TXU Power Comanche Peak Steam Electric Station P. O. Box 1002 (E01) Glen Rose, TX 76043 Tel: 254 897 5209 Fax: 254 897 6652 mike.blevins@txu.com Mike Blevins Senior Vice President & Chiel Nuclear Officer

Ref: 10CFR50.73(a)(2)(i)(B)

CPSES-200500005 Log # TXX-05002

January 21, 2005

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES) DOCKET NO. 50-446 CONDITION PROHIBITED BY TECHNICAL SPECIFICATIONS LICENSEE EVENT REPORT 446/04-003-00

Gentlemen:

Enclosed is Licensee Event Report (LER) 04-003-00 for Comanche Peak Steam Electric Station Unit 2, "Containment Pressure Channel Inoperable Due to a Secondary Ground."

This communication contains no new licensing basis commitments regarding CPSES Units 1 and 2.

A member of the STARS (Strategic Tearning and Resource Sharing) Alliance

Callaway • Comanche Peak • Diablo Canyon • Palo Verde • South Texas Project •

Eðo

Wolf Creek



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Sincerely,

TXU Generation Company LP

By: TXU Generation Management Company LLC, Its General Partner

Mike Blevins

L. By: Fred W. Madden

Director, Regulatory Affairs

GLM Enclosures

c - B. S. Mallett, Region IV
W. D. Johnson, Region IV
M. C. Thadani, NRR
Resident Inspectors, CPSES

| LICENSEE EVENT REPORT (LER) |           |            |  |                                   |                     |         |                   | ON   | All<br>Es<br>Re<br>inc<br>Se<br>00<br>Ini<br>and<br>do | PPROVE<br>stimated le<br>cported le<br>dustry. S<br>ervice Bra<br>001, or by<br>formation<br>ad Budget<br>bes not dis<br>consor, an | ED BY OMB N<br>burden per resp<br>ssons learned a<br>end comments<br>inch (T-5 F52),<br>internet e-mail<br>a and Regulator<br>Washington, '<br>splay a currentil<br>d a person is no | iO. 3150-0104<br>onse to comply w<br>re incorporated in<br>regarding burden<br>U.S. Nuclear Re<br>to infocollects@<br>y Affairs, NEOB<br>DC 20503. If a m<br>y valid OMB con<br>ot required to resp | vith this :<br>nto the li<br>estimat<br>gulatory<br>f.nrc.gov<br>-10202 (<br>neans us<br>ntrol num<br>pond to, | mandatory coll<br>icensing process<br>e to the Record<br>Commission,<br>, and to the De<br>(3150-0104), C<br>ed to impose a<br>iber, the NRC<br>the information | EXPIRES<br>ection reque<br>ss and fed b<br>is and FOIA<br>Washington<br>sk Officer,<br>Mfice of Ma<br>in information<br>may not con<br>a collection. | 6 06/30/2007<br>est: 50 hours.<br>ack to<br>/Privacy<br>, DC 20555-<br>Office of<br>nagement<br>on collection<br>aduct or |                              |                    |          |   |
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| СОМА                        | ANCH      | E PEA      | K STEA   | M E                               | ELECTRI             | CS      | TAI               | <b>FION</b>  | UNIT   | 2   |  |   | 0  | )500(   | )446   |   | - 1                          | 1 OF (             | 6        |   |
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| Power                       | L         | <u> </u>   | 20.220   | (b)                               |                     |         | ╉╼┨               | 20.22  | 203(a)(3)  | (1)   |  |   | -  | 50.73   | $\frac{(a)(2)(1)(C)}{(a)(2)(1)(A)}$  |   | ╂─╂                          | 50.73(a)(2         |          | <u>,                                     </u> |
| Level                       | 10        | n F        | 20.2201(d)   |                                   |                     |         | 20.2203(a)(3)(11) |  |  | 50.73(a)(2)(i)(A) 50  |  |   | 50.73(a)(2   | 50.73(a)(2)(viii)(R)  |  |   |                              |                    |          |   |
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| Name                        |           |            |  |                                   |                     |         |                   |  | Licensee (   | omact Fo  | n ti   | as LEK (1   | <b>4)</b>  |   | <u> </u>   | Telephone Numb  | er (loch                     | de Area Code       | )        |   |
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On November 23, 2004, Comanche Peak Steam Electric Station Unit 2 was in Mode 1, Power Operation, operating at approximately 100 percent power. On November 23, Engineering personnel determined that one of the four Engineered Safety Feature Actuation System (ESFAS) intermediate range containment pressure channels had been inoperable for a period of time longer than allowed by the Technical Specifications.

