 3.1.3	1	7	10/27/2004
B 3.1.4	0	13	03/19/2001
B 3.1.5	0	5	03/19/2001
B 3.1.6	0	6	03/19/2001
B 3.1.7	0	8	03/19/2001
B 3.1.8	0	7	03/19/2001
B 3.2 POWER DISTRIBUTION LIMITS			
B 3.2.1	0	7	03/19/2001
B 3.2.2	0	7	03/19/2001
B 3.2.3	0	9	03/19/2001
B 3.2.4	0	7	03/19/2001
B 3.3 INSTRUMENTATION			
B 3.3.1	1	59	09/30/2002
B 3.3.2	3	45	12/04/2002
B 3.3.3	2	19	09/30/2002
B 3.3.4	0	7	03/19/2001
B 3.3.5	1	6	10/27/2004
B 3.3.6	0	10	03/19/2001
B 3.3.7	0	6	03/19/2001
B 3.3.8	1	4	03/17/2003
B 3.4 REACTOR COOLANT SYSTEM			
B 3.4.1	0	6	03/19/2001
B 3.4.2	0	3	03/19/2001
B 3.4.3	1	9	10/27/2004
B 3.4.4	0	4	03/19/2001
B 3.4.5	0	6	03/19/2001
B 3.4.6	0	6	03/19/2001
B 3.4.7	0	7	03/19/2001
B 3.4.8	0	4	03/19/2001
B 3.4.9	2	5	06/20/2003
B 3.4.10	0	5	03/19/2001
B 3.4.11	0	8	03/19/2001
B 3.4.12	1	20	10/27/2004
B 3.4.13	2	6	11/19/2001
B 3.4.14	0	10	03/19/2001
B 3.4.15	2	7	11/19/2001
B 3.4.16	0	7	03/19/2001
B 3.5 ECCS			
B 3.5.1	1	10	10/27/2004
B 3.5.2	0	13	03/19/2001
B 3.5.3	0	4	03/19/2001
B 3.5.4	0	9	03/19/2001

BASES SECTION	REV	NUMBER OF PAGES	EFFECTIVE DATE
B 3.6 CONTAINMENT			
B 3.6.1	0	5	03/19/2001
B 3.6.2	0	9	03/19/2001
B 3.6.3	0	17	03/19/2001
B 3.6.4	0	3	03/19/2001
B 3.6.5	1	5	06/20/2003
B 3.6.6	1	13	12/04/2002
B 3.6.7	0	6	03/19/2001
B 3.6.8	0	6	03/19/2001
B 3.6.9	0	8	03/19/2001
B 3.6.10	0	12	03/19/2001
B 3.7 PLANT SYSTEMS			
B 3.7.1	1	6	12/04/2002
B 3.7.2	0	10	03/19/2001
B 3.7.3	1	7	05/18/2001
B 3.7.4	0	5	03/19/2001
B 3.7.5	1	9	02/25/2005
B 3.7.6	1	4	12/04/2002
B 3.7.7	1	4	12/17/2004
B 3.7.8	0	7	03/19/2001
B 3.7.9	1	9	09/30/2002
B 3.7.10	0	3	03/19/2001
B 3.7.11	3	9	01/24/2005
B 3.7.12	0	4	03/19/2001
B 3.7.13	2	7	06/20/2003
B 3.7.14	0	3	03/19/2001
B 3.7.15	0	5	03/19/2001
B 3.7.16	0	6	03/19/2001
B 3.7.17	0	4	03/19/2001
B 3.8 ELECTRICAL POWER			
B 3.8.1	1	32	01/22/2002
B 3.8.2	0	7	03/19/2001
B 3.8.3	0	13	03/19/2001
B 3.8.4	1	11	01/22/2002
B 3.8.5	0	4	03/19/2001
B 3.8.6	0	8	03/19/2001
B 3.8.7	1	8	06/20/2003
B 3.8.8	1	4	06/20/2003
B 3.8.9	2	14	06/20/2003
B 3.8.10	0	4	03/19/2001
B 3.9 REFUELING OPERATIONS			
B 3.9.1	0	4	03/19/2001
B 3.9.2	0	4	03/19/2001
B 3.9.3	1	8	03/17/2003
B 3.9.4	0	4	03/19/2001
B 3.9.5	0	4	03/19/2001
B 3.9.6	1	3	03/07/2005

