



SMUD

SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

MPC&D 04-120

January 24, 2005

U.S. Nuclear Regulatory Commission
Attn.: Document Control Desk
Washington, DC 20555

Docket No. 50-312
Rancho Seco Nuclear Generating Station
License No. DPR-54
**PROPOSED LICENSE AMENDMENT NO. 198 TO CLEAN UP FACILITY
OPERATING LICENSE DPR-54**

Attention: John Hickman

In accordance with 10 CFR 50.90, we are submitting Proposed Amendment No. 198 (PA-198) to Rancho Seco Facility Operating License DPR-54. The proposed license amendment removes unnecessary and obsolete information from the facility operating license. As discussed in Attachment 2, we have concluded that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c). Accordingly, a finding of "no significant hazards consideration" is justified.

Attachment 1 provides the insertion removal instructions for the affected pages. Attachment 2 provides the description and analysis of the proposed change and the No Significant Hazards Consideration. Attachment 3 provides a copy of the revised Facility Operating License pages and Attachment 4 provides a redline/strikeout copy of the affected pages to facilitate NRC review.

In accordance with 10 CFR 50.91(b)(1), we have informed the Radiological Health Branch of the California State Department of Health Services of the proposed amendment by sending them a copy of this submittal package.

NLMSS01

Members of your staff with questions requiring additional information or clarification may contact Robert Jones at (916) 732-4843.

Sincerely,



Steve Redeker
Manager, Plant Closure and Decommissioning

Attachments (4)

Cc w/ attachments: B.S. Mallett, NRC, Region IV
Radiological Health Branch, California State Department of Health
Services

CALIFORNIA ALL-PURPOSE ACKNOWLEDGMENT

State of California
County of Sacramento } ss.

On 1/24/05, before me, Christina Kemmler, Notary Public
Date Name and Title of Officer (e.g., "Jane Doe, Notary Public")
personally appeared Steve Redeker
Name(s) of Signer(s)

- personally known to me
- proved to me on the basis of satisfactory evidence

to be the person(s) whose name(s) is/are subscribed to the within instrument and acknowledged to me that he/she/they executed the same in his/her/their authorized capacity(ies), and that by his/her/their signature(s) on the instrument the person(s), or the entity upon behalf of which the person(s) acted, executed the instrument.



WITNESS my hand and official seal.

[Signature]
Signature of Notary Public

Place Notary Seal Above

OPTIONAL

Though the information below is not required by law, it may prove valuable to persons relying on the document and could prevent fraudulent removal and reattachment of this form to another document.

Description of Attached Document

Title or Type of Document: Proposed License Amendment

Document Date: Jan. 24, 2005 Number of Pages: 2 (w/4 attachments)

Signer(s) Other Than Named Above: _____

Capacity(ies) Claimed by Signer

- Signer's Name: Steve Redeker
- Individual
 - Corporate Officer — Title(s): manager, plant closure and decommissioning
 - Partner — Limited General
 - Attorney in Fact
 - Trustee
 - Guardian or Conservator
 - Other: _____

Signer Is Representing: SMUD



Attachment 1

Rancho Seco Facility Operating License

Proposed License Amendment No. 198

REMOVAL/INSERTION INSTRUCTIONS

REMOVE OPERATING LICENSE

Pages 1 through 10

INSERT OPERATING LICENSE

Pages 1 through 4

Attachment 2

***Description of the Proposed Change, and No Significant Hazards
Consideration***

Background

The Sacramento Municipal Utility District (SMUD) shut down Rancho Seco Nuclear Generating Station permanently on June 7, 1989, after approximately 15 years of operation. On August 29, 1989, SMUD formally informed the NRC that the plant was shut down permanently. On May 20, 1991, SMUD submitted the Rancho Seco decommissioning plan and on March 20, 1995, the NRC issued an Order approving the decommissioning plan and authorizing the decommissioning of Rancho Seco.

SMUD began actively decommissioning Rancho Seco in February 1997, and completed the transfer of all of the spent nuclear fuel to the 10 CFR Part 72 licensed Independent Spent Fuel Storage Installation (ISFSI) on August 21, 2002. Accordingly, the only quality-related structures, systems, or components (SSCs) at the Rancho Seco 10 CFR Part 50 licensed site are the radioactive sources used to calibrate the instrumentation used to measure radioactivity in gaseous and liquid effluents.