TXU Generation Company LP (TXU Power) believes that this event was caused by a secondary ground in the junction box associated with the pressure transmitter. Corrective actions include cleaning and rerouting the affected cable shield wire and identification and inspection of any other potential similar conditions.

All times in this report are approximate and Central Standard Time unless noted otherwise.

### Enclosure to TXX-05002

| NRC FORM 366A<br>(1-2001)                   |          | U.S. NUCLEAR REGULATORY COMMISSION        |     |
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| LICENSEE EVEN                               | Г REPORI | Г (LER)                                   |     |
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| Facility Name (1)                           | Docket   | LER Number (6) Page                       | (3) |
| COMANCHE PEAK STEAM ELECTRIC STATION UNIT 2 |          | Year Sequential Revision<br>Number Number |     |

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

## I. DESCRIPTION OF REPORTABLE EVENT

## A. REPORTABLE EVENT CLASSIFICATION

Any operation or condition prohibited by the plant's Technical Specifications.

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## **B.** PLANT OPERATING CONDITIONS PRIOR TO THE EVENT

On November 23, 2004, at the time of discovery for this reportable event, Comanche Peak Steam Electric Station (CPSES) Unit 2 was in Mode 1, Power Operation, operating at approximately 100 percent power.

# C. STATUS OF STRUCTURES, SYSTEMS, OR COMPONENTS THAT WERE INOPERABLE AT THE START OF THE EVENT AND THAT CONTRIBUTED TO THE EVENT

There were no inoperable structures, systems, or components that contributed to the event.

# D. NARRATIVE SUMMARY OF THE EVENT, INCLUDING DATES AND APPROXIMATE TIMES

On August 5, 2004, CPSES Unit 2 was in Mode 1, Power Operation, operating at approximately 100 percent power. At 0755 hours, Operators in the Unit 2 Control Room received a computer point deviation alarm on one of the four Engineered Safety Feature Actuation System (ESFAS) intermediate range containment pressure channels (channel 2-P-0935) [EIIS:(JE)(CHA)]. At that time, the channel was found to be reading approximately 2.0 psig low with respect to the other channels. Over the next 25 hours, channel 2-P-0935 acted erratically and failed to track the other three containment pressure channels. The channel deviations during this time ranged from approximately 5.0 psig low to approximately 1.0 psig low.

