Form ES-301-1

Facility: <u>Limerick Generating</u> Examination Level (circle or	-	Date of Examination: <u>01/10/2005</u> Operating Test Number: t
Administrative Topic (See Note)	Type Code*	Describe activity to be performed
Conduct of Operations		
Conduct of Operations	(N)(S) (P)	2.1.10 Interpretation and Application of Overtime Limits
Equipment Control	(N)(S)	2.2.12 Review a Control Rod Exercise Test
Radiation Control	(N)(S)	2.3.10 Calculate Stay Time
Emergency Plan	(N)(S)	2.4.27 Determination of Equipment Availability for Fire Safe Shutdown
		or SROs. RO applicants require only 4 items unless nistrative topics, when 5 are required.
* Type Codes & Criteria: (C)ontrol roon	n
([)irect from I	bank (\leq 3 for ROs; \leq 4 for SROs & RO retakes)
٩)	l)ew or (M)c	odified from bank (<u>></u> 1)
(F)revious 2 e	exams (< 1; randomly selected)
(S)imulator	

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Administrative Topics Outline

Form ES-301-1

Facility: <u>Limerick Generating</u> Examination Level (circle or		Date of Examination: <u>01/10/05</u> Operating Test Number:(
Administrative Topic (See Note)	Type Code*	Describe activity to be performed
Conduct of Operations	(N)(S)	2.1.3 Complete a Turnover Checklist
Conduct of Operations	(N)(S) (P)	2.1.10 Interpretation and Application of Overtime Limits
Equipment Control	(N)(S)	2.2.12 Review a Control Rod Exercise Test
Radiation Control	(N)(S)	2.3.10 Calculate Stay Time
Emergency Plan	(N)(S)	2.4.41 ERP Classification and Reporting (SRO)
		or SROs. RO applicants require only 4 items unless nistrative topics, when 5 are required.
* Type Codes & Criteria: (C)ontrol room	n
(E)irect from b	bank (\leq 3 for ROs; \leq 4 for SROs & RO retakes)
(N	l)ew or (M)o	dified from bank (≥ 1)
(F)revious 2 e	exams (<u><</u> 1; randomly selected)
(S)imulator	

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Control Room/In-Plant Systems Outline

Facility: Limerick Generating Station		f Examination:								
Exam Level (circle one): RO	Operati	ng Test Numbe	er:1							
Control Room Systems ^e (8 for RO, 7 for SRO-I, 2 or	3 for SRO-U)									
System / JPM Title		Type Code*	Safety Function							
a. Reactor Recirc / Start MG Set - High Vibration o	ccurs / 202001 A3.02	(A)(N)(S)(L)	1							
b. Feedwater / Place 3 rd RFP In Service / 259001 A	4.02	(D)(S)	2							
c. Main Turbine / ST-6-001-660-1 Main Turbine CIN EOC/RPT Channel Functional Test / 241000 K4.		(N)(S)	3							
 RHR / Shutdown Cooling Flow Adjustments (RHI 205000 K1.15 	(A)(D)(S)(L) (P)	4								
e. Containment / T-228 Drywell Inerting with Nitroge	en / 500000 EA1.07	(A)(N)(S)	5							
f. A. C. Power / Supplying Power to a 480 VAC No Center from its alternate source / 262001 A2.10	n-Safeguard Load	(A)(N)(S)	6							
g. Instrumentation / ST-6-107-880-1, APRM and LPRM Noise Level (N)(S) 7 Determination / 215005 A4.06										
h. MCR HVAC / Manually Initiate a Control Room Chlorine Isolation / (D)(S) 9 290003 A3.01										
In-Plant Systems [®] (3 for RO, 3 for SRO-I, 3 or 2 for S	SRO-U)									
i. HCU / Scram Discharge Volume Draining 29501	5 AA1.01	(R)(D)(E)	1							
j. UPS / Bypassing and Removing the *A RPS and Service 262002 K6.02	UPS Static Inverter from	(R)(D)	6							
k. SRV / Inadvertent Opening of a Relief Valve 239	002 A2.03	(P)(R)(D) (E)	3							
@ All control room (and in-plant) systems must be d systems and functions may overlap those tested in the systems and functions may overlap those tested in the		it safety functio	ns; in-plant							
*Type Codes	Criteria for RO / SRO-I /	SRO-U								
(A)Iternate path	4-6	6/4-6/2-3								
(C)ontrol room										
(D)irect from bank	<u><</u> 9	/ ≤8 /≤4								
(E)mergency or abnormal in-plant	<u>≥</u> 1	/ <u>≥</u> 1 / <u>≥</u> 1								
(L)ow-Power	<u>></u> 1	/ <u>≥</u> 1 / <u>≥</u> 1								
(N)ew or (M)odified from bank including 1(A)	<u>≥</u> 2	/ ≥2 /≥1								
(P)revious 2 exams	<u><</u> 3	/ <u>≤</u> 3 / <u>≤</u> 2								
	(rando	mly selected)								
(R)CA	<u>≥</u> 1 /	′ <u>≥</u> 1 / <u>≥</u> 1								
(S)imulator ES-301 pa										