Plant dismantlement is substantially complete and most of the SSCs that *were* safety-related or important-to-safety have been removed from the plant and shipped for disposal. The pressurizer was shipped to Envirocare for disposal in April 2004. Removal of the steam generators is in progress. One steam generator was shipped to Envirocare in December 2004, and the second is scheduled for spring 2005. Activities in preparation for the reactor vessel internals segmentation are underway and mobilization of the segmentation contractor is schedule to begin in early 2005.

Description of the Proposed Operating License Change

The proposed amendment (PA-198) removes unnecessary and obsolete information from the facility operating license. These changes are editorial and administrative and are consistent with the NRC's desire to improve operating license content by removing inappropriate and unnecessary information. The proposed amendment complies with current regulatory requirements and does not alter or modify any feature associated with the Rancho Seco facility or the site.

Table 1 identifies each section of the operating license and provides a description of the license condition, the proposed revision, and the basis for the proposed change.

As discussed further in this Attachment, we have concluded that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c). Accordingly, a finding of "no significant hazards consideration" is justified.

No Significant Hazards Consideration

SMUD has reviewed the proposed license and PDTs change against each of the criteria in 10 CFR 50.92 and has concluded that the amendment request involves no significant hazards consideration. The following provides SMUD's analysis of the issue of no significant hazards consideration:

1. Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The proposed changes are administrative and involve deleting unnecessary and obsolete information from the facility operating license. These changes do not affect possible initiating events for accidents previously evaluated or alter the configuration or operation of the facility. Safety limits, limiting safety system settings, and limiting control systems are no longer applicable to Rancho Seco in the permanently defueled mode, and are therefore not relevant.

The proposed changes do not affect the boundaries used to evaluate compliance with liquid or gaseous effluent limits, and have no impact on plant operations. Therefore, the proposed license amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed license amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

No. As described above, the proposed changes are administrative. The safety analysis for the facility remains complete and accurate. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid.

The operating procedures and emergency procedures are not affected. The proposed changes do not affect the emergency planning zone, the boundaries used to evaluate compliance with liquid or gaseous effluent limits, and have no impact on plant operations. Consequently, no new failure modes are introduced as the result of the proposed changes. Therefore, the proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed license amendment involve a significant reduction in a margin of safety?

No. As described above, the proposed changes are administrative. There are no changes to the design or operation of the facility. The proposed changes do not affect the emergency planning zone, the boundaries used to evaluate compliance with liquid or gaseous effluent

limits, and have no impact on plant operations. Accordingly, neither the design basis nor the accident assumptions in the Defueled Safety Analysis Report (DSAR), nor the Technical Specification Bases are affected. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Section 1 (Subsections 1.A through 1.I)	Section 1 is an historical proclamation that is not subject to change.	N/A
Subsection 2.A	<p>Subsection 2.A specifies that the Rancho Seco facility is described in the Final Safety Analysis Report (FSAR) and the Environmental Report, as supplemented and amended. The FSAR has been replaced by the Defueled Safety Analysis Report (DSAR) and the Environmental Report has been supplemented with the "Supplement to the Environmental Report – Post Operating Phase."</p> <p>This subsection is being revised editorially to remove the parenthetical information that names the specific amendment numbers.</p>	The specific amendment numbers contained in the parenthesis in Section 2.A are no longer accurate. The FSAR has been replaced by the Defueled Safety Analysis Report (DSAR) and the Environmental Report has been supplemented with the "Supplement to the Environmental Report – Post Operating Phase."
Subsection 2.B, Paragraphs (1) through (5)	Subsection 2.B, Paragraphs (1) through (5) continue to apply and will continue to be implemented by various Radiation Protection, Surveillance, and Administrative procedures.	N/A

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Subsection 2.C, Paragraph (1)	Subsection 2.C, Paragraph (1) establishes the license condition that revokes SMUD's authority to operate the reactor. This paragraph continues to apply and is implemented by plant administrative procedures.	N/A
Subsection 2.C, Paragraph (2)	Subsection 2.C, Paragraph (2) specifies that the Permanently Defueled Technical Specifications (PDTS) requirements are license conditions and are included in Appendix A to the license. Subsection 2.C, Paragraph (2) also identifies the most recent amendment to the PDTS. This paragraph continues to apply and will remain unchanged.	N/A
Subsection 2.C, Paragraph (3)	This section was deleted in Amendment 131.	N/A