| Enclosure to TXX-05002  | 1  |   |  |   |   |   |
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| NRC FORM 366A<br>(1-201)  |  | U.S   | S. NUCLEAR R   | EGULA   | TORY COMM   | AISSION   |
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| COMANCHE PEAK STEAM ELECTRIC STATION UNIT 2   | 05000446   | Year 1  | Sequential<br>Number<br>003  | <b>X</b>  | Revision<br>Number  | 3 OF 6  |
| NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)  | 1  | <u> </u>  | 1  |   |   |   |
| On August 6, 2004, at 0900 hour<br>troubleshooting the deviation pro<br>Engineering suggested that a Cha<br>ascertain the channel's operabilit<br>points for the High 1, High 2, an<br>calibration range. However, upo<br>2-P-0935 did not return to a norm<br>containment pressure channels.<br>Technical Specification (TS) 3.3<br>that an inoperable channel be pla<br>affected Unit must be in Mode 3<br>Troubleshooting activities by Ma<br>the discovery of a grounded shie<br>with the pressure transmitter [EII<br>was corrected and the loop was r<br>pressure indications.<br>On August 10, 2004, Engineerin<br>0935 had been inoperable while<br>November 23, 2004, Engineering<br>channel 2-P-0935 had been inoper<br>August 6, 2004, for a total of 31<br>completion time of 12 hours to b<br>Technical Specifications occurre | s Engineering<br>oblem with cha<br>annel Operatio<br>y. The COT e<br>d High 3 activa<br>n restoration o<br>nal reading wh<br>At 1030 hours<br>2. Condition I<br>ced in bypass<br>within 12 hou<br>intenance pers<br>d wire conditi<br>(S:(JE)(CHA)(<br>estored to serv<br>g began an eva<br>the grounded s<br>g completed th<br>erable from 07<br>hours and 11 r<br>e in Mode 3. 7<br>d. | was con<br>unnel 2-F<br>nal Test<br>stablishe<br>ations w<br>of the loc<br>ien comp<br>, the cha<br>D was er<br>or trip w<br>rs and M<br>sonnel or<br>on in the<br>(PT)]. A<br>vice, with<br>aluation the<br>shield wi<br>e evalua<br>55 on A<br>ninutes.<br>Therefor | tacted fc<br>-0935.<br>(COT) b<br>ed that the<br>ere well<br>op from t<br>bared to b<br>annel was<br>nel was<br>nel was<br>nel was<br>intered. T<br>vithin 6 h<br>fode 4 w<br>n August<br>t 1506 h<br>n normal<br>to determine<br>tre condi-<br>tion and<br>ugust 5,<br>This ex-<br>re, a repo | or as:<br>At the period of the constant of the co | sistance<br>hat time,<br>erformed<br>stable se<br>in the<br>COT, cha<br>other<br>in bypa<br>TS requ<br>s or the<br>18 hou<br>2004 lea<br>x associ<br>this con<br>tainmen<br>if chann<br>existed.<br>ermined<br>4 to 150<br>led the T<br>le violat | in<br>d to<br>et<br>annel<br>ass and<br>ires<br>ars.<br>d to<br>ated<br>addition<br>at<br>hel 2-P-<br>On<br>that<br>6 on<br>TS<br>tion of |
| E. THE METHOD OF DISCOVE<br>FAILURE, OR PROCEDURA   | L OR PERSC   | H COM   | ERROF  | 81. C   | JR SYS  | TEM   |
| Engineering personnel (utility, n<br>been inoperable for longer than a  | on-licensed) de<br>llowed by TS.   | etermine  | ed that ch   | ann   | el 2-P-0  | 935 had   |
|   |  |   |  |   |   |   |
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#### Enclosure to TXX-05002 NRC FORM 366A (1-2001)

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U.S. NUCLEAR REGULATORY COMMISSION

# LICENSEE EVENT REPORT (LER)

Facility Name (1)

COMANCHE PEAK STEAM ELECTRIC STATION UNIT 2

| Docket   |      | Page(3) |                      |   |                    |        |
|----------|------|---------|----------------------|---|--------------------|--------|
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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

# II. COMPONENT OR SYSTEM FAILURES

# A. FAILURE MODE, MECHANISM, AND EFFECTS OF EACH FAILED COMPONENT

Not applicable – No component or system failures were identified during this event.

# B. CAUSE OF EACH COMPONENT OR SYSTEM FAILURE

Not applicable – No component or system failures were identified during this event.

# C. SYSTEMS OR SECONDARY FUNCTIONS THAT WERE AFFECTED BY FAILURE OF COMPONENTS WITH MULTIPLE FUNCTIONS

Not applicable – No component or system failures were identified during this event.

# D. FAILED COMPONENT INFORMATION

Not applicable – No component or system failures were identified during this event.