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Control Room/In-Plant Systems Outline

Form ES-301-2

Facility: <u>Limerick Generating Station</u> Exam Level (circle one): SRO(I)		Examination: <u>-</u> ng Test Numbe					
Control Room Systems [®] (8 for RO, 7 for SRO-I, 2 or	3 for SRO-U)	<u></u>					
System / JPM Title		Type Code*	Safety Function				
a. Reactor Recirc / Start MG Set - High Vibration o	ccurs / 202001 A3.02	(A)(N)(S)(L)	1				
b. Feedwater / Place 3rd RFP In Service / 259001 A	4.02	(D)(S)	2				
c. Main Turbine / ST-6-001-660-1 Main Turbine CIV EOC/RPT Channel Functional Test / 241000 K4.		(N)(S)	3				
 RHR / Shutdown Cooling Flow Adjustments (RHI 205000 K1.15 	RSW High Rad) /	(A)(D)(S)(L) (P)	4				
e. Containment / T-228 Drywell Inerting with Nitroge	en / 500000 EA1.07	(A)(N)(S)	5				
f. A. C. Power / Supplying Power to a 480 VAC No. Center from its alternate source / 262001 A2.10	n-Safeguard Load	(A)(N)(S)	6				
g. Instrumentation / ST-6-107-880-1, APRM and LF Determination / 215005 A4.06	(N)(S)	7					
h.							
In-Plant Systems [®] (3 for RO, 3 for SRO-I, 3 or 2 for 5	SRO-U)						
i. HCU / Scram Discharge Volume Draining 29501	5 AA1.01	(R)(D)(E)	1				
 J. UPS / Bypassing and Removing the *A RPS and Service 262002 K6.02 	UPS Static Inverter from	(R)(D)	6				
k. SRV / Inadvertent Opening of a Relief Valve 239	002 A2.03	(R)(D)(E) (P)	3				
@ All control room (and in-plant) systems must be d systems and functions may overlap those tested in the systems and functions may overlap those tested in the		t safety functio	ns; in-plant				
*Type Codes	Criteria for RO / SRO-I /	SRO-U					
(A)Iternate path	4-6	6/4-6/2-3					
(C)ontrol room							
(D)irect from bank	<u>≤</u> 9	/ <u><</u> 8 /≤4					
(E)mergency or abnormal in-plant	<u>≥ 1 / ≥1 /≥1</u>						
(L)ow-Power	≥1	/ ≥1 /≥1					
(N)ew or (M)odified from bank including 1(A)		/ ≥2 /≥1					
(P)revious 2 exams	<u>≤</u> 3	/ <u>≤</u> 3 / <u>≤</u> 2					
	(rando	mly selected)					
(R)CA	≥1 /	′ <u>≥</u> 1 / <u>≥</u> 1	:				
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Control Room/In-Plant Systems Outline

Form ES-301-2

Facility: Limerick Generating Station Exam Level (circle one): SRO(U)		Examination: _	
Control Room Systems [@] (8 for RO, 7 for SRO-I,	2 or 3 for SRO-U)		<u>.</u>
System / JPM Title		Type Code*	Safety Function
a. RHR / Shutdown Cooling Flow Adjustments (205000 K1.15	RHRSW High Rad) /	(A)(D)(S)(L) (P)	4
b. Containment / T-228 Drywell Inerting with Nit	rogen / 500000 EA1.07	(A)(N)(S)	5
 c. A. C. Power / Supplying Power to a 480 VAC Center from its alternate source 262001 A2.1 	•	(A)(N)(S)	6
d.			
e.	·····		
f.	, , , , <u>, , , , , , , , , , , , , , , </u>		
g.	· · · · · · · · · · · · · · · · · · ·		
h.			
In-Plant Systems [@] (3 for RO, 3 for SRO-I, 3 or 2	for SRO-U)		
i. HCU / Scram Discharge Volume Draining 29	5015 AA1.01	(R)(D)(E)	1
j. SRV / Inadvertent Opening of a Relief Valve	239002 A2.03	(R)(D)(E) (P)	3
k.			
@ All control room (and in-plant) systems must l plant systems and functions may overlap those to		erent safety fur	nctions; in-
*Type Codes	Criteria for RO / SRO-I	/ SRO-U	
(A)Iternate path	4.	-6/ 4-6 /2-3	
(C)ontrol room			
(D)irect from bank	≤	9/ <u>≤</u> 8 / <u>≤</u> 4	
(E)mergency or abnormal in-plant	≥	1 / ≥1 /≥1	
(L)ow-Power	≥	1 / <u>≥</u> 1 / <u>≥</u> 1	
(N)ew or (M)odified from bank including 1(A)	≥:	2/ <u>≥</u> 2 / <u>≥</u> 1	
(P)revious 2 exams		3/ <u>≤</u> 3/ <u>≤</u> 2	
	(ranc	lomly selected)	
(R)CA	1	/ ≥1 /≥1	
(S)imulator			

