

January 26, 2005

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U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
License Nos. DPR-24 and DPR-27

Response to Request for Additional Information
Regarding the Point Beach Nuclear Plant License Renewal Application
(TAC Nos. MC2099 and MC2100)

By letter dated February 25, 2004, Nuclear Management Company, LLC (NMC), submitted the Point Beach Nuclear Plant (PBNP) Units 1 and 2 License Renewal Application (LRA). On January 10, 2005, the Nuclear Regulatory Commission (NRC) requested additional information regarding the Feedwater and Condensate System (Section 2.3.4.2 of the LRA). The enclosure to this letter contains NMC's response to the staff's questions.

Should you have any questions concerning this submittal, please contact Mr. James E. Knorr at (920) 755-6863.

This letter contains no new commitments and no revisions to existing commitments.

I declare under penalty of perjury that the forgoing is true and correct. Executed on January 26, 2005.



Dennis L. Koehl
Site Vice-President, Point Beach Nuclear Plant
Nuclear Management Company, LLC

Enclosure

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cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

ENCLOSURE

RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION REGARDING POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 LICENSE RENEWAL APPLICATION

The following information is provided in response to the Nuclear Regulatory Commission (NRC) staff's request for additional information (RAI) regarding the Point Beach Nuclear Plant (PBNP) License Renewal Application (LRA).

The NRC staff's question is restated below, with the Nuclear Management Company (NMC) response following.

Section 2.3.4 Steam and Power Conversion Systems

2.3.4.2 Feedwater and Condensate System Request for Additional Information (RAI)

RAI 2.3.4.2-4

In a letter dated November 16, 2004, the Nuclear Regulatory Commission (NRC) requested additional information regarding the Auxiliary Systems (LRA Section 2.3.3) and Steam and other Balance of Plant (BOP) Systems in LRA Sections 2.3.3 and 2.3.4 respectively. PBNP response to this request was submitted to the NRC in Nuclear Management Company, LLC letter dated December 22, 2004. The NRC review of the PBNP response to RAI 2.3.4.2-1 identified two follow-up questions regarding the scoping for the small bore branch piping from the NSR portion of the 16" main feed water header (FW) between the feed regulating valve and the downstream steam generator FW inlet check valve:

1. Recognizing the NRC would not require a formal HELB evaluation be performed on piping 1-inch and less, did PBNP perform any evaluation (i.e., walkdown, etc.) to confirm that a break in the branch piping would not impact any safety related equipment in the immediate vicinity of the possible break location?
2. Please discuss flooding associated with a failure in the branch piping and its impact on safety related equipment.

NMC Response:

1) The section of main feedwater piping in question was included in scope to protect the safety-related (SR) (but non-Environmentally Qualified) feedwater flow transmitters (1/2FT-466, -467, -476, -477). Walkdowns were performed to verify that no high energy branch piping (including branch piping one-inch and less) was in proximity to the SR

transmitters where pipe whip or jet impingement could be a hazard. Additionally, the transmitters are in large open areas where the energy input from a failure of a small bore line off of this header would not create a sufficiently harsh environment to affect the function of these transmitters.

2) The SR feedwater flow transmitters in question are located on the 26' elevation for Unit 2 and the 39' elevation for Unit 1, and mounted about four feet off of the floor. Any leakage from the main feedwater header branch connections at these elevations would cascade to lower elevations, and not affect the function of these transmitters nor any other safety-related equipment due to flooding.