

March 3, 2005

Mr. Mark B. Bezilla  
Vice President-Nuclear, Davis-Besse  
FirstEnergy Nuclear Operating Company  
Davis-Besse Nuclear Power Station  
5501 North State Route 2  
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - GENERIC LETTER  
96-06, "ASSURANCE OF EQUIPMENT OPERABILITY AND CONTAINMENT  
INTEGRITY DURING DESIGN-BASIS ACCIDENT CONDITIONS"  
(TAC NO. M96803)

Dear Mr. Bezilla:

Generic Letter (GL) 96-06 was issued by the U.S. Nuclear Regulatory Commission (NRC) on September 30, 1996. GL 96-06 requested that licensees evaluate cooling water systems that serve containment air coolers to assure that they are not vulnerable to waterhammer and two-phase flow conditions, and that licensees evaluate thermally-induced overpressurization of isolated water-filled piping sections in containment. This letter pertains to the thermally-induced overpressurization issue.

You responded to the thermally-induced overpressurization issue in GL 96-06 by letters dated January 28, February 28, July 28 and September 30, 1997, and August 28, 1998. In your January 28, 1997, response, you identified 13 penetrations potentially vulnerable to a water solid volume that may be subjected to an increase in pressure due to heating of trapped fluid. In your operability review of these penetrations, you determined that four penetrations meet the design-basis criteria and the remaining penetrations, where the criteria are exceeded, are operable based on piping plastic deformation and potential leakage through packing/valve seating surfaces.

In response to the NRC staff's request for additional information on March 18, 1998, by letter dated August 28, 1998, you provided your evaluation of the criteria and long-term corrective action plan to resolve the issue. You indicated that four of the penetrations meet the design-basis American Society of Mechanical Engineers *Boiler and Pressure Vessel Code* faulted stress allowables, four other penetrations had check valves installed, and one penetration had a relief valve installed. Procedural changes have been made for two penetrations to assure that the affected piping remains partially drained to prevent potential overpressurization. For the remaining two penetrations, you determined that the seating surfaces of the air-operated isolation valves (globe and diaphragm valves) associated with these penetrations provide inherent relief to prevent overpressurization.

The NRC staff has reviewed your analytical evaluation criteria and finds it to meet the licensing commitments in the Updated Safety Analysis Report. The NRC staff concludes, based on the letters set out above, that your evaluation and proposed corrective actions provide an acceptable resolution for the issue of thermally-induced overpressurization of piping in containment.

However, due to recent piping modifications, the NRC staff requests that you either affirm that your responses are still valid in this area or supplement your responses. Therefore, the NRC staff requests that you respond within 45 days of the date of this letter with your schedule for response.

Sincerely,

**/RA/**

Jon B. Hopkins, Sr. Project Manager, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-346

cc: See next page

M. Bezilla

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March 3, 2005

The NRC staff has reviewed your analytical evaluation criteria and finds it to meet the licensing commitments in the Updated Safety Analysis Report. The NRC staff concludes, based on the letters set out above, that your evaluation and proposed corrective actions provide an acceptable resolution for the issue of thermally-induced overpressurization of piping in containment.

However, due to recent piping modifications, the NRC staff requests that you either affirm that your responses are still valid in this area or supplement your responses. Therefore, the NRC staff requests that you respond within 45 days of the date of this letter with your schedule for response.

Sincerely,

**/RA/**

Jon B. Hopkins, Senior Project Manager  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-346

cc: See next page

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**ADAMS ACCESSION NUMBER: ML050540613** \*memo from K. Manoly to S. Richards

OFFICE	LPM:PDIV-1	PM:PDIII-2	LA:PDIII-2	SC:EMEB *	SC:PDIII-2
NAME	TAlexion	JHopkins	DClarke	KManoly	GSuh
DATE	3/1/05	3/1/05	2/28/05	12/28/98	3/3/05

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