

March 1, 2005

Mr. Steven G. Frantz, Director  
Reed Reactor Facility  
3203 SE Woodstock Blvd.  
Portland, OR 97202

SUBJECT: RETAKE EXAMINATION REPORT NO. 50-288/OL-05-01, REED COLLEGE

Dear Mr. Frantz:

During the week of February 7, 2005, you administered an operator licensing examination at your Reed College Reactor. The examination was conducted according to NUREG-1478, "Non-Power Reactor Operator Licensing Examiner Standards," Revision 1.

In accordance with 10 CFR 2.390 of the Commission's regulations, a copy of this letter and the enclosures will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at (the Public Electronic Reading Room) <http://www.nrc.gov/NRC/ADAMS/index.html>. The NRC is forwarding the individual grades to you in a separate letter which will not be released publicly. Should you have any questions concerning this examination, please contact Paul V. Doyle, Jr. at 301-415-1058 or via internet E-mail at [pvd@nrc.gov](mailto:pvd@nrc.gov).

Sincerely,

**/RA/**

Patrick M. Madden, Section Chief  
Research and Test Reactors Section  
New, Research and Test Reactors Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No. 50-288

Enclosures: 1. Initial Examination Report No. 50-288/OL-05-01  
2. Examination and answer key

cc w/encls: Please see next page

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**DISTRIBUTION:**

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Facility File (EBarnhill) O-6 F-2 DHughes

EXAMINATION PACKAGE ACCESSION NO.: ML050100331

EXAMINATION REPORT ACCESSION #: ML050550257

TEMPLATE #: NRR-074

OFFICE	RNRP:CE		IROB:LA	E	RNRP:SC	
NAME	PDoyle		EBarnhill		PMadden	
DATE	2/ 25 /2005		2/ 28 /2005		2/ 28 /2005	

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Reed College

Docket No. 50-288

cc:

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Portland, OR 97204

Reed College  
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Dean of Faculty  
3203 S.E. Woodstock Boulevard  
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Reed College  
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Oregon Department of Energy  
ATTN: David Stewart-Smith, Director  
Division of Radiation Control  
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Salem, OR 97310

Test, Research, and Training  
Reactor Newsletter  
University of Florida  
202 Nuclear Sciences Center  
Gainesville, FL 32611

U. S. NUCLEAR REGULATORY COMMISSION  
OPERATOR LICENSING INITIAL EXAMINATION REPORT

REPORT NO.: 50-288/OL-05-01

FACILITY DOCKET NO.: 50-288

FACILITY LICENSE NO.: R-112

FACILITY: Reed College

EXAMINATION DATES: February 11, 2005

SUBMITTED BY:                     /RA/                     2/28/05  
Paul V. Doyle Jr., Chief Examiner Date

**SUMMARY:**

The NRC mailed a Section A only operator licensing written examination to the facility for administration to a Reactor Operator candidate who had failed section a of the examination administered May, 2004. The facility administered the examination on February 11, 2005. The candidate passed the examination.