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Subsection 2.C, Paragraph (4)	<p>Subsection 2.C, Paragraph (4) contains fire protection requirements that are applicable to an operating nuclear power plant. Decommissioning Order, Section IV(B) contains updated fire protection program requirements for the permanently shutdown and defueled Rancho Seco plant. The requirements in Decommissioning Order, Section IV(B) have been incorporated into the Rancho Seco Defueled Safety Analysis Report (DSAR).</p> <p>Since the requirements in Decommissioning Order, Section IV(B) have been incorporated into the Rancho Seco DSAR, Subsection 2.C, Paragraph (4) may be deleted from the operating license.</p>	<p>The fire protection program requirements in the Decommissioning Order are appropriate given the permanently shutdown and defueled conditions at the Rancho Seco plant. Since the requirements in Decommissioning Order have been incorporated into the Rancho Seco DSAR, Subsection 2.C, Paragraph (4) may be deleted from the operating license.</p>
Subsection 2.C, Paragraphs (5) and (6)	<p>Subsection 2.C, Paragraphs (5) and (6) requires supplementary in-service tests and inspections. These requirements are no longer applicable with Rancho Seco permanently shutdown and defueled.</p> <p>Subsection 2.C, Paragraphs (5) and (6) will be deleted.</p>	<p>These requirements are no longer applicable with Rancho Seco permanently shutdown and defueled.</p>

Table 1

Proposed License Revision

<p>Facility Operating License Section</p>	<p>Description of License Condition and Proposed Revision</p>	<p>Reason for Revision</p>
<p>Subsection 2.C, Paragraph (7)</p>	<p>Subsection 2.C, Paragraph (7) requires the implementation of a leakage reduction program on systems outside of containment that would or could contain highly radioactive fluids during a serious transient or accident. The type of transients or accidents for which this program was originally intended are no longer possible given the decommissioning status of the plant.</p> <p>Accordingly, Subsection 2.C, Paragraph (7) will be deleted.</p>	<p>The type of transients or accidents for which this program was originally intended are no longer possible given the decommissioning status of the plant.</p>
<p>Subsection 2.C, Paragraphs (8) through (11)</p>	<p>Subsection 2.C, Paragraphs (8) through (11) were deleted in previous amendments.</p>	<p>N/A</p>

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Subsection 2.C, Paragraph (12)	<p>Subsection 2.C, Paragraph (12) addresses NUREG-0737 Control Room habitability requirements. With all of the spent nuclear fuel in dry storage at the Rancho Seco Independent Spent Fuel Storage Installation (ISFSI), the Rancho Seco control room is no longer in use.</p> <p>Accordingly, Subsection 2.C, Paragraph (7) will be deleted.</p>	<p>With all of the spent nuclear fuel in dry storage at the Rancho Seco ISFSI, the Rancho Seco control room is no longer in use.</p>
Subsection 2.C, Paragraph (13)	<p>Subsection 2.C, Paragraph (13) is an NRC Confirmatory Order that prohibits the movement of nuclear fuel into the Rector Building. This paragraph will remain but will be re-numbered to reflect previously deleted paragraphs.</p>	<p>Re-numbering this paragraph to reflect previously deleted paragraphs is editorial and does not affect the specific requirement.</p>

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Subsection 2.D	<p>Subsection 2.D requires SMUD to provide the NRC with an analysis, including corrective actions, if the environmental monitoring programs included in the Technical Specifications show harmful effects or irreversible damage to the environment. The environmental monitoring programs that were previously included in the Technical Specifications have been relocated to the Rancho Seco Quality Manual (Appendix A).</p> <p>This requirement continues to apply and will remain as a license condition. Subsection 2.D will be revised to reflect that the environmental monitoring program requirements have been relocated to the Rancho Seco Quality Manual.</p>	Revising Subsection 2.D to reflect that the environmental monitoring program requirements have been relocated to the Rancho Seco Quality Manual is editorial and does not affect the intend of the license condition or any environmental monitoring program requirements.
Subsection 2.E	This paragraph continues to apply and will remain unchanged.	N/A