TECHNICAL SPECIFICATION BASES
REVISION HISTORY

REVISION HISTORY FOR BASES

AFFECTED SECTIONS	REV	EFFECTIVE DATE	DESCRIPTION
ALL	0	03/19/01	Initial issue of Bases derived from NUREG-1431, in conjunction with Technical Specification Amendment 205 for conversion of 'Current Technical Specifications' to 'Improved Technical Specifications'.
BASES UPDATE PACKAGE 01-031901			
B 3.4.13 B 3.4.15	1	03/19/01	Changes regarding containment sump flow monitor per NSE 01-3-018 LWD Rev 0. Change issued concurrent with Rev 0.
BASES UPDATE PACKAGE 02-051801			
Table of Contents	1	05/18/01	Title of Section B 3.7.3 revised per Tech Spec Amend 207
B 3.7.3	1	05/18/01	Implementation of Tech Spec Amend 207
BASES UPDATE PACKAGE 03-111901			
B 3.3.2	1	11/19/01	Correction to statement regarding applicability of Function 5, to be consistent with the Technical Specification.
B 3.3.3	1	11/19/01	Changes to reflect reclassification of certain SG narrow range level instruments as QA Category M per NSE 97-3-439, Rev 1.
B 3.4.13 B 3.4.15	2	11/19/01	Changes to reflect installation of a new control room alarm for 'VC Sump Pump Running'. Changes per NSE 01-3-018, Rev 1 and DCP 01-3-023 LWD.
B 3.7.11	1	11/19/01	Clarification of allowable flowrate for CRVS in 'incident mode with outside air makeup.'
BASES UPDATE PACKAGE 04-012202			
B 3.3.2	2	01/22/02	Clarify starting logic of 32 ABFP per EVL-01-3-078 MULTI, Rev 0.
B 3.8.1	1	01/22/02	Provide additional guidance for SR 3.8.1.1 and Condition Statements A.1 and B.1 per EVL-01-3-078 MULTI, Rev 0.
B 3.8.4	1	01/22/02	Revision of battery design description per plant modification and to reflect Tech Spec Amendment 209.
B 3.8.9	1	01/22/02	Provide additional information regarding MCC in Table B 3.8.9-1 per EVL-01-3-078 MULTI, Rev 0.
BASES UPDATE PACKAGE 05-093002			
B 3.0	1	09/30/02	Changes to reflect Tech Spec Amendment 212 regarding delay period for a missed surveillance. Changes adopt TSTF 358, Rev 6.
B 3.3.1	1	09/30/02	Changes regarding description of turbine runback feature per EVAL-99-3-063 NIS.
B 3.3.3	2	09/30/02	Changes to reflect Tech Spec Amendment 211 regarding CETs and other PAM instruments.
B 3.7.9	1	09/30/02	Changes regarding SWN -35-1 and -2 valves per EVAL-00-3-095 SWS, Rev 0.

**TECHNICAL SPECIFICATION BASES
REVISION HISTORY**

AFFECTED SECTIONS	REV	EFFECTIVE DATE	DESCRIPTION
BASES UPDATE PACKAGE 06-120402			
B 3.3.2	3	12/04/02	Changes to reflect Tech Spec Amendment 213 regarding 1.4% power uprate.
B 3.6.6	1		
B 3.7.1	1		
B 3.7.6	1		
BASES UPDATE PACKAGE 07-031703			
B 3.3.8	1	03/17/2003	Changes to reflect Tech Spec Amendment 215 regarding implementation of Alternate Source Term analysis methodology to the Fuel Handling Accident.
B 3.7.13	1		
B 3.9.3	1		
BASES UPDATE PACKAGE 08-032803			
B 3.4.9	1	03/28/2003	Changes to reflect Tech Spec Amendment 216 regarding relaxation of pressurizer level limits in MODE 3.
BASES UPDATE PACKAGE 09-062003			
B 3.4.9	2	06/20/2003	Changes to reflect commitment for a dedicated operator per Tech Spec Amendment 216.
B 3.6.5	1	06/20/2003	Implements Corrective Action 11 from CR-IP3-2002-02095; 4 FCUs should be in operation to assure representative measurement of containment air temperature.
B 3.7.11	2	06/20/2003	Correction to Background description regarding system response to Firestat detector actuation per ACT 02-62887.
B 3.7.13	2	06/20/2003	Revision to Background description of FSB air tempering units to reflect design change per DCP 95-3-142.
B 3.8.7	1	06/20/2003	Changes to reflect replacement of Inverter 34 per DCP-01-022.
B 3.8.8	1	06/20/2003	
B 3.8.9	2	06/20/2003	
BASES UPDATE PACKAGE 10-102704			
B 3.1.3	1	10/27/2004	Clarification of the surveillance requirements for TS 3.1.3 per 50.59 screen.
B 3.3.5	1	10/27/2004	Clarify the requirements for performing a Trip Actuating Device Operational Test (TADOT) on the 480V degraded grid and undervoltage relays per 50.59 screen.
B 3.4.3	1	10/27/2004	Extension of the RCS pressure/temperature limits and corresponding OPS limits from 16.17 to 20 EFPY (TS Amendment 220).
B 3.4.12	1		
B 3.5.1	1	10/27/2004	Changes to reflect Tech Spec Amendment 222 regarding extension of completion time for Accumulators.
BASES UPDATE PACKAGE 11-121004			
B 3.7.7	1	12/17/2004	Addition of valves CT-1300 and CT-1302 to Surveillance SR 3.7.7.2 to verify that all city water header supply isolation valves are open. Reflects Tech Spec Amendment 218.
BASES UPDATE PACKAGE 12-012405			
B 3.7.11	3	01/24/2005	Temporary allowance for use of KI/SCBA for unfiltered inleakage above limit.