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Appendix D	
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Scenario Outline

Form ES-D-1

Facility:	Limerick Sc	enario No.:	1 Op-Test No.:
Examine	ers:		Operators:
		····	
mainten Turnove up to an operatin (step 4.4 mainten tomorrov for feed	ance r: A Reactor Star d including step 3 g for ST-6-049-2 4.30) should be c ance, and will be w. The bypass v water stability per	tup is in pro 3.4.15. Rea 30-1, RCIC ompleted wi out of servi alve jack is l GP-2 step	r Startup in progress, "1B" EHC pump blocked for gress with GP-2, Normal Plant Startup completed actor power is approximately 4%. RCIC is Pump Valve and Flow Test, and RT-6-049-701-1 ithin the next hour. "1B" EHC pump is blocked for ce until a new pressure compensator arrives being used to ensure partial bypass valve opening 3.3.20.4. The crew must continue power ascension reparation for main turbine startup. Control Rod
moveme	ent is per the Con	trol Rod Mo	ve Sheets, with Sequence Step Number 14, Rod rod to withdraw (12-48).
Event No.	Malf. No.	Event Type*	Event Description
1	N/A	R (RO/ CRS)	Withdraw control rods until 4 bypass valves open
2	Override FIC46-1R600 flow controller pot to 0	I (RO/ CRS)	CRD flow controller fails downscale in AUTO
3	MED280B	I (ALL)	Loss of 1BY160 due to underfrequency trip of a series supply breaker (TS)
4	MRD016F	C (RO/ CRS)	Control Rod 22-35 inadvertently scrams (TS)
5	MRC465 (0-100%)	M (ALL)	Steam Leak from RCIC piping
6	MRC464A MRC464B	C (PRO/ CRS)	RCIC isolation valves fail to close
7	MAD141D	C (PRO/ CRS)	"1E" SRV fails to open
*	(N)ormal, (R)ea	activity, (I)	nstrument, (C)omponent, (M)ajor

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Appendix D

Scenario Outline

Form ES-D-1

Facility:	Limerick Sc	enario No.:	4 Op-Test No.:
Examine	ers:	,	Operators:
	onditions: 83% po st is in progress.	ower, ST-6-0	052-232-1, B Loop Core Spray Pump, Valve and
Enginee ST-6-05 been co	ring is completin 2-232-1, B Loop mpleted up to an	g a ReMA. Core Spray d including s	owing a rod pattern adjustment. Reactor Power restoration will resume within the next hour. Pump, Valve and Flow Test is in progress and has step 4.7.21. "1D" Core Spray pump has been I 45 minutes for step 4.7.22.
Event No.	Malf. No.	Event Type*	Event Description
1	MVI231 B	I (ALL)	RPV instrument line break, XV42-1F045B closes causing Division 2 LOCA signal (TS)
2	MHP452 MHP453	C (PRO/ CRS)	HPCI Injection valves will fail to reopen after initial isolation/trip of HPCI
3	MSW489B	C (PRO/ CRS)	"0B" ESW Pump trip (TS)
4	Override HS05-102C to close	C (ALL)	"1C" Condensate Pump Discharge Valve strokes closed due to a short in the control switch
5	MRR507A	I (PRO/ CRS)	"1A" Reactor Recirculation MG Set fails to runback
6	MFW252B (0-100%)	M (ALL)	Feedwater line rupture inside Primary Containment
7	Override HS41-1F032B to fail " as-is"	C (ALL)	"B" Feedwater line isolation valve fails to close
8	MAD146B MAD146E	C (PRO/ CRS)	"1K" SRV opens
9	MAD151D	C (PRO/ CRS)	"1E" SRV downcomer leak
*	(N)ormal. (R)ea	activity (I)	nstrument. (C)omponent. (M)aior