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
<p>Order for Modification of License, dated 07/21/78.</p>	<p>The Order for Modification of License, dated 07/21/78, requires a re-evaluation of Emergency Core Cooling System performance, provides limits on steady state reactor core power, and set a completion time for operator action following accidents that occur during power operations. This Order is no longer applicable with Rancho Seco permanently shutdown and defueled.</p> <p>Accordingly, this Order will be deleted.</p>	<p>The Order for Modification of License, dated 07/21/78, is no longer applicable with Rancho Seco permanently shutdown and defueled.</p>
<p>Order for Modification of License, dated 04/01/80.</p>	<p>Order for Modification of License, dated 04/01/80, Item #1 concerns the independent verification of valve line-ups and equipment operability whenever safety-related equipment is removed from, or placed into, service. Items 2 and 3 refer to historical events that happened in 1980. These items are editorial in nature and may be deleted.</p> <p>With all of the spent nuclear fuel in dry storage at the Rancho Seco ISFSI, there is no longer any safety-related equipment at Rancho Seco.</p> <p>Accordingly, this Order will be deleted.</p>	<p>With all of the spent nuclear fuel in dry storage at the Rancho Seco ISFSI, there is no longer any safety-related equipment at Rancho Seco. Therefore, the Order for Modification of License, dated 04/01/80, is no longer applicable.</p>

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Order for Modification of License, dated 09/19/80.	<p>Order for Modification of License, dated 09/19/80, deals with the implementation of I&E Bulletin 79-01B. The issues addressed in this bulleting are no longer applicable with Rancho Seco permanently shutdown and defueled.</p> <p>Accordingly, this Order will be deleted.</p>	<p>Order for Modification of License, dated 09/19/80, deals with the implementation of I&E Bulletin 79-01B and is no longer applicable with Rancho Seco permanently shutdown and defueled.</p>
Order for Modification of License, dated 04/20/81.	<p>Order for Modification of License, dated 04/20/81, implements new PCS pressure isolation valve Technical Specification surveillances and LCOs and check valve-testing requirements. These valve-testing requirements are no longer applicable and have been deleted from the Technical Specifications.</p> <p>Accordingly, this Order will be deleted.</p>	<p>The PCS pressure isolation valve Technical Specification surveillances and LCOs and check valve testing requirements contained in NRC Order for Modification of License, dated 04/20/81, are no longer applicable and have been deleted from the Technical Specifications.</p>

Table 1

Proposed License Revision

Facility Operating License Section	Description of License Condition and Proposed Revision	Reason for Revision
Order for Modification of License, dated 12/10/82.	<p>Order for Modification of License, dated 12/10/82, requires designing and installing an ICC instrumentation system. This is a reactor coolant inventory monitoring issue that comes from NUREG-0737. This requirement is no longer applicable with Rancho Seco permanently shutdown and defueled.</p> <p>Accordingly, this Order will be deleted.</p>	The reactor coolant inventory monitoring requirement specified in NRC Order for Modification of License, dated 12/10/82, is no longer applicable with Rancho Seco permanently shutdown and defueled.
Order for Modification of License, dated 05/02/90.	<p>Order for Modification of License, dated 05/02/90, prohibits placing any nuclear fuel into the Rancho Seco Reactor Building without prior NRC approval.</p> <p>This requirement is redundant with Subsection 2.C, Paragraph (13) and may be deleted.</p>	This requirement is redundant with Subsection 2.C, Paragraph (13) and may be deleted.

Attachment 3

Revised Facility Operating License Pages

UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, DC 20545

SACRAMENTO MUNICIPAL UTILITY DISTRICT

DOCKET NO. 50-312

(Rancho Seco Nuclear Generating Station)

FACILITY OPERATING LICENSE

License No. DPR-54

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by Sacramento Municipal Utility District (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Rancho Seco Nuclear Generating Station (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-56 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
 - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-54, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
 - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Part 30, 40, and 70, including 10 CFR Section 30.33, 40.32, 70.23, and 70.31.
2. Facility Operating License No. DPR-54 is hereby issued to the Sacramento Municipal Utility District to read as follows:
- A. This license applies to the Rancho Seco Nuclear Generating Station, a pressurized water reactor and associated equipment (the facility), owned by the Sacramento Municipal Utility District. The facility is located in Sacramento County, California, and is described in the "Final safety Analysis Report" as supplemented and amended and the Environmental Report as supplemented and amended.
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Sacramento Municipal Utility District
 - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Sacramento County, California, in accordance with the procedures and limitations set forth in this license;

(Amendment 117 - 03/17/92)

- (2) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources, sealed sources for reactor instrumentation and radiation monitoring equipment, calibration, and as fission detectors in amounts as previously required for reactor operation; and to possess but not use special nuclear material previously received as reactor fuel;

(Amendment 117 - 03/17/92)

- (3) Pursuant to the Act and 10 CFR Part 30, to receive, possess, and use at any time 100 millicuries each of any byproduct material, without restriction to chemical or physical form, for sample analysis or instrument calibration;

(Amendment 117 - 03/17/92)

- (4) Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time 100 milligrams each of any source or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration;

(Amendment 117 - 03/17/92)

- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as were produced by the previous operation of the facility.

