

Indiana Michigan  
Power Company  
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December 28, 2004

AEP:NRC:4046-01  
10 CFR 50.46

Docket Nos: 50-315  
50-316

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Mail Stop O-P1-17  
Washington, D. C. 20555-0001

Donald C. Cook Nuclear Plant Units 1 and 2  
10 CFR 50.46 LOSS-OF-COOLANT ACCIDENT REANALYSIS SCHEDULE

- References:
1. Letter from S. A. Greenlee, Indiana Michigan Power Company (I&M), to U. S. Nuclear Regulatory Commission (NRC) Document Control Desk, "Donald C. Cook Nuclear Plant Units 1 and 2, Annual Report of Loss-Of-Coolant Accident Evaluation Model Changes," C0801-19, dated August 31, 2001.
  2. Letter from John A. Zwolinski, I&M, to NRC Document Control Desk, "Donald C. Cook Nuclear Plant Units 1 and 2, Annual Report of Loss-of-Coolant Accident Evaluation Model Changes," AEP:NRC:3046, dated August 27, 2003.
  3. Letter from John A. Zwolinski, I&M, to NRC Document Control Desk, "Donald C. Cook Nuclear Plant Units 1 and 2, Annual Report of Loss-of-Coolant Accident Evaluation Model Changes," AEP:NRC:4046, dated August 26, 2004.

Pursuant to 10 CFR 50.46, Indiana Michigan Power Company (I&M), the licensee for the Donald C. Cook Nuclear Plant (CNP), is submitting a schedule for reanalysis of the Unit 1 limiting small break, the Unit 2 limiting large break, and Unit 2 limiting small break analyses of record. I&M previously submitted a schedule for performing new analyses in Reference 1 because previously reported changes to these analyses were classified as significant. I&M submitted

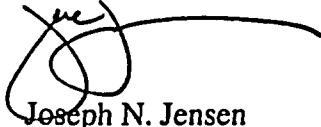
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a revised Unit 2 schedule in Reference 2 to support the transition to a new fuel vendor for CNP. Since development of the revised schedule, I&M has subsequently contracted with a different fuel vendor to perform these services. I&M is evaluating fuel design changes for Unit 1 which will require reanalysis of the loss-of-coolant accidents (LOCA). Unit 2 LOCA reanalysis will follow Unit 1 reanalysis. I&M, therefore, is revising the currently proposed schedule for performing the Unit 1 limiting small break, the Unit 2 limiting large break, and the Unit 2 limiting small break analyses. By Reference 3, I&M committed to provide a new schedule for reanalysis of these events.

I&M plans to meet with NRC staff to discuss the attached schedule in early 2005. This schedule meets the commitment made in Reference 3.

Should you have any questions, please contact Mr. John A. Zwolinski, Director of Safety Assurance, at (269) 466-2428.

Sincerely,



Joseph N. Jensen  
Site Vice President

KS/rdw

Attachment

c: J. L. Caldwell, NRC Region III  
K. D. Curry, Ft. Wayne AEP, w/o attachment  
J. T. King, MPSC, w/o attachment  
C. F. Lyon, NRC Washington, DC  
MDEQ – WHMD/HWRPS  
NRC Resident Inspector

ATTACHMENT TO AEP:NRC:4046-01

COMMITMENTS

The following table identifies those actions committed to by Indiana Michigan Power Company (I&M) in this document. Any other actions discussed in this submittal represent intended or planned actions by I&M. They are described to the Nuclear Regulatory Commission (NRC) for the NRC's information and are not regulatory commitments.

Commitment	Date
A new Unit 1 small break loss-of-coolant accident (SBLOCA) analysis of the safety injection cross-tie valves closed case will be provided.	March 2007
A new Unit 2 SBLOCA analysis of both the safety injection cross-tie valves closed case and the safety injection cross-tie valves open case will be provided.	March 2009
A new Unit 2 large break loss-of-coolant accident analysis will be provided.	March 2009