

January 10, 2005

Mr. Dennis L. Koehl
Site Vice President
Nuclear Management Company, LLC
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL
APPLICATION

Dear Mr. Koehl:

By letter dated February 25, 2004, Nuclear Management Company, LLC, (NMC or the applicant) submitted an application pursuant to 10 CFR Part 54, to renew the operating licenses for Point Beach Nuclear Plant (PBNP), Units 1 and 2, for review by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application (LRA) and has identified, in the enclosure, areas where additional information is needed to complete the review.

These RAIs were discussed with your staff, Mr. Jim Knorr, and a mutually agreeable date for this response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-2232 or e-mail MJM2@nrc.gov.

Sincerely,

/RA/

Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosure: As stated

cc w/encls: See next page

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Point Beach Nuclear Plant, Units 1 and 2

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DISTRIBUTION: Ltr. To D. Koehl, RAI Re: Point Beach Nuclear Plant, Units 1 and 2, Dated:
January 10, 2005

Accession no.: **ML050110469**

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POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION (LRA)
REQUEST FOR ADDITIONAL INFORMATION (RAI)

Section 2.3.4 Steam and Power Conversion Systems

2.3.4.2 Feedwater and Condensate System Request for Additional Information (RAI)

RAI 2.3.4.2-4

In a letter dated November 16, 2004, the Nuclear Regulatory Commission'(NRC) requested additional information regarding the Auxiliary Systems (LRA Section 2.3.3) and Steam and other Balance of Plant (BOP) Systems in LRA Sections 2.3.3 and 2.3.4 respectively. PBNP response to this request was submitted to the NRC in Nuclear Management Company, LLC letter dated December 22, 2004. The NRC review of the PBNP response to RAI 2.3.4.2-1 identified two follow-up questions regarding the scoping for the small bore branch piping from the NSR portion of the 16" main feed water header (FW) between the feed regulating valve and the downstream steam generator FW inlet check valve:

1. Recognizing the NRC would not require a formal HELB evaluation be preformed on piping 1-inch and less, did PBNP perform any evaluation (i.e., walkdown, etc.) to confirm that a break in the branch piping would not impact any safety related equipment in the immediate vicinity of the possible break location?
2. Please discuss flooding associated with a failure in the branch piping and its impact on safety related equipment.