

January 31, 2005

Mr. Joseph M. Solymossy
Site Vice President
Prairie Island Nuclear Generating Plant
Nuclear Management Company, LLC
1717 Wakonade Drive East
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT (PRAIRIE ISLAND),
UNITS 1 AND 2 - REQUEST FOR ADDITIONAL INFORMATION (RAI) -
LICENSE AMENDMENT REQUEST (LAR) FOR RESOLUTION OF
UNRESOLVED ITEMS RELATED TO METHODS FOR EVALUATION OF
COOLING WATER SYSTEM (TAC NOS. MC3043 AND MC3044)

Dear Mr. Solymossy:

For the above referenced LAR, the initial RAI was sent to you via e-mail on 12/8/04. On January 12, 2005, a follow-up telephone conference was held between Prairie Island representatives and the U. S. Nuclear Regulatory Commission (NRC) staff, during which the initial RAI was discussed in more detail. As a result of this discussion, your representatives agreed to provide response to the enclosed NRC staff questions by February 11, 2005. If you have any further questions, please feel free to contact me at 301-415-8371.

Sincerely,

/RA/

Mahesh L. Chawla, Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosure: Request For Additional Information

cc w/encls: See next page

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OFFICE	PDIII-1/PM	PDIII-1/LA	SPLB/SC	IROB/SC	EMEB/SC	PDIII-1/SC
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DATE	01/27/05	01/27/05	01/27/05	01/31/05	01/28/05	01/31/05

REQUEST FOR ADDITIONAL INFORMATION
RELATED TO METHODS FOR EVALUATION OF COOLING WATER SYSTEM
PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT 1 AND 2

- 1) You indicated that the evaluation applies acceptance criteria from American Society of Mechanical Engineers (ASME) Section III, Subsection ND, Service Level D when performing stress analyses of the cooling water (CL) system non-Class I piping with design-basis seismic loads. The proposed methodology and acceptance criteria provide a conservative approach for demonstrating that the CL system will perform its safety function following a design-basis seismic event. Provide a description of analysis including the methodology, loss-of-coolant accident loads for the faulted condition, load combinations, input response spectra to the seismic analysis, modeling and computer code used, assumptions and resulting stresses in comparison against the code allowable limits.
- 2) Though Nuclear Regulatory Commission (NRC) is reviewing the criteria for analyzing CL system non-seismic piping, confirm that the piping will continue to be protected from the failure of non-seismic structures, systems and components in accordance with the existing plant licensing basis. Please explain any exceptions.
- 3) The licensee proposes to credit automatic isolation valves for excluding CL system non-seismic piping from consideration. Confirm that these valves are seismic Category I, safety-related valves, which will automatically actuate to close during a seismic event, and that the single-failure criterion will be satisfied.
- 4) Explain in detail specific operator actions that are being relied upon for postulated pipe break conditions for NRC assessment of human factors considerations.

ENCLOSURE

Prairie Island Nuclear Generating Plant,
Units 1 and 2

cc:

Jonathan Rogoff, Esquire
Vice President, Counsel & Secretary
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