

February 28, 2005

Mr. R. T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station, FC-2-4 Adm.
P.O. Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – REQUEST FOR RELIEF FROM
ASME CODE REPAIR REQUIREMENTS AND USING AN ALTERNATIVE FOR
THE PRESSURIZER NOZZLE REPAIR (TAC NO. MC0196)

Dear Mr. Ridenoure:

By letter dated July 25, 2003, Omaha Public Power District (OPPD/the licensee) requested relief from American Society of Mechanical Engineers (ASME) Code repair requirements for its TE-108 pressurizer temperature nozzle weld repair for the Fort Calhoun Station, Unit 1 (FCS). In response to our request for additional information dated March 16, 2004, the licensee provided additional information on April 29, 2004. After discussions with the staff, OPPD provided a revised response dated September 30, 2004. By letter dated February 23, 2005, OPPD revised its request to reflect that the relief is only needed until the pressurizer is replaced in the fall of 2006. OPPD has cited Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(a)(3)(ii) as the basis for requesting relief. As an alternative to an ASME Code repair, OPPD has proposed to use a flaw evaluation to justify leaving the flaw in the J-groove weld. This flaw evaluation is based on the acceptance criteria of ASME Code, Section XI, IWB-3612, "Acceptance Criteria Based on Applied Stress Intensity Factor."

The staff has determined that the FCS can be operated until the replacement of the pressurizer in the Fall of 2006 since the calculated applied stress intensity factor for the final flaw, considering six years' crack growth, meets the IWB-3612 criteria for all loading conditions. The licensee has demonstrated that compliance with the ASME Code requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety. Therefore, the licensee's proposed alternative is authorized pursuant to 10 CFR 50.55a(a)(3)(ii) for leaving the flaw in-place in the J-groove weld of the pressurizer.

The staff's evaluation and conclusions are contained in the enclosed safety evaluation (SE). Pursuant to Section 2.390 of 10 CFR, we have determined that the enclosed SE does not contain proprietary information. However, we will delay placing the SE in the public document room for a period of ten working days from the date of this letter to provide you with the opportunity to comment on the proprietary aspects. If you believe that any information in the enclosure is proprietary, please identify such information line-by-line and define the basis pursuant to the criteria of 10 CFR 2.390.

R. Ridenoure

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If you have any questions, please contact Alan Wang at (301) 415-1445. This closes TAC No. MC0196.

Sincerely,

/RA/

Robert A. Gramm, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: Safety Evaluation

cc w/encl: See next page

R. Ridenoure

- 2 -

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Office of Nuclear Reactor Regulation

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cc w/encl: See next page

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ADAMS Accession No.: ML050120357

*** Memo dated**

NRR-028

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