**TECHNICAL SPECIFICATION BASES
REVISION HISTORY**

AFFECTED SECTIONS	REV	EFFECTIVE DATE	DESCRIPTION
BASES UPDATE PACKAGE 13-022505			
B 3.7.5	1	02/25/2005	Clarification on Surveillance Requirement 3.7.5.3 as it relates to plant condition/frequency of performance of Auxiliary Feedwater Pump full flow testing.
BASES UPDATE PACKAGE 14-030705			
B 3.9.6	1	03/07/2005	Changes to reflect that the decay time prior to fuel movement is a minimum of 84 hours per Tech Spec Amendment 215.

B 3.9 REFUELING OPERATIONS

B 3.9.6 Refueling Cavity Water Level

BASES

BACKGROUND

The movement of irradiated fuel assemblies or performance of CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts, within containment requires a minimum water level of 23 ft above the top of the reactor vessel flange. During refueling, this maintains sufficient water level in the containment, refueling canal, fuel transfer canal, refueling cavity, and spent fuel pit. Sufficient water is necessary to retain iodine fission product activity in the water in the event of a fuel handling accident (Refs. 1 and 2). Sufficient iodine activity would be retained to limit offsite doses from the accident to within RG 1.183 limits (Ref. 4).

APPLICABLE SAFETY ANALYSES

During CORE ALTERATIONS and movement of irradiated fuel assemblies, the water level in the refueling canal and the refueling cavity is an initial condition design parameter in the analysis of a fuel handling accident in containment, as postulated by Regulatory Guide 1.25 (Ref. 1). In the Fuel Handling Accident (FHA) analysis (Ref. 5), a fuel assembly is assumed to be dropped and damaged during refueling. It is assumed that all of the fuel rods in one assembly are damaged to the extent that all of the gap activity is released. The fuel handling accident is described in Reference 2.

Doses from the FHA are calculated in accordance with the Alternate Source Term methodology of Regulatory Guide 1.183 (Ref. 4). For water level of 23 ft or greater above the fuel, RG 1.183 specifies an overall decontamination factor of 200. There is no retention of noble gases in the water. The decay time prior to fuel movement is a minimum of 84 hours. Credit is not taken for removal of iodine by filters, nor is credit taken for isolation of release paths.

Using RG 1.183 methodology, all calculated offsite and control room doses are determined to be within the RG 1.183 specified fractions of the 10CFR50.67 limits for decay periods of ≥ 84 hours.

(continued)

BASES

APPLICABLE SAFETY ANALYSES (continued)

Further reductions in the amount of radioactivity potentially released following a fuel handling accident inside containment are expected because the containment will be isolated either automatically or through operator action following a fuel handling accident. Specifically, LCO 3.3.6, "Containment Purge System and Pressure Relief Line Isolation Instrumentation," requires the Operability of radiogas monitors R-11 and R-12, either of which could generate an automatic isolation signal, during the movement of irradiated fuel.

Refueling cavity water level satisfies Criterion 2 of 10 CFR 50.36.

LCO

A minimum refueling cavity water level of 23 ft above the reactor vessel flange is required to ensure that the radiological consequences of a postulated fuel handling accident inside containment are within acceptable limits, as per Reference 6.

APPLICABILITY

LCO 3.9.6 is applicable during CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts, and when moving irradiated fuel assemblies within containment. The LCO minimizes the possibility of a fuel handling accident in containment that is beyond the assumptions of the safety analysis. If irradiated fuel assemblies are not present in containment, there can be no significant radioactivity release as a result of a postulated fuel handling accident. Requirements for fuel handling accidents in the spent fuel pool are covered by LCO 3.7.14, "Spent Fuel Pit Water Level."

ACTIONS

A.1 and A.2

With a water level of < 23 ft above the top of the reactor vessel flange, all operations involving CORE ALTERATIONS or movement of irradiated fuel assemblies within the containment shall be suspended immediately to ensure that a fuel handling accident cannot occur.

The suspension of CORE ALTERATIONS and fuel movement shall not preclude completion of movement of a component to a safe position.

(continued)

BASES

SURVEILLANCE REQUIREMENTS

SR 3.9.6.1

Verification of a minimum water level of 23 ft above the top of the reactor vessel flange ensures that the design basis for the analysis of the postulated fuel handling accident during refueling operations is met. Water at the required level above the top of the reactor vessel flange limits the consequences of damaged fuel rods that are postulated to result from a fuel handling accident inside containment (Ref. 2).

The Frequency of 24 hours is based on engineering judgment and is considered adequate in view of the large volume of water and the normal procedural controls of valve positions, which make significant unplanned level changes unlikely.

REFERENCES

1. Regulatory Guide 1.25, March 23, 1972.
2. FSAR, Section 14.2.
3. NUREG-0800, Section 15.7.4.
4. Regulatory Guide 1.183, July 2002.
5. Letter INT 02-29 Westinghouse Electric Co., Radiological Consequences of a Fuel Handling Accident, May 2002.
6. Safety Evaluation Report for Amendment No. 215 to Facility Operating License No. DPR-64, March 17, 2003.