**REPORT DETAILS**

1. Examiners:  
Paul V. Doyle Jr., Chief Examiner

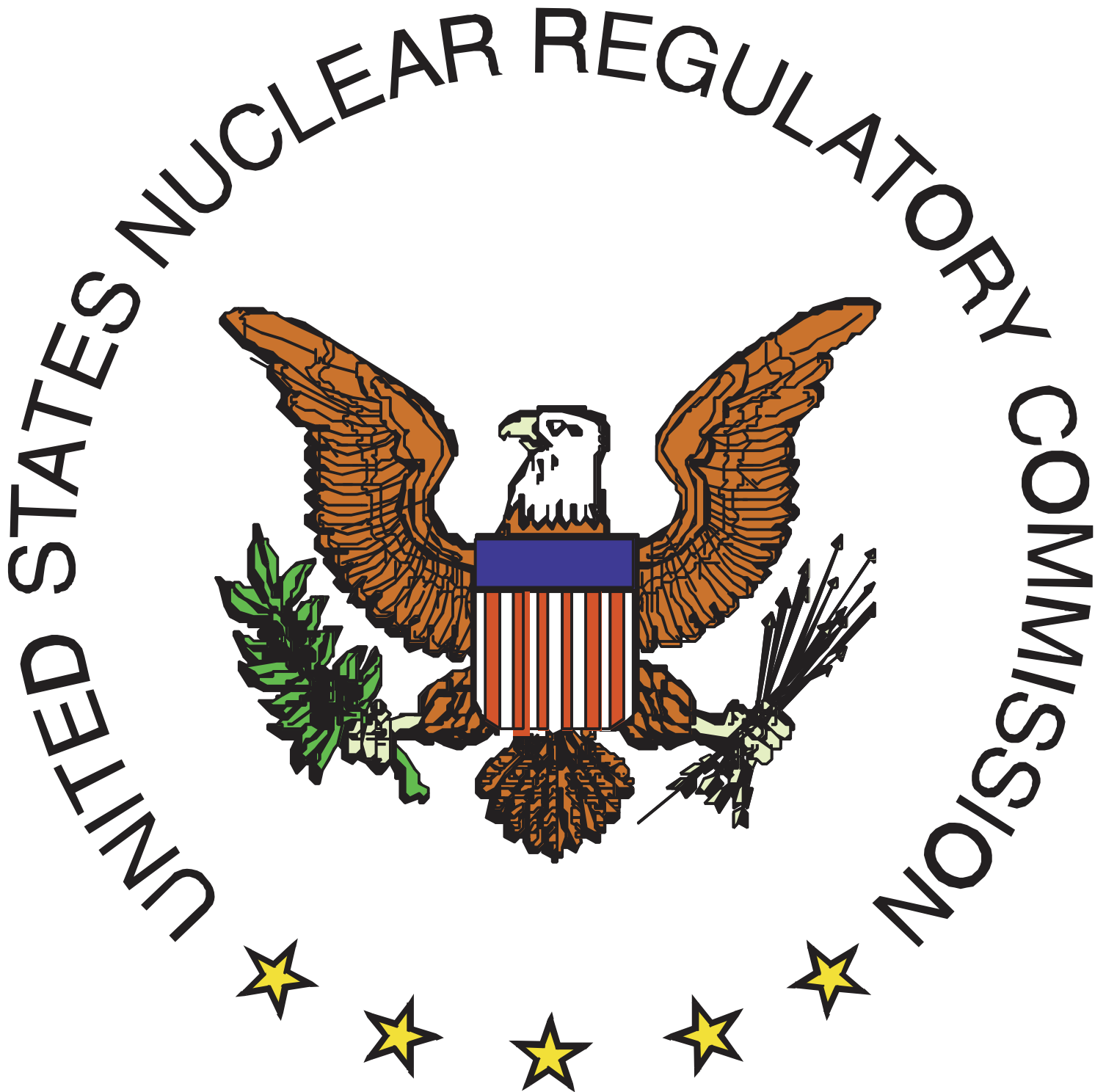
2. Results:

	RO PASS/FAIL	SRO PASS/FAIL	TOTAL PASS/FAIL
Written	1/0	0/0	1/0
Operating Tests	0/0	0/0	0/0
Overall	1/0	0/0	1/0

3. Exit Meeting:

There was no exit meeting. The facility mailed the written examination to the NRC where it was graded. The facility had no comments on the examination.

**U.S. Nuclear Regulatory Commission**  
**OPERATOR LICENSING EXAMINATION**  
**With Answer Key**



**REED REACTOR FACILITY**  
**Week of February 7, 2005**

**ENCLOSURE 2**

## QUESTION A.01 [2.0 points, ½ each]

The listed isotopes are all potential daughter products due to the radioactive decay of  ${}_{35}\text{Br}^{87}$ . Identify the type of decay necessary (Alpha, Beta, Gamma or Neutron emission) to produce each of the isotopes.

- a.  ${}_{33}\text{As}^{83}$
- b.  ${}_{35}\text{Br}^{86}$
- c.  ${}_{35}\text{Br}^{87}$
- d.  ${}_{36}\text{Kr}^{87}$

## QUESTION A.02 [1.0 point]

What is the definition of reactivity? A measure of the ...

- a. number of neutrons being produced in the core.
- b. number of neutrons being absorbed by the fuel.
- c. reactor's multiplication factor.
- d. reactor's departure from critical.

