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W3F1-2005-0013

February 21, 2005

Alan J. Harris  
Director, Nuclear Safety Assurance  
Waterford 3

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: Supplement to Amendment Request NPF-38-258,  
Modify Technical Specification (TS) 5.3.1, Fuel Assemblies and TS  
6.9.1.11.1, Core Operating Limits  
Waterford Steam Electric Station, Unit 3  
Docket No. 50-382  
License No. NPF-38

REFERENCES: 1. Entergy letter to the NRC dated June 20, 2004, "License Amendment Request to Modify Technical Specification (TS) 5.3.1, Fuel Assemblies and TS 6.9.1.11.1, Core Operating Limits" (W3F1-2004-0036)  
2. Entergy letter to the NRC dated October 18, 2004, "Supplement to Amendment Request NPF-38-258, Modify Technical Specification (TS) 5.3.1, Fuel Assemblies and TS 6.9.1.11.1, Core Operating Limits" (W3F1-2004-0099)  
3. Entergy letter to the NRC dated February 2, 2005, "Supplement to Amendment Request NPF-38-258, Modify Technical Specification (TS) 5.3.1, Fuel Assemblies and TS 6.9.1.11.1, Core Operating Limits" (W3F1-2005-0003)

Dear Sir or Madam:

By letters (References 1, 2 and 3), Entergy Operations, Inc. (Entergy) proposed a change to the Waterford Steam Electric Station, Unit 3 (Waterford 3) Technical Specifications (TSs) to support Cycle 14 core reload.

On February 16, 2005, Entergy and members of your staff held a conference call to discuss the need to include additional information in the proposed TS associated with the removal of a reference to a Combustion Engineering Topical Report number CENPD-107 from TS 6.9.1.11.1, item 6) "CESEC-Digital Simulation for a Combustion Engineering Nuclear Steam Supply System." As a result of the call, Entergy will add reference information, "CE letter LD-82-001 and NRC SE to CE dated April 3, 1984," to the proposed TS change. Please see the attached markup of the affected TS page. No changes are proposed to any of the other marked up TS pages or to the inserts included in the original submittal dated June 20, 2004.


There are no technical changes proposed. The original no significant hazards consideration included in Reference 1 is not affected by any information contained in the supplemental letter. There are no new commitments contained in this letter.

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If you have any questions or require additional information, please contact Dana Millar at 601-368-5445.

I declare under penalty of perjury that the foregoing is true and correct. Executed on February 21, 2005.

Sincerely,

A handwritten signature in black ink, appearing to read "Alan J. Harts". The signature is fluid and cursive, with the first name "Alan" and last name "Harts" clearly distinguishable.

AJH/DM

Attachments:

1. Revised Markup of Technical Specification Pages

cc: Dr. Bruce S. Mallett  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

NRC Senior Resident Inspector  
Waterford 3  
P.O. Box 822  
Killona, LA 70066-0751

U.S. Nuclear Regulatory Commission  
Attn: Mr. Nageswaran Kalyanam MS O-7D1  
Washington, DC 20555-0001

Wise, Carter, Child & Caraway  
Attn: J. Smith  
P.O. Box 651  
Jackson, MS 39205

Winston & Strawn  
Attn: N.S. Reynolds  
1400 L Street, NW  
Washington, DC 20005-3502

Louisiana Department of Environmental Quality  
Office of Environmental Compliance  
Surveillance Division  
P. O. Box 4312  
Baton Rouge, LA 70821-4312

American Nuclear Insurers  
Attn: Library  
Town Center Suite 300S  
29<sup>th</sup> S. Main Street  
West Hartford, CT 06107-2445

**Attachment 1**

**To**

**W3F1-2005-0013**

**Revised Markup of Technical Specification Pages**

(CE letter LD-82-001 and NRC  
SE to CE dated April 3, 1984)

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT COLR (Continued)

6) "CESEC - Digital Simulation for a Combustion Engineering Nuclear Steam Supply System," ~~CENPD-107~~ (Methodology for Specification 3.1.1.1 and 3.1.1.2 for Shutdown Margins, 3.1.1.3 for MTC, 3.1.3.1 for Movable Control Assemblies - CEA Position, 3.1.3.6 for Regulating and group P CEA Insertion Limits, and 3.2.3 for Azimuthal Power Tilt).

7) "Qualification of Reactor Physics Methods for the Pressurized Water Reactors of the Entergy System," ENEAD-01-P. (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margins, 3.1.1.3 for MTC, 3.1.3.6 for Regulating and group P CEA Insertion Limits, 3.1.2.9 Boron Dilution (Calculation of CBC & IBW), and 3.9.1 Boron Concentration).

8) "Fuel Rod Maximum Allowable Gas Pressure," CEN-372-P-A. (Methodology for Specification 3.2.1, Linear Heat Rate).

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6.9.1.11.2 The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.11.3 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted in accordance with 10 CFR 50.4 within the time period specified for each report.

6.10 Not Used