R. A. Saccone Vice President - Nuclear Operations

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U.S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Stop O-P1-17 Washington, DC 20555-0001

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 50-387/2005-001-00 PLA-5858

Docket 50-387

Attached is Licensee Event Report 50-387/2005-001-00. While analysis of the situation is not yet completed, there is a potential that six primary containment instrument lines found penetrating the Unit 1 Reactor Building's Railroad Bay could prevent structures or systems needed to control the release of radioactive material from fulfilling their safety function. Although the Railroad Bay can be aligned to Secondary Containment, it has not normally been placed in this configuration. The instrument line location is not consistent with station licensing documents that require containment of the lines from a dose analysis and a Secondary Containment design configuration standpoint. Accordingly, this event is being reported in accordance with 10 CFR 50.73(a)(2)(v)(C). There were no actual consequences to the health and safety of the public as a result of this event.

No new regulatory commitments have been created through issuance of this report.

Robert A. Saccone

Vice President - Nuclear Operations

Attachment

JE22

cc: Mr. S. J. Collins
Regional Administrator
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19408

Mr. A. J. Blamey Sr. Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 35 Berwick, PA 18603-0035

Mr. R. Osborne Allegheny Electric Cooperative P. O. Box 1266 Harrisburg, PA 17108-1266

Mr. R. R. Janati Bureau of Radiation Protection Rachel Carson State Office Building P. O. Box 8469 Harrisburg, PA 17105-8469

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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO

On 11/22/2004, it was identified that six Primary Containment instrument lines penetrated the Unit 1 Reactor Building's Railroad Bay. The Railroad Bay door serves as an access/egress point for equipment and materials to and from the Unit 1 Reactor Building. Although the area can be aligned to Secondary Containment, it has not normally been maintained in this configuration. The instrument lines that are located in the Railroad Bay, and have been typically located outside Secondary Containment due to ventilation system alignment practices, are not consistent with assumptions currently found in station licensing documents. The principal causes for the non-compliance can be traced to past licensing basis changes that credited Secondary Containment in the accident analysis and removed normal alignment of the Railroad Bay to Secondary Containment Zone III. In response to this situation, administrative controls have been enacted that align the Railroad Bay to the Secondary Containment Zone III when the Railroad Bay door is not open. Additional actions are planned to evaluate and resolve those shortcomings, as stated above, that have contributed to the non-compliance. Preliminary analysis Indicates that postulated leakage from the lines in question is within the margin available in the LOCA dose analysis for Secondary Containment Bypass Leakage. Further, the analysis indicates that the instrument lines, if cracked or broken, will not introduce radiological consequences that exceed those presented in the Safety Analysis Report for a break outside of Secondary Containment. Because these are only preliminary assessments, this event is being reported in accordance with 10 CFR 50.73(a)(2)(v)(C) as a condition that could have prevented structures or systems needed to control the release of radioactive material from fulfilling their safety function.

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DATE

2005

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET			3. PAGE		
Susquehanna Steam Electric Station – Unit 1	05000387	YEAR SEQUENTIAL NUMBER		REVISION NUMBER	2 OF 3	
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17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

EVENT DESCRIPTION

On 11/22/2004, it was identified that six Primary Containment (EIIS Code: NH) instrument lines penetrated the Unit 1 Reactor Building's Railroad Bay (EIIS Code: NG). Two of the instrument lines are associated with the Unit 1 Reactor Recirculation system (EIIS Code: AD) while the remaining four are associated with the Unit 1 Residual Heat Removal system (EIIS Code: BO). These instrument lines have been located in the Railroad Bay since plant startup. Although the area can be aligned to Secondary Containment (EIIS Code: NH), it has not normally been configured as such since 1995. The original Safety Analysis Review (SAR) did not rely upon instrument lines being located within Secondary Containment. The SAR was, however, changed in 1998 to credit Secondary Containment in the SAR accident analysis. The instrument lines that are located in the Railroad Bay, and are thus typically located outside Secondary Containment, are not then consistent with assumptions currently found in station licensing documents.

The non-dompliance described above is being reported in accordance with 10 CFR 50.73(a)(2)(v)(C) as a condition that could have prevented structures or systems needed to control the release of radioactive material from fulfilling their safety function. No radioactive releases resulted from this event.

CAUSE OF EVENT

Preliminary results pending completion of the evaluation under the Corrective Action Process:

The principal causes for the current non-compliance with the licensing basis were the changes made to the accident analysis that credited Secondary Containment and the change in 1995 that established normal alignment of the Railroad Bay outside Secondary Containment.

ANALYSIS / SAFETY SIGNIFICANCE

Preliminary analysis of Secondary Containment bypass leakage has concluded that the postulated leakage from these lines is within the margin available in the LOCA dose analysis for Secondary Containment Bypass Leakage. Further, the analysis indicates that the instrument lines, if cracked or broken, will not introduce radiological consequences that exceed those presented in the SAR for a break outside of Secondary Containment. Because these are only preliminary assessments, this event is being reported in accordance with 10 CFR 50.73(a)(2)(v)(C) as a condition that could have prevented structures or systems needed to control the release of radioactive material from fulfilling their safety function.

NRC FORM 366A (1-2001)

(1-2001)

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	YEAR SEQUENTIAL		6. LER NUMBER		3. PAGE		
Susquehanna Steam Electric Station – Unit 1				REVISION NUMBER	3 OF 3		
		00	3013				

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

CORRECTIVE ACTIONS

The following corrective actions have been completed:

 Administrative controls have been enacted to align the Railroad Bay to the Secondary Containment Zone III when the Railroad Bay door is not open.

The following corrective actions are planned:

• This non-conformance has been entered into the Corrective Action program for evaluation and resolution.

ADDITIONAL INFORMATION

None

NRC FORM 366A (1-2001)