## QUESTION A.03 [2.0 points, ½ each]

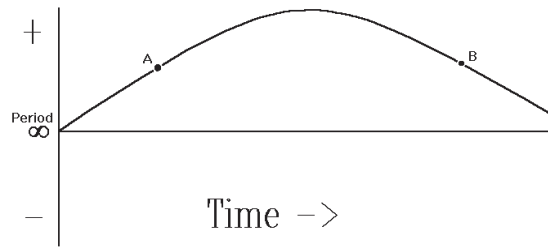
Using the drawing of the Integral Rod Worth Curve provided, identify each of the following reactivity worths.

- |  |          |
|--|----------|
| a. Total Rod Worth                               | 1. B - A |
| b. Actual Shutdown Margin                        | 2. C - A |
| c. Technical Specification Shutdown Margin Limit | 3. C - B |
| d. Excess Reactivity                             | 4. D - C |
|  | 5. E - C |
|  | 6. E - D |
|  | 7. E - A |

## QUESTION A.04 [1.0 point]

Shown below is a trace of reactor period as a function of time. Between points A and B reactor power is:

- a. continually increasing.
- b. continually decreasing.
- c. increasing, then decreasing.
- d. constant.



## QUESTION A.05 [1.0 point]

What is the kinetic energy range of a thermal neutron?

- a.  $> 1 \text{ MeV}$
- b.  $100 \text{ KeV} - 1 \text{ MeV}$
- c.  $1 \text{ eV} - 100 \text{ KeV}$
- d.  $< 1 \text{ eV}$

## QUESTION A.06 [1.0 point]

Which ONE of the following is the difference between prompt and delayed neutrons? Prompt neutrons ...

- a. account for less than 1% of the neutron population, while delayed neutrons account for the rest.
- b. are released during fast-fission events, while delayed neutrons are released during the decay process.
- c. are released during the fission process (fast & thermal), while delayed neutrons are released during the decay process.
- d. are the dominating factor in determining reactor period, while delayed neutrons have little effect on reactor period.

## QUESTION A.07 [1.0 point]

Suppose the temperature coefficient of a core is  $-2.5 \times 10^{-4} \Delta\text{K/K/EC}$  and the average control rod worth of the regulating control rod is  $5.895 \times 10^{-3} \Delta\text{K/K/inch}$ . If the temperature **INCREASES** by 50EC what will the automatic control command the regulating rod to do? Select the answer that is closest to the calculated value.

- a. 5.6 inches in
- b. 2.1 inches out
- c. 0.5 inches in
- d. 4.3 inches out

## QUESTION A.08 [1.0 point]

Given the following data, which ONE of the following is the closest to the half life of the material?

TIME	ACTIVITY
0	2400 cps
10 min.	1757 cps
20 min.	1286 cps
30 min.	941 cps
60 min.	369 cps

- a. 11 minutes
- b. 22 minutes
- c. 44 minutes
- d. 51 minutes

## QUESTION A.09 [1.0 point]

During a fuel loading of the core, as the reactor approaches criticality, the value of  $1/M$ :

- a. Increases toward one
- b. Decreases toward one
- c. Increases toward infinity
- d. Decreases toward zero

## QUESTION A.10 [1.0 point]

Which ONE of the following is the major source of energy released during fission?

- a. Prompt gamma ray absorption
- b. Slowing down of fission fragments
- c. Neutrino interactions
- d. fission neutron scattering reactions

## QUESTION A.11 [1.0 point]

Which one of the following is the definition of the **FAST FISSION FACTOR?**

- a. The ratio of the number of neutrons produced by fast fission to the number produced by thermal fission
- b. The ratio of the number of neutrons produced by thermal fission to the number produced by fast fission
- c. The ratio of the number of neutrons produced by fast and thermal fission to the number produced by thermal fission
- d. The ratio of the number of neutrons produced by fast fission to the number produced by fast and thermal fission

## QUESTION A.12 [1.0 point]

In a reactor at full power, the thermal neutron flux ( $\phi$ ) is  $2.5 \times 10^{12}$  neutrons/cm<sup>2</sup>/sec. and the macroscopic fission cross-section  $G_f$  is  $0.1 \text{ cm}^{-1}$ . The fission reaction rate is:

- a.  $2.5 \times 10^{11}$  fissions/sec.
- b.  $2.5 \times 10^{13}$  fissions/sec.
- c.  $2.5 \times 10^{11}$  fissions/cm<sup>3</sup>/sec.
- d.  $2.5 \times 10^{13}$  fissions/cm<sup>3</sup>/sec.