(Amendment 117 - 03/17/92)

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- (1) Maximum Power

The licensee is not authorized to operate the reactor.

(Amendment 117 - 03/17/92)

- (2) Permanently Defueled Technical Specifications

The Permanently Defueled Technical Specifications contained in Appendix A, as revised through Amendment No. 131, are hereby incorporated in the license. Sacramento Municipal Utility District shall maintain the facility in accordance with the Permanently Defueled Technical Specifications.

(Amendment 131 - 10/10/02)

(3) Confirmatory Order

The movement of nuclear fuel into the Reactor Building is prohibited without prior NRC approval.

(Amendment 117 - 03/17/92)

- D. This license is subject to the following additional condition for the protection of the environment:

If harmful effects or evidence of irreversible damage are detected by the monitoring programs included in the Rancho Seco Quality Manual, the Applicant will provide an analysis of the problem and a proposed course of action to alleviate the problem.

- E. This license is effective as of the date of issuance and shall expire at midnight, October 11, 2008.

FOR THE ATOMIC ENERGY COMMISSION

/s/ ROGER BOYD for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Date of Issuance: August 16, 1974

Attachment: Appendix A - Technical Specifications (Amendment 120 - 10/13/92)

Attachment 4

Redline/Strikeout Version of the Affected Pages

UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, DC 20545

SACRAMENTO MUNICIPAL UTILITY DISTRICT

DOCKET NO. 50-312

(Rancho Seco Nuclear Generating Station)

FACILITY OPERATING LICENSE

License No. DPR-54

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by Sacramento Municipal Utility District (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Rancho Seco Nuclear Generating Station (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-56 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. The licensee is technically and financially qualified to engage in the activities authorized by this operating license in accordance with the rules and regulations of the Commission;
 - F. The licensee has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-54, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
 - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Part 30, 40, and 70, including 10 CFR Section 30.33, 40.32, 70.23, and 70.31.
2. Facility Operating License No. DPR-54 is hereby issued to the Sacramento Municipal Utility District to read as follows:
- A. This license applies to the Rancho Seco Nuclear Generating Station, a pressurized water reactor and associated equipment (the facility), owned by the Sacramento Municipal Utility District. The facility is located in Sacramento County, California, and is described in the "Final safety Analysis Report" as supplemented and amended (~~Amendments 1 through 28~~) and the Environmental Report as supplemented and amended. (~~Amendments 1 and 2~~).
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Sacramento Municipal Utility District
 - (1) Pursuant to Section 104b of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Sacramento County, California, in accordance with the procedures and limitations set forth in this license;

(Amendment 117 - 03/17/92)

- (2) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources, sealed sources for reactor instrumentation and radiation monitoring equipment, calibration, and as fission detectors in amounts as previously required for reactor operation; and to possess but not use special nuclear material previously received as reactor fuel;

(Amendment 117 - 03/17/92)

- (3) Pursuant to the Act and 10 CFR Part 30, to receive, possess, and use at any time 100 millicuries each of any byproduct material, without restriction to chemical or physical form, for sample analysis or instrument calibration;

(Amendment 117 - 03/17/92)

- (4) Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time 100 milligrams each of any source or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration;

(Amendment 117 - 03/17/92)

- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as were produced by the previous operation of the facility.

(Amendment 117 - 03/17/92)

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- (1) Maximum Power

The licensee is not authorized to operate the reactor.

(Amendment 117 - 03/17/92)

- (2) Permanently Defueled Technical Specifications

The Permanently Defueled Technical Specifications contained in Appendix A, as revised through Amendment No. 131, are hereby incorporated in the license. Sacramento Municipal Utility District shall maintain the facility in accordance with the Permanently Defueled Technical Specifications.