APPENDIX D, Page 38 of 39

BWR Examination Outline

Form ES-401-1

Facility: Lim	erick												Date	of Exa	ım: 1/1	0/05
					RC) K/	A C	ate	gor	ÿР	SRO-Only Points					
Tier	Group	K 1	K 2	K 3	K 4	K 5	6 X	A 1	A 2	A 3	A 4	G	Total	A2	G*	Total
1.	1	3	4	5			8	4	3			1	20	4	3	7
Emergency	2	0	1	3			21.0°	2	0			1	7	2	1	3
& Abnormal Plant Evolutions	Tier Totals	3	5	8		N/A		6	3	N	/A	2	27	6	4	10
2.	1	3	2	4	4	1	2	3	1	3	2	1	26	3	2	5
Plant	2	1	0	1	2	1	2	1	1	1	1	1	12	1	2	3
Systems	Tier Totals	4	2	5	6	2	4	4	2	4	3	2	38	4	4	8
3. Generic Abilities	Knowlee Catego	-		d		1 3		2 2		3 4 2 3		-	10	1 2 2 2	3 4 1 2	7

Note:

 Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO outlines (i.e. except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).

2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ± 1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.

- 3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements..
- 4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.

5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.

- 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
- 7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.
- 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. Use duplicate pages for RO and SRO-only exams.
- 9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

2

ES-401 Eme	ergen	cy an					ation Outline Evolutions - Tier 1/Group 1 (RO)	Form E	S-401-
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G		IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4		0 3					Reactor water level	3.6	1
295003 Partial or Complete Loss of AC / 6	0 6						Station blackout: Plant-Specific	3.8	1
295004 Partial or Total Loss of DC Pwr / 6		0 3					D.C. bus loads	3.3	1
295005 Main Turbine Generator Trip / 3		0 1			0 7		RPS; Reactor water level	3.8; 3.5	2
295006 SCRAM / 1			0 1				Reactor water level response	3.8	1
295016 Control Room Abandonment / 7					0 2		Reactor water level	4.2	1
295018 Partial or Total Loss of CCW / 8			0 7				Cross-connecting with backup systems	3.1	1
295019 Partial or Total Loss of Inst. Air / 8		0 3					Reactor feedwater	3.2	1
295021 Loss of Shutdown Cooling / 4	0 1				i. P		Decay heat	3.6	1
295023 Refueling Acc / 8				0 6			Neutron monitoring	3.3	1
295024 High Drywell Pressure / 5	0 1						Drywell integrity: Plant-Specific	4.1	1
295025 High Reactor Pressure / 3			0 6			04. 9	Ireference to procedures those actions that require immediate	4.2; 4	2
295026 Suppression Pool High Water Temp. / 5				0 3			Temperature monitoring	3.9	1
295027 High Containment Temperature / 5									0
295028 High Drywell Temperature / 5					0 4		Drywell pressure	4.1	1
295030 Low Suppression Pool Wtr Lvl / 5			0 2				HPCI operation: Plant-Specific	3.5	1
295031 Reactor Low Water Level / 2			0 4				Steam cooling	4	1
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1				0 9			SPDS/ERIS/CRIDS/GDS; Plant-Specific	2.8	1
295038 High Off-site Release Rate / 9				0 1			Stack-gas monitoring system: Plant-Specific	3.9	1
600000 Plant Fire On Site / 8									0
K/A Category Totals:	3	4	5	4	3	1	Group Point Total:		20

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ES-401							tion Outline	Form E	S-401-
	rgen K	cy an K	d Abi K	A	al Pia		volutions - Tier 1/Group 2 (RO)	Б	<u> </u>
E/APE # / Name / Safety Function	1	2	3	1	2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									0
295007 High Reactor Pressure / 3									0
295008 High Reactor Water Level / 2									0
295009 Low Reactor Water Level / 2			0 1				Recirculation pump run back: Plant-Specific	3.2	1
295010 High Drywell Pressure / 5									0
295011 High Containment Temp / 5									0
295012 High Drywell Temperature / 5				0 2			Drywell cooling system	3.8	1
295013 High Suppression Pool Temp. / 5			0 2				Limiting heat additions	3.6	1
295014 Inadvertent Reactivity Addition / 1									0
295015 Incomplete SCRAM / 1				0 2			RPS	4	1
295017 High Off-site Release Rate / 9									0
295020 Inadvertent Cont. Isolation / 5 & 7			0 3				Drywell/containment temperature response	3.2	1
295022 Loss of CRD Pumps / 1									0
295029 High Suppression Pool Wtr Lvl / 5						01 32	Ability to explain and apply system limits and precautions.	3.4	1
295032 High Secondary Containment Area Temperature / 5									0
295033 High Secondary Containment Area Radiation Levels / 9									0
295034 Secondary Containment Ventilation High Radiation / 9									0
295035 Secondary Containment High Differential Pressure / 5									0
295036 Secondary Containment High Sump/Area Water Level / 5		<u> </u>			15				0
500000 High CTMT Hydrogen Conc. / 5		0 5					Hydrogen and oxygen recombiners	3.2	1
K/A Category Totals:	0	1	3	2	0	1	Group Point Total:	<u>.</u>	7