## QUESTION A.13 [1.0 point]

The number of neutrons passing through a one square centimeter of target material per second is the definition of which one of the following?

- a. Neutron Population (np)
- b. Neutron Impact Potential (nip)
- c. Neutron Flux (nv)
- d. Neutron Density (nd)

## QUESTION A.14 [1.0 point]

Which ONE of the following explains the response of a **SUBCRITICAL** reactor to equal insertions of positive reactivity as the reactor approaches criticality? Each insertion causes a ...

- a. **SMALLER** increase in the neutron flux resulting in a **LONGER** time to stabilize.
- b. **LARGER** increase in the neutron flux resulting in a **LONGER** time to stabilize.
- c. **SMALLER** increase in the neutron flux resulting in a **SHORTER** time to stabilize.
- d. **LARGER** increase in the neutron flux resulting in a **SHORTER** time to stabilize.

## QUESTION A.15 [1.0 point]

Which ONE of the following atoms will cause a neutron to lose the most energy in an elastic collision?

- a. Uranium<sup>238</sup>
- b. Carbon<sup>12</sup>
- c. Hydrogen<sup>2</sup>
- d. Hydrogen<sup>1</sup>

## QUESTION A.16 [1.0 point]

A thin foil target of 10% copper and 90% aluminum is in a thermal neutron beam. Given  $\sigma_{a,Cu} = 3.79$  barns,  $\sigma_{a,Al} = 0.23$  barns,  $\sigma_{s,Cu} = 7.90$  barns, and  $\sigma_{s,Al} = 1.49$  barns, which ONE of the following reactions has the highest probability of occurring? A neutron ...

- a. scattering reaction with aluminum
- b. scattering reaction with copper
- c. absorption in aluminum
- d. absorption in copper

## QUESTION A.17 [1.0 point]

When performing rod calibrations, many facilities pull the rod out a given increment, then measure the time for reactor power to double (doubling time), then calculate the reactor period. If the doubling time is 42 seconds, what is the reactor period?

- a. 29 sec
- b. 42 sec
- c. 61 sec
- d. 84 sec

## QUESTION A.18 [1.0 point]

Which ONE of the following statements concerning reactor poisons is NOT true?

- a. Following shutdown, Samarium concentration will increase to some value then stabilize.
- b. Following shutdown, Xenon concentration will initially increase to some value then decrease exponentially
- c. During reactor operation, Samarium concentration is independent of reactor power level.
- d. During reactor operation, Xenon concentration is dependent on reactor power level.

A.01 a, alpha; b, neutron; c, gamma; d, Beta  
 Ref: Training Manual, Reed Reactor Facility § 2.4

A.02 d  
 REF: Training Manual, Reed Reactor Facility § 9.2

A.03 a, 7; b, 5; c, 6; d, 2  
 REF: Training Manual, Reed Reactor Facility § 10.2

A.04 a  
 REF: Standard NRC question

A.05 d  
 REF: Training Manual, Reed Reactor Facility § 7.3

A.06 c  
 REF: Training Manual, Reed Reactor Facility § 10.4

A.07 B  
 REF: Training Manual, Reed Reactor Facility § 10.6

The temperature increase will result in reactivity change of:  $-2.5 \times 10^{-4} \Delta K/K/EC \times 50EC = -1.25 \times 10^{-2} \Delta K/K$ . Since the temperature rise is a negative reactivity insertion, the control rod must drive out to add positive reactivity.  $D = (1.25 \times 10^{-2} \Delta K/K) \div (5.895 \times 10^{-3} \Delta K/K/in.) = 2.12 \text{ in.}$

A.08 b  
 REF: Training Manual, Reed Reactor Facility § 2.6

A.09 d  
 REF: Training Manual, Reed Reactor Facility § 8.4

A.10 b  
 REF: Training Manual, Reed Reactor Facility § 7.1

A.11 c  
 REF: Training Manual, Reed Reactor Facility § 8.2

A.12 c  
 REF:  $R = \phi G_r = (2.5 \times 10^{12}) \times 0.1 = 2.5 \times 10^{11}$  NRC Question Bank