(Amendment 131 - 10/10/02)

(3) ~~Intentionally Deleted~~

~~(Amendment 131 - 10/10/02)~~

~~(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program, as described in the Fire Protection Plan for Rancho Seco Nuclear Generating Station, and as approved in the SER dated September 10, 1990, subject to the following provision:~~

~~The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire:~~

~~(Amendment 115 - 09/10/90)~~

~~(5) The licensee shall perform the supplementary in-service tests and inspections of Nuclear Class 2 and Class 3 systems and components committed to in the letter of February 27, 1978, from J. J. Mattimoe to R. W. Reid:~~

~~(Amendment 20 - 05/30/78)~~

~~(6) In accordance with the provisions of 10 CFR 50.12(a) an exemption from the provisions of 10 CFR 50.55a(g)(4)(v) is granted, and the effective date for the start of the next 40-month period is extended from August 18, 1978 to October 18, 1979:~~

~~(Amendment 20 - 05/30/78)~~

(7) ~~Systems Integrity~~

~~The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:~~

- ~~1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and~~
- ~~2. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals:~~

~~(Amendment 31 - 03/27/81)~~

~~(8) Iodine Monitoring~~

~~Deleted~~

~~(Amendment 117 - 03/17/92)~~

~~(9) Backup Method for Determining Sub-cooling Margin~~

~~Deleted~~

~~(Amendment 117 - 03/17/92)~~

~~(10) IAEA Safeguards~~

~~Deleted~~

~~(Amendment 65 - 04/01/85)~~

~~(11) High Pressure Injection (HPI) Nozzles C and D Inspection~~

~~Deleted~~

~~(Amendment 117 - 03/17/92)~~

~~(12) (a) The schedule for completion of Item III.D.3.4 under Section III of the Commission's March 14, 1983, as revised November 10, 1983, Order Confirming Licensee Commitments on Post-TMI-Related Issues, is revised to show the licensee's completion date as start-up from the refueling outage currently estimated to begin in September 1986.~~

~~(b) Prior to start-up from the refueling outage which started in March 1985, the licensee will:~~

~~1. complete the installation of a redundant, two-train Heating Ventilating and Air Conditioning System and, on loss of off-site power, load the B train automatically onto the existing diesel generators; and~~

~~2. change procedures to reflect the interim installation, and instruct the operators on the modified procedures.~~

~~(Amendment 67 - 05/30/85)~~

~~(13) Confirmatory Order~~

The movement of nuclear fuel into the Reactor Building is prohibited without prior NRC approval.

(Amendment 117 - 03/17/92)

- D. This license is subject to the following additional condition for the protection of the environment:

If harmful effects or evidence of irreversible damage are detected by the monitoring programs included in the Technical Specifications Rancho Seco Quality Manual, the Applicant will provide an analysis of the problem and a proposed course of action to alleviate the problem.

- E. This license is effective as of the date of issuance and shall expire at midnight, October 11, 2008.

FOR THE ATOMIC ENERGY COMMISSION

/s/ ROGER BOYD for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Date of Issuance: August 16, 1974

Attachment: Appendix A - Technical Specifications (Amendment 120 - 10/13/92)

Orders for Modification of License

~~1. As soon as possible, the licensee shall submit a re-evaluation of ECCS cooling performance calculated in accordance with the B&W Evaluation Model for operation with operating procedures described in its letters of April 14, 1978 and April 21, 1978, which is wholly in conformance with 10 CFR 50.46 except for the credit for operator action within 10 minutes after initiation of the event.~~

~~2. The steady state reactor core power level shall not exceed 2772 Mwt.~~

~~3. Until further authorization by the Commission, the licensee shall operate in accordance with the procedures described in its letter of April 14, 1978, supplemented by letters dated April 21, and July 7, 1978, except that the maximum time for completion of operator action shall be 10 minutes after initiation of the event, and~~

~~4. As soon as possible, the licensee shall submit a description and safety evaluation of a proposed plant modification which will eliminate reliance on prompt operator action described herein.~~

(Order dated
07/21/78)

~~1. License DPR-54 is modified, as follows:~~

~~(a) Administrative Procedures shall be adopted and implemented to require independent verification of valve line-ups and equipment operability whenever safety-related equipment is removed from or placed into service.~~

~~(b) Existing administrative procedures and controls shall be reviewed and revised as necessary to assure that operating procedures are readily updated and maintained to reflect changes in safety-related plant systems or their operation.~~

~~2. The licensee shall complete the actions required by Paragraphs 1(a) and 1(b) by April 14, 1980, and submit a report of the results of the review required by paragraph 1(b) to the Director of NRC's Region V office by April 25, 1980.~~