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ES-401						Ρ						tion Outline 2/Group 1 (RO)	Form E	S-401-
E/APE # / Name / Safety Function	K 1	К 2	К 3	К 4	K 5	К 6	_	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection				0 3								Pump minimum flow protection	3.2	1
205000 Shutdown Cooling Mode						0 1						A.C. electrical power	3.3	1
206000 HPCI			0 1									Reactor water level control: BWR-2, 3, 4	4	1
207000 Isolation (Emergency) Condenser														0
209001 LPCS									0 2	19671		Pump start	3.8	1
209002 HPCS														0
211000 SLC				0 8								System iniliation upon operation of SBLC control switch	4.2	1
212000 RPS									0 4			System status lights and alarms	3.9	1
215003 IRM										0 5		Trip bypasses	3.4	1
215004 Source Range Monitor			0 1				0 4					RPS; Control rod block status	3.4; 3.5	2
215005 APRM / LPRM							0 4					SCRAM and rod block trip setpoints	4.1	1
217000 RCIC		0 2										RCIC initiation signals (logic)	2.8	1
218000 ADS							0 5					Reactor water level	4.1	1
223002 PCIS/Nuclear Steam Supply Shutoff										0 3		Reset system isolations	3.6	1
239002 SRVs	0 8											Automatic depressurization system	4	1
259002 Reactor Water Level Control									0 6			Reactor water level setpoint setdown following a reactor scram: Plant-Specific	3	1
261000 SGTS	0 8					0 1						Process radiation monitoring system ; A.C. electrical distribution	2.8; 2.9	2
262001 AC Electrical Distribution											01. 10	Knowledge of conditions and limitations in the facility license.	2.7	1
262002 UPS (AC/DC)		 	1 0	0 1								Conlainment isolation: Plant-Specific; Transfer from preferred power to alternate power supplies	2.7; 3.1	2
263000 DC Electrical Distribution	0 2		0									Battery charger and battery; Emergency generators: Plant Specific	3.2; 3.4	2
264000 EDGs					0 6			0 4				Load sequencing; Consequences of operating under/over excited	3.4; 2.9	2
300000 Instrument Air		0 1					 		† —			Instrument air compressor	2.8	1
400000 Component Cooling Water				0 1					<u> </u>			Automatic start of standby pump	3.4	1
K/A Category Totals:	3	2	4	4	1	2	3	1	3	2	1	Group Point Total:		26