A.13 c  
 REF: Training Manual, Reed Reactor Facility § 6.3

A.14 b  
 REF: Training Manual, Reed Reactor Facility § 8.4

A.15 d  
 REF: Training Manual, Reed Reactor Facility § 3.3

A.16 a  
 REF: Training Manual, Reed Reactor Facility § 6.2

A.17 c  
 REF: Training Manual, Reed Reactor Facility § 9.4  
 $\ln(2) = -\text{time}/\tau$   $\tau = \text{time}/(\ln(2)) = 60.59 \text{ . } 61 \text{ seconds}$

A.18 c  
 REF: Training Manual, Reed Reactor Facility § 10.4

U. S. NUCLEAR REGULATORY COMMISSION  
RESEARCH AND TEST REACTOR OPERATOR LICENSING EXAMINATION

FACILITY: Reed College

REACTOR TYPE: TRIGA

DATE ADMINISTERED: 2005/01/\_\_\_\_

CANDIDATE: \_\_\_\_\_

INSTRUCTIONS TO CANDIDATE:

Answers are to be written on the answer sheets provided. Points for each question are indicated in brackets for each question. You must score 70% to pass. Examinations will be picked up one (1) hour after the examination starts.

Category	% of	% of	Category	Category
<u>Value</u>	<u>Total</u>	<u>Candidates</u>	<u>Value</u>	
		<u>Score</u>		
<u>20.00</u>	<u>100.0</u>	_____	_____	A. Reactor Theory, Thermodynamics and Facility Operating Characteristics
<u>20.00</u>		_____	_____%	TOTALS
		_____	FINAL GRADE	

All work done on this examination is my own. I have neither given nor received aid.

\_\_\_\_\_  
Candidate's Signature

## NRC RULES AND GUIDELINES FOR LICENSE EXAMINATIONS

During the administration of this examination the following rules apply:

1. Cheating on the examination means an automatic denial of your application and could result in more severe penalties.
2. After the examination has been completed, you must sign the statement on the cover sheet indicating that the work is your own and you have neither received nor given assistance in completing the examination. This must be done after you complete the examination.
3. Restroom trips are to be limited and only one candidate at a time may leave. You must avoid all contacts with anyone outside the examination room to avoid even the appearance or possibility of cheating.
4. Use black ink or dark pencil only to facilitate legible reproductions.
5. Print your name in the blank provided in the upper right-hand corner of the examination cover sheet and each answer sheet.
6. Mark your answers on the answer sheet provided. **USE ONLY THE PAPER PROVIDED AND DO NOT WRITE ON THE BACK SIDE OF THE PAGE.**
7. The point value for each question is indicated in [brackets] after the question.
8. If the intent of a question is unclear, ask questions of the proctor only.
9. When turning in your examination, assemble the completed examination with examination questions, examination aids and answer sheets. In addition turn in all scrap paper.
10. Ensure all information you wish to have evaluated as part of your answer is on your answer sheet. Scrap paper will be disposed of immediately following the examination.
11. To pass the examination you must achieve a grade of 70 percent or greater.
12. There is a time limit of one (1) hour for completion of the examination.
13. When you have completed and turned in your examination, leave the examination area. If you are observed in this area while the examination is still in progress, your license may be denied or revoked.



