

February 8, 2005

Bill Eaton, BWRVIP Chairman
Entergy Operations, Inc.
Echelon One
1340 Echelon Parkway
Jackson, MS 39213-8202

SUBJECT: ADDENDUM TO SAFETY EVALUATIONS OF THE BOILING WATER REACTOR VESSEL AND INTERNALS PROJECT REPORTS REGARDING REPAIR DESIGN CRITERIA (BWRVIP-16, BWRVIP-19, BWRVIP-50, BWRVIP-51, BWRVIP-52, BWRVIP-55, BWRVIP-56, AND BWRVIP-57 REPORTS)

Dear Mr. Eaton:

Representatives of the BWRVIP brought to the staff's attention that a statement in the BWRVIP's response to the staff's request for additional information regarding material requirements was misinterpreted in the staff's safety evaluations (SEs) dated July 12, 2004, for the BWRVIP-16 and BWRVIP-19 reports; SEs dated August 20, 2004, for the BWRVIP-50, BWRVIP-51, BWRVIP-52, BWRVIP-55, and BWRVIP-56 reports; and SE dated September 3, 2004, for the BWRVIP-57 report. A few statements regarding the material requirements, in each of the SEs listed above, were not accurately described. Therefore, the staff has attached an addendum to the subject repair design criteria SEs, to revise the statements regarding the material requirements. We apologize for any inconveniences.

Sincerely,

/RA/

Matthew A. Mitchell, Chief
Vessels & Internals Integrity and Welding Section
Materials and Chemical Engineering Branch
Division of Engineering
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: BWRVIP Service List

Bill Eaton, BWRVIP Chairman
Entergy Operations, Inc.
Echelon One
1340 Echelon Parkway
Jackson, MS 39213-8202

SUBJECT: ADDENDUM TO SAFETY EVALUATIONS OF THE BOILING WATER REACTOR VESSEL AND INTERNALS PROJECT REPORTS REGARDING REPAIR DESIGN CRITERIA (BWRVIP-16, -19, -50, -51, -52, -55, -56, AND -57 REPORTS)

Dear Mr. Eaton:

Representatives of the BWRVIP brought to the staff's attention that a statement in the BWRVIP's response to the staff's request for additional information regarding material requirements was misinterpreted in the staff's safety evaluations (SEs) dated July 12, 2004, for the BWRVIP-16 and BWRVIP-19 reports; SEs dated August 20, 2004, for the BWRVIP-50, BWRVIP-51, BWRVIP-52, BWRVIP-55, and BWRVIP-56 reports; and SE dated September 3, 2004, for the BWRVIP-57 report. A few statements regarding the material requirements, in each of the SEs listed above, were not accurately described. Therefore, the staff has attached an addendum to the subject repair design criteria SEs, to revise the statements regarding the material requirements. We apologize for any inconveniences.

Sincerely,
/RA/
Matthew A. Mitchell, Chief
Vessels & Internals Integrity and Welding Section
Materials and Chemical Engineering Branch
Division of Engineering
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: BWRVIP Service List

DISTRIBUTION:

EMCB R/F AHiser WCullen
JRajan CEMoyer

ADAMS Accession No.: ML050560212

INDICATE IN BOX: "C"=COPY W/O ATTACHMENT/ENCLOSURE, "E"=COPY W/ATT/ENCL, "N"=NO COPY EMCB:DE

OFFICE	EMCB:DE	E	EMCB:DE	E	EMCB:DE	
NAME	MKhanna		BElliot		MMitchell	
DATE	02/2/2005		02/2/2005		02 /8/2005	

OFFICIAL RECORD COPY

ADDENDUM TO THE SAFETY EVALUATIONS (SE)
OF THE BOILING WATER REACTOR VESSEL AND INTERNALS PROJECT REPORTS
REGARDING REPAIR DESIGN CRITERIA
(BWRVIP-16, -19, -50, -51, -52, -55, -56, AND -57 REPORTS)

The staff is proposing to replace the statements in Section 3 of the safety evaluations for each of the repair design criteria reports (BWRVIP-16, -19, -50, -51, -52, -55, -56, and -57). Because the statements in some of the reports are stated differently, each SE for the applicable report is addressed, below.