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ES-401			_										orm E	S-401-1
	к	к	к	к	к	PI K	lant A	Sys A	sterr A	ns -		2/Group 2 (RO)		
E/APE # / Name / Safety Function	1	2	3	4	5	6	1	A 2	3	4	G	K/A Topic(s)	IR	#
201001 CRD Hydraulic											Anti			0
201002 RMCS				0 3			L					Detection of drifting control rods	3.6	1
201003 Control Rod and Drive Mechanism	1.0000-2		-							SCHOOL		The instrument of a local and a second se	10050404018044444	0
201004 RSCS														0
201005 RCIS														0
201006 RWM				0 1								Insert blocks/errors: P-Spec(Not-BWR6)	3.4	1
202001 Recirculation														0
202002 Recirculation Flow Control			0 5					1860 - 1860 1860 - 1860 1860 - 1860				Recirculation pump speed: Plant-Specific	3.2	1
204000 RWCU						0 7						SBLC logic	3.3	1
214000 RPIS					Γ									0
215001 Traversing In-core Probe														0
215002 RBM					0 1							Trip reference selection: Plant-Specific	2.6	1
216000 Nuclear Boiler Inst.			Γ				Γ		and the second	Γ				0
219000 RHR/LPCI: Torus/Pool Cooling Mode		·				0 6			ur fasiskille			Suppression pool	3.7	1
223001 Primary CTMT and Aux.										Γ				0
226001 RHR/LPCI: CTMT Spray Mode	Γ	Γ			Γ						01. 20	Ability to execute procedure steps.	4.3	1
230000 RHR/LPCI: Torus/Pool Spray Mode		Γ	Γ		Γ					Γ				0
233000 Fuel Pool Cooling/Cleanup		Γ			Γ		Γ			Γ				0
234000 Fuel Handling Equipment														0
239001 Main and Reheat Steam														0
239003 MSIV Leakage Control														0
241000 Reactor/Turbine Pressure Regulator			1	Γ	Γ	[0 8	835 2005 2825	10000	Γ		Control/governor valve position	3.3	1
245000 Main Turbine Gen. / Aux.				Γ		Γ		1641		Γ	1232			0
256000 Reactor Condensate		Γ					Γ			Γ				0
259001 Reactor Feedwater	Γ					Γ		0 7		Γ		Reactor water level control system malfunctions	3.7	1
 268000 Radwaste			Γ							Γ	996-306			0
271000 Offgas			ſ	T		F				Γ				0
272000 Radiation Monitoring	T	T	T	T	Γ	T	T	A CONTRACTOR		0		Recorder indications	2.9	1
286000 Fire Protection	ſ	\square	ſ	Γ		Γ	T	10 10 10 10 10 10 10 10 10 10 10 10 10 1		ŕ				0
288000 Plant Ventilation	\square		\uparrow	ſ	T	T	T	W. A.		Γ				0
290001 Secondary CTMT	0	t	T		T	T	T	A CONTRACT OF		T		Primary containment system: Plant-Specific	3.4	1
290003 Control Room HVAC	Ē	T	1	1-	Γ	T	┢		0	Г		Initiation/reconfiguration	3.3	1
290002 Reactor Vessel Internals	\vdash	\vdash	┢	\square	t	${}^{\uparrow}$	t			\vdash	9-03			0
K/A Category Totals:	1	0	1	2	1	2	1	1	1		1	Group Point Total:	<u></u>	12

Generic Knowledge and Abilities Outline (Tier 3)

Facility Nam	e: Da	ate of Exam:				
Category	K/A #	Торіс	IR	0 #	SI	२ <u>०</u>
	2.1. 19	Ability to use plant computer to obtain and evaluate parametric information on system or co		1		
	2.1.21	Ability to obtain and verify controlled procedure copy.	3.1	1		
1.	2.1. 25	Ability to obtain and interpret station reference materials such as graphs, monographs, and	2.8	1		
Conduct of Operations	2.1.					
	2.1.					
	2.1.					
	Subtota			3		
	2.2. 22	Knowledge of limiting conditions for operations and safety limits.	3.4	1		
	2.2. 27	Knowledge of the refueling process.	2.6	1		
2.	2.2.				1	
Equipment Control	2.2.					
	2.2.					
	2.2.					
	Subtota			2		
	2.3. 04	Knowledge of radiation exposure limits and contamination control, including permissible lev	2.5	1		
	2.3. 09	Knowledge of the process for performing a containment purge.	2.5	1		
3.	2.3.					
Radiation Control	2.3.					-
	2.3.					
	2.3.					
	Subtota			2		
	2.4. 22	Knowledge of the bases for prioritizing safety functions during abnormal/emergency operat	3	1		
	2.4. 47	Ability to diagnose and recognize trends in an accurate and timely manner utilizing the app	3.4	1		
4. Emorgonov	2.4. 46	Ability to verify that the alarms are consistent with the plant conditions.	3.5	1		
Emergency Procedures	2.4.					
/ Plan	2.4.					
	2.4.					
	Subtota		•	3		
Tier 3 Point	Total			10		

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ES-401 Emei	rgenc	y and					ation Outline volutions - Tier 1/Group 1 (SRO)	Form E	S-401-
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2	G		IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4				- · ·			·		0
295003 Partial or Complete Loss of AC / 6					0 2		Reactor power, pressure, and level	4.3	1
295004 Partial or Total Loss of DC Pwr / 6									0
295005 Main Turbine Generator Trip / 3								_	0
295006 SCRAM / 1	 					1.1 2	Ability to apply technical specifications for a system.	4	1
295016 Control Room Abandonment / 7					0 3		Reactor pressure	4.4	1
295018 Partial or Total Loss of CCW / 8									0
295019 Partial or Total Loss of Inst. Air / 8									0
295021 Loss of Shutdown Cooling / 4					0 1		Reactor water heatup/cooldown rate	3.6	1
295023 Refueling Acc / 8					0 2		Fuel pool level	3.7	1
295024 High Drywell Pressure / 5									0
295025 High Reactor Pressure / 3									0
295026 Suppression Pool High Water Temp. / 5						2.2 3	Ability to track limiting conditions for operations.	3.8	1
295027 High Containment Temperature / 5									0
295028 High Drywell Temperature / 5									0
295030 Low Suppression Pool Wtr Lvl / 5						4.1 6	Knowledge of EOP implementation hierarchy and coordination with other support procedures.	4	1
295031 Reactor Low Water Level / 2									0
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1									0
295038 High Off-site Release Rate / 9									0
600000 Plant Fire On Site / 8									0
K/A Category Totals:	0	0	0	0	4	3	Group Point Total:		7