A.1a alpha beta gamma neutron \_\_\_\_

A.7 a b c d \_\_\_\_

A.1b alpha beta gamma neutron \_\_\_\_

A.8 a b c d \_\_\_\_

A.1c alpha beta gamma neutron \_\_\_\_

A.9 a b c d \_\_\_\_

A.1d alpha beta gamma neutron \_\_\_\_

A.10 a b c d \_\_\_\_

A.2 a b c d \_\_\_\_

A.11 a b c d \_\_\_\_

A.3a 1 2 3 4 5 6 7 \_\_\_\_

A.12 a b c d \_\_\_\_

A.3b 1 2 3 4 5 6 7 \_\_\_\_

A.13 a b c d \_\_\_\_

A.3c 1 2 3 4 5 6 7 \_\_\_\_

A.14 a b c d \_\_\_\_

A.3d 1 2 3 4 5 6 7 \_\_\_\_

A.15 a b c d \_\_\_\_

A.4 a b c d \_\_\_\_

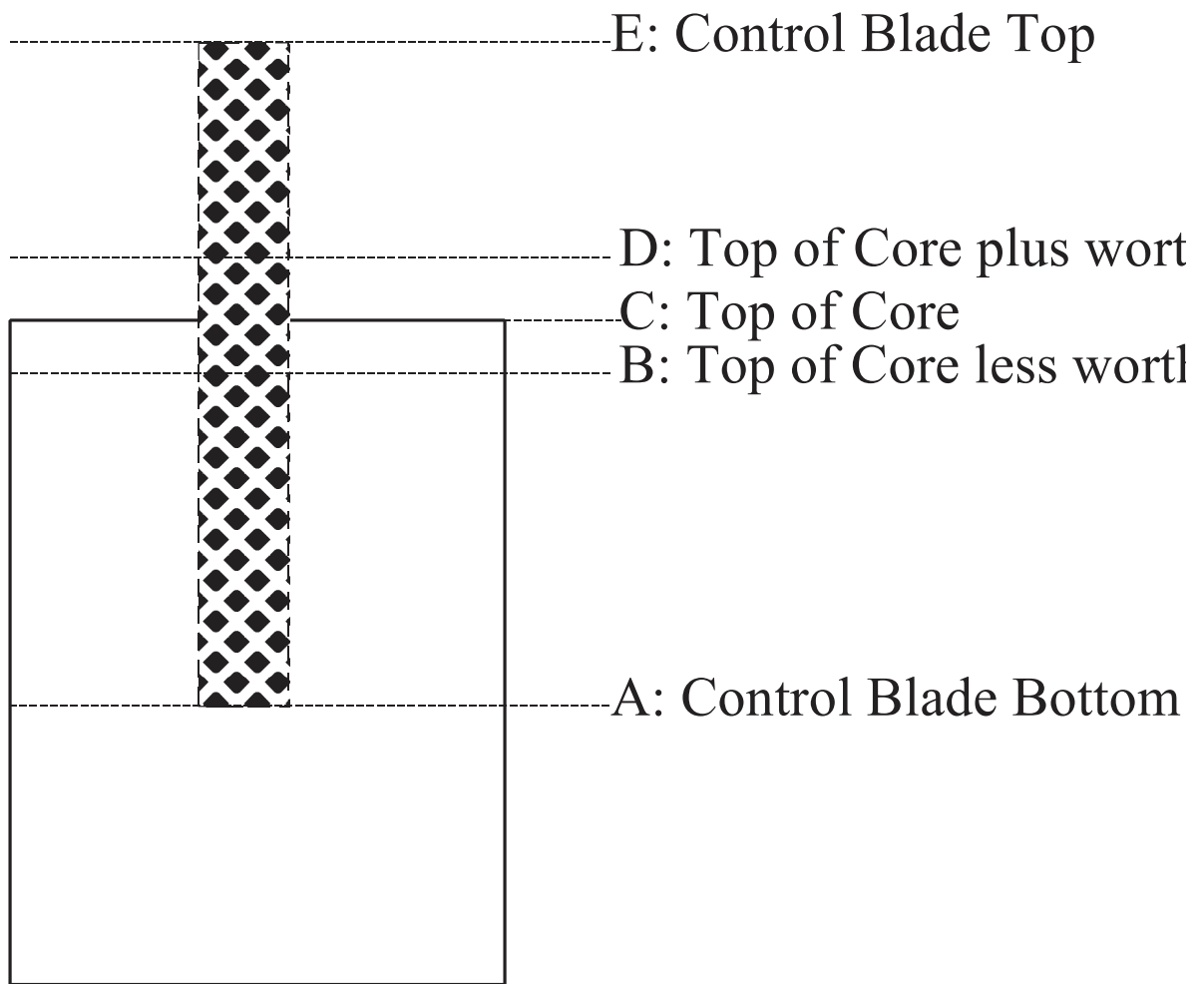
A.16 a b c d \_\_\_\_

A.5 a b c d \_\_\_\_

A.17 a b c d \_\_\_\_

A.6 a b c d \_\_\_\_

A.18 a b c d \_\_\_\_



Reactor at Cold, Clean Critical.