Replace the following paragraph, as currently stated in the Staff's Evaluation section of the safety evaluations for the BWRVIP-16 and -19 reports:

“Section 3.2 of the BWRVIP-84 report states, “materials must meet the requirements of ASME Section II specifications, ASME Code Case, ASTM specification, or other material specifications that have been previously accepted by the regulatory authority. Otherwise, a material that is necessary for a design must be submitted on a case-by-case basis to the governing regulatory authority for approval, either on a plant-specific basis or through a mechanism such as a BWRVIP repair design criteria topical report.” The staff interprets this statement to mean that materials will meet ASTM specifications that have been previously accepted for use by the staff and/or ASME Code Cases that have been previously accepted for use by the staff (Item 4-1). Therefore, Item 4-1 is resolved. This statement does indicate that materials not meeting ASME Section II specifications will be submitted to the governing regulatory authority for approval. Therefore, Item 4-2 is resolved. There is no discussion of EPRI NP-7032 and EPRI #84-MG-18 in BWRVIP-84. Therefore, the staff will request the BWRVIP to discuss Item 4-3, above, in its review of BWRVIP-84. The staff finds the BWRVIP's response acceptable because the material requirements will be removed from BWRVIP-16 and BWRVIP-19, and the remaining issues will be resolved in the staff's review of BWRVIP-84.”

with:

The BWRVIP-A repair design criteria reports will be revised to reference the BWRVIP-84 report for all material requirements. Hence, the staff will request the BWRVIP to discuss Items 4-1, 4-2, and 4-3, including relevant aspects of reports NP-7032 and 84-MG-18, in its review of BWRVIP-84. Therefore, Items 4-1, 4-2, and 4-3 are resolved with respect to the repair design criteria and will be addressed in the BWRVIP-84 report.

ATTACHMENT

Remove the statements, listed below, from the Staff's Evaluation section of the SEs for the BWRVIP-50, -51, -52, -55, -56, and -57 reports:

"The staff interprets this statement to mean that the materials will meet ASME Code Section II, ASTM specifications that have been previously accepted for use by the staff, and/or ASME Code Cases that have been previously accepted for use by the staff. This statement does indicate that materials not meeting ASME Code Section II specifications will be submitted to the governing regulatory authority for approval."

The staff confirmed that the information requested has been included in BWRVIP-84.

The staff determined that these statements are not necessary because the BWRVIP has committed to revise the BWRVIP-84 report to clarify the material requirements for the reactor vessel internals components. The BWRVIP-A repair design criteria reports will be revised to reference the BWRVIP-84 report for all material requirements.

cc:

Tom Mulford, EPRI BWRVIP
Integration Manager
Raj Pathania, EPRI BWRVIP
Mitigation Manager
Ken Wolfe, EPRI BWRVIP
Repair Manager
Larry Steinert, EPRI BWRVIP
Electric Power Research Institute
P.O. Box 10412
3412 Hillview Ave.
Palo Alto, CA 94303

George Inch, Technical Chairman
BWRVIP Assessment Committee
Constellation Nuclear
Nine Mile Point Nuclear Station (M/S ESB-1)
348 Lake Road
Lycoming, NY 13093

William C. Holston, Executive Chairman
BWRVIP Integration Committee
Constellation Generation Group
Nine Mile Point Nuclear Station
P. O. Box 63
Lycoming, NY 13093

Jim Meister, BWRVIP Vice-Chairman
Exelon Corp.
Cornerstone II at Cantera
4300 Winfield Rd.
Warrenville, IL 60555-4012

Al Wrape, Executive Chairman
BWRVIP Assessment Committee
PPL Susquehanna, LLC
2 N. 9th St.
Allentown, PA 18101-1139

H. Lewis Sumner, Executive Chairman
BWRVIP Mitigation Committee
Vice President, Hatch Project
Southern Nuclear Operating Co.
M/S BIN B051, P.O. BOX 1295
40 Inverness Center Parkway
Birmingham, AL 35242-4809

Robin Dyle, Technical Chairman
BWRVIP Integration Committee
Southern Nuclear Operating Co.
42 Inverness Center Parkway (M/S B234)
Birmingham, AL 35242-4809

Denver Atwood, Technical Chairman
BWRVIP Repair Focus Group
Southern Nuclear Operating Co.
Post Office Box 1295
40 Inverness Center Parkway (M/S B031)
Birmingham, AL 35242-4809

Jeff Goldstein, Technical Chairman
BWRVIP Mitigation Committee
Entergy Nuclear NE
440 Hamilton Ave. (M/S K-WPO-11c)
White Plains, NY 10601

Dale Atkinson, BWRVIP Liason to EPRI
Nuclear Power Council
Energy Northwest
Columbia Generating Station (M/S PEO8)
P. O. Box 968
Snake River Complex
North Power Plant Loop
Richland, WA 99352-0968

Richard Ciemiewicz, Technical Vice Chairman
BWRVIP Assessment Committee
Exelon Corp.
Peach Bottom Atomic Power Station
M/S SMB3-6
1848 Lay Road
Delta, PA 17314-9032

Charles J. Wirtz, Chairman
BWRVIP Inspection Focus Group
FirstEnergy Corp.
Perry Nuclear Power Plant (M/S A250)
10 Center Road
Perry, OH 44081

Robert Carter, EPRI BWRVIP
Assessment Manager
Greg Selby, EPRI BWRVIP
Inspection Manager
EPRI NDE Center
P.O. Box 217097
1300 W. T. Harris Blvd.
Charlotte, NC 28221