~~3. The licensee shall meet with the Office of Inspection and Enforcement management, on or before May 5, 1980, in a meeting open to the public in the vicinity of the Rancho Seco site to describe how the above requirements will be implemented. The Director, Region V, will inform the licensee at least one week in advance of the specific time and location of the meeting.~~

~~(Order dated~~
~~04/01/80)~~

~~Information which fully and completely responds to the staff's request as specified in I&E Bulletin 79-01B, shall be submitted to the Director, Region V, Office of Inspection and Enforcement, by the licensee not later than November 1, 1980.~~

~~(Order dated~~
~~09/19/80)~~

~~1. Implement Technical Specifications (Attachment 3) which require periodic surveillance over the life of the plant and which specify limiting conditions for operation for PCS pressure isolation valves.~~

~~2. If check valves have not been (a) individually tested within 12 months preceding the date of the Order, and (b) found to comply with the leakage rate criteria set forth in the Technical Specifications described in Attachment 3, the MOV in each line shall be closed within 30 days of the effective date of this Order and quarterly In-service Inspection (ISI) MOV cycling ceased until the check valve tests have been satisfactorily accomplished. (Prior to closing the MOV, procedures shall be implemented and operators trained to assure that the MOV remains closed. Once closed, the MOV shall be tagged closed to further preclude inadvertent valve opening.)~~

~~3. The MOV shall not be closed as indicated in paragraph 2 above unless a supporting safety evaluation has been prepared. If the MOV is in an emergency core cooling system (ECCS), the safety evaluation shall include a determination as to whether the requirements of 10 CFR 50.46 and Appendix K to 10 CFR Part 50 will continue to be satisfied with the MOV closed. If the MOV is not in an ECCS, the safety evaluation shall include a determination as to whether operation with the MOV closed presents an unreviewed safety question as defined in 10 CFR 50.59(a)(2). If the requirements of 10 CFR 50.46 and Appendix K have not been satisfied, or if an unreviewed safety question exists as defined in 10 CFR 50.59, then the facility shall be shut down within 30 days of the date of this Order and remain shutdown until check valves are satisfactorily tested in accordance with the Technical Specifications set forth in Attachment 3.~~

~~4. The records of the check valve tests required by this Order shall be made available for inspection by the NRC's Office of Inspection and Enforcement.~~

(Order dated
04/20/81)

~~1. The licensee shall install an ICC instrumentation system consisting of sub-cooling margin monitors, core exit thermocouples and a reactor coolant inventory tracking system all of which conform to the design parameters specified in NUREG-0737, Item 11.F.2. Installation of the ICC instrumentation system shall be completed and made operational in accordance with the timetables approved pursuant to the terms of this Order.~~

~~2. Within 90 days of the date of this Order, the licensee must complete a conceptual design review for a reactor coolant inventory tracking system, identify the design selected and submit to the Director, Division of Licensing detailed schedules for engineering, procurement and installation of the inventory tracking system. References to generic design descriptions, where applicable, are acceptable.~~

~~3. Within 90 days of the date of this Order, the licensee shall review the status of conformance of all components of the ICC instrumentation system with NUREG-0737, Item 11.F.2, and submit a report on the status of such conformance. For your convenience in performing the status review, a checklist of the nine items of documentation cited on pp. 11.F.2-3 and 4 of that document is provided in the Appendix to this Order.~~

~~4. The licensee shall develop a schedule for installation and making operational the ICC instrumentation system which ensures installation during the earliest refueling shutdown consistent with the existing status of the plant and practical design and procurement considerations. This schedule shall be subject to approval by the Director, Division of Licensing.~~

~~5. Prior to using the installed ICC instrumentation system as a basis for operator decisions or actions, final documentation, including calibration data and proposed emergency procedure revisions, shall be submitted for NRC review and approval, and the task analysis portion of the control room design review must be completed by the licensee. Prior to NRC approval, the system may be used for the purpose of operator training and familiarization and may be used with prudence as supplemental input to plant operating decisions.~~

~~6. The licensee may request from the Commission an extension of time for submittal of the required information. Such a request must set forth the proposed schedule and justification for the delay. Such request shall be directed to the Secretary of the Commission, U.S. NRC, Washington, DC 20555. Any such request must be submitted at least 30 days prior to the date the information is required.~~

12/10/82) (Order dated

~~The licensee is prohibited from placing any nuclear fuel into the Rancho Seco reactor building without prior approval from the NRC.~~

05/02/90) (Order dated