ES-401				BWF	RExa	mina	tion Outline F	orm E	S-401-1
Emer	genc	y and	d Abn	orma	al Pla	nt Ev	volutions - Tier 1/Group 2 (SRO)		
E/APE # / Name / Safety Function	К 1	К 2	К 3	A 1	A 2,	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									0
295007 High Reactor Pressure / 3									0
295008 High Reactor Water Level / 2					0 1		Reactor water level	3.9	1
295009 Low Reactor Water Level / 2									0
295010 High Drywell Pressure / 5									0
295011 High Containment Temp / 5									0
295012 High Drywell Temperature / 5									0
295013 High Suppression Pool Temp. / 5									0
295014 Inadvertent Reactivity Addition / 1									0
295015 Incomplete SCRAM / 1									0
295017 High Off-site Release Rate / 9									0
295020 Inadvertent Cont. Isolation / 5 & 7					0 1		Drywell/containment pressure	3.7	1
295022 Loss of CRD Pumps / 1						1.20	Abilily to execute procedure steps.	4.2	1
295029 High Suppression Pool Wtr Lvl / 5									0
295032 High Secondary Containment Area Temperature / 5									0
295033 High Secondary Containment Area Radiation Levels / 9									0
295034 Secondary Containment Ventilation High Radiation / 9									0
295035 Secondary Containment High Differential Pressure / 5									0
295036 Secondary Containment High Sump/Area Water Level / 5									0
500000 High CTMT Hydrogen Conc. / 5									0
K/A Category Totals:	0	0	0	0	2	1	Group Point Total:		3

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Form	ES-401	-1

ES-401								B/	NR I	Exa	mina	tion Outline	Form E	S-401-'
			1		<u> </u>		.			ns -	Tier	2/Group 1 (SRO)		
E/APE # / Name / Safety Function	К 1		К 3	К 4		К 6	A 1		A 2 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection														0
205000 Shutdown Cooling Mode												Loss of heat exchanger cooling	3.5	1
206000 HPCI							-	1	88			System isolation: BWR-2, 3, 4	4.1	1
207000 Isolation (Emergency) Condenser														0
209001 LPCS														0
209002 HPCS														0
211000 SLC								J.C			4.0 6	Knowledge symptom based EOP mitigation strategies.	4	1
212000 RPS														0
215003 IRM														0
215004 Source Range Monitor							Γ							0
215005 APRM / LPRM														0
217000 RCIC														0
218000 ADS							Γ	03	2223			Loss of air supply to ADS valves: Plant-Specific	3.6	1
223002 PCIS/Nuclear Steam Supply Shutoff														0
239002 SRVs														0
259002 Reactor Water Level Control														0
261000 SGTS														0
262001 AC Electrical Distribution														0
262002 UPS (AC/DC)									and					0
263000 DC Electrical Distribution														0
264000 EDGs											2.2 4	Ability to analyze the affect of maintenance activities on LCC	3.8	1
300000 Instrument Air							ſ							0
400000 Component Cooling Water														0
K/A Category Totals:	0	0	0	0	0	0	0) 3	0	0	2	Group Point Total:	•	5

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ES-401						PI	ant						tion Outline F 2/Group 2 (SRO)	orm E	S-401-
E/APE # / Name / Safety Function	к 1		К 3			-		A		Ţ	A 4	G	K/A Topic(s)	IR	#
201001 CRD Hydraulic	†	Ĩ	Ť	† ·	Ť	Ť	Ť	-		T	·				0
201002 RMCS				╞	\uparrow		T			ϯ					0
201003 Control Rod and Drive Mechanism	1			<u> </u>		┢				╈		<u></u>			0
201004 RSCS				┢╴		†	\square			T					0
201005 RCIS							1			1					0
201006 RWM				ſ		F	\uparrow			ϯ			· · · · ·		0
202001 Recirculation							┢			╈					0
202002 Recirculation Flow Control	<u> </u>					$\left \right $				+-					0
204000 RWCU			-				\uparrow	.0		\dagger			Loss of component cooling water	3.4	1
214000 RPIS			T	\square						\dagger	-				0
215001 Traversing In-core Probe			\vdash				1			\uparrow					0
215002 RBM				ŀ		[T			╞					0
216000 Nuclear Boiler Inst.			F				1						Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.	4	1
219000 RHR/LPCI: Torus/Pool Cooling Mode			F				┢╴			Ť		2			0
223001 Primary CTMT and Aux.		Γ			\square		1			T					0
226001 RHR/LPCI: CTMT Spray Mode							┢			t			-		0
230000 RHR/LPCI: Torus/Pool Spray Mode		\square		T			┢			t					0
233000 Fuel Pool Cooling/Cleanup			F	Γ	Γ		\uparrow			T	-		Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure.	3.3	1
234000 Fuel Handling Equipment										ŀ		Ţ,			0
239001 Main and Reheat Steam											T		·		0
239003 MSIV Leakage Control			\square	T	Γ	┢	┢								0
241000 Reactor/Turbine Pressure Regulator					Γ					T					0
245000 Main Turbine Gen. / Aux.										1					0
256000 Reactor Condensate							T			1					0
259001 Reactor Feedwater	Γ						T					8			0
268000 Radwaste	Γ									+					0
271000 Offgas	T					\vdash	1			T					0
272000 Radiation Monitoring	1				F	F				t	1				0
286000 Fire Protection	ſ	F	\square	\square	[+-			t	NOR ROOM				0
288000 Plant Ventilation	T	F	F	 	F	F	T		T	T					0
290001 Secondary CTMT	T	F	 		T		ſ			T					0
290003 Control Room HVAC	T	F	F	[t					0
290002 Reactor Vessel Internals		Γ		\square			ſ			t					0
K/A Category Totals:	0	0	0	0	0	0	0	1	0		$\frac{1}{1}$	2	Group Point Total:		3

ES-401		Generic Knowledge and Abilities Outline (Tier 3)			orm ES-	401-
Facility Nam	e: Da	ite of Exam:				
Category	K/A #	Торіс		20 #	SRO- IR	<u>Only</u> #
	2.1. 14	Knowledge of system status criteria which require the notification of plant personnel.			3.3	1
	2.1.06	Ability to supervise and assume a management role during plant transients and upset cond	titions.		4.3	1
1.	2.1.					
Conduct of Operations	2.1.		<u> </u>			
oporatione	2.1.					<u>, , , , , , , , , , , , , , , , , , , </u>
	2.1.					
	Subtotal			and the		2
2. Equipment Control	2.2. 17	Knowledge of the process for managing maintenance activities during power operations.			3.5	1
	2.2. 32	Knowledge of the effects of allerations on core configuration.			3.3	1
	2.2.					
	2.2.					
	2.2.					
	2.2.					
	Subtota					2
	2.3. 1	Knowledge of 10 CFR 20 and related facility radiation control requirements.			3	1
	2.3.					
3.	2.3.					
Radiation Control	2.3.					
	2.3.					
	2.3.					
<u>_</u>	Subtota			0		1
	2.4. 1	Knowledge of EOP entry conditions and immediate action steps.			4.6	1
	2.4.45	Ability to prioritize and interpret the significance of each annunciator or alarm.	 		3.6	1
4. Emergency	2.4.					
Procedures	2.4.					_
/ Plan	2.4.					
	2.4.					
	Subtota	I		0		2

Record of Rejected K/As

	Tier / Group	Randomly Selected K/A	Reason for Rejection
	SRO 1 / 1	295024EA2.02	K/A 295020AA2.01 (Existing K/A) and K/A29524EA2.02 (Replaced K/A) test very similar topic areas. Writing questions on both very similar K/As will result in double jeopardy for both questions. K/A 295023AA2.02 was randomly selected to replace K/A 295024EA2.02
	SRO 2 / 1	263000A2.01	After many attempts to write an exam question on this K/A, the exam team was unable to create a question that would satisfy the K/A requirements. K/A 206000A2.10 was randomly selected to replace K/A 263000A2.01
	RO 1/1	600000 A2.17	Could not write an adequate question pertaining to this K/A without it being a power supply question. Also attempted to select another K/A from category 600000, but could not write an adequate question for those either. K/A 295005 A2.07 was randomly selected to replace K/A 600000 A2.17.
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/			