## **III. ANALYSIS OF THE EVENT**

# A. SAFETY SYSTEM RESPONSES THAT OCCURRED

Not applicable – No safety system responses occurred as a result of this event.

# B. DURATION OF SAFETY SYSTEM TRAIN INOPERABILITY

Engineering determined that intermediate range containment pressure channel 2-P-0935 was inoperable from 0755 hours on August 5, 2004, until 1506 hours on August 6, 2004 (a total of 31 hours and 11 minutes).

#### Enclosure to TXX-05002 NRC FORM 366A (1-2001)

| U.S. NUCLEAR | REGULATORY | COMMISSION |
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# LICENSEE EVENT REPORT (LER)

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| COMANCHE PEAK STEAM FLECTRIC STATION UNIT?   | 05000446 | Year |                    | Sequential<br>Number |        | Revision<br>Number |         |
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| NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17) |          |      |                    |                      |        |                    |         |

# C. SAFETY CONSEQUENCES AND IMPLICATIONS

The containment pressure sensing channels are necessary for the Safety Injection (SI), Main Steam Line Isolation (MSLI), and Containment Spray (CT) ESFAS functions. For the SI and MSLI functions, three of the channels are used with a 2 out of 3 logic. For the CT function, four channels are employed with a 2 out of 4 logic. The additional redundancy is added for system reliability due to the CT function being an "energize to trip" design.

During the 31 hours and 11 minutes that channel 2-P-0935 was inoperable, no event occurred that required the SI or MSLI actuations on high containment pressure. Had the SI and MSLI actuations been necessary, it is highly likely that the 2 out of 2 logic would have been satisfied. Even though the trip function is not single failure proof in this configuration, these channels are highly reliable with little history of failure. In addition, the SI or MSLI actuations could have been initiated from other diverse indications, such as low pressurizer pressure or low compensated steam line pressure.

If a parameter is used only for input to the protection circuits, three channels with a 2 out of 3 logic are sufficient to provide the required reliability and redundancy. If one channel fails in a direction that would not result in a partial function trip, the function is still operable with a 2 out of 2 logic. Therefore, the CT function would still have been satisfied with a 2 out of 3 logic, including the potential for a single failure of one channel.

As discussed above, no event requiring a SI, MSLI, or CT actuation actually occurred. The SI and MSLI functions are backed up by other diverse indications that would have initiated equivalent mitigative functions. Finally, it is highly likely that any required SI, MSLI, and CT actuations would still have occurred, even with the inoperable channel. Therefore, no safety system functional failures occurred.

Based on this analysis it was concluded that this event did not adversely affect the safe operation of CPSES Unit 2 or the health and safety of the public.

# Enclosure to TXX-05002

See See

**U.S. NUCLEAR REGULATORY COMMISSION** 

# LICENSEE EVENT REPORT (LER)

Facility Name (1)

(1-2001)

COMANCHE PEAK STEAM ELECTRIC STATION UNIT 2

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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

# IV. CAUSE OF THE EVENT

TXU Power believes that this event was caused by a secondary ground in the junction box associated with the pressure transmitter. The expansion anchors, securing the junction box to the reactor building wall, were found to be exuding a white, mineral-rich liquid from the concrete. This liquid had over time dripped on the cable shield at the bottom of the junction box. The built up mineral deposits produced a current path to ground, essentially causing the shield for one of the cables to be grounded at both ends. This caused a grounding loop, which lead to the intermittent erroneous indications.

# V. CORRECTIVE ACTIONS

This condition was corrected and the loop was restored to service, with normal containment pressure indications. Per the CPSES Corrective Action Program, Engineering will review other potential locations where this condition may exist and inspect those locations for the existence of this condition as appropriate.

# VI. PREVIOUS SIMILAR EVENTS

There has been one other reportable event in the last three years involving channel 2-P-0935 being inoperable (see Unit 2 LER 2003-004-00). However, the cause of that event (random electronic failure) was sufficiently different from the event described in this LER such that the previous corrective actions could not